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Substantiation of boiling water reactors safety in case of large spills coolant S22

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Obosnovanie bezopasnosti korpusnogo kipyashchego reaktora pri krupnykh techakh kontura
teplonositelya

p. 5-14

(RU)

8 refs., 2 figs., 3 tabs.

Izvestiya Vysshikh Uchebnykh Zavedenij. Yadernaya Ehnergetika

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(no.2)

NUCLEAR POWER; NUCLEAR ENGINEERING; REACTORS; FAST REACTORS; VK-50
REACTOR; BWR TYPE REACTORS

The article is devoted to the results of investigation of radiation safety of the VK-50 reactor facility. The VK-50 reactor is the reactor with natural circulation of coolant. Author describes the features of the catalytic combustion systems in accidents. The feature of phase transfer radioactivity can limit the consequences of severe accidents on the boiling water reactor

В работе приведены результаты исследований внутренне присущих свойств радиационной безопасности установки ВК-50 с естественной циркуляцией теплоносителя в корпусе реактора. Приведены выявленные особенности работы систем каталитического сжигания водорода в условиях повышенной влажности парогазовой смеси при авариях. Обосновано, что особенности фазового переноса радиоактивности позволяют ограничивать последствия тяжелой запроектной аварии на корпусном кипящем реакторе

Accident analysis of reactor facility by using thermal hydraulic code KORSAR S21

*Parshikov, I.A.; Solov'ev, D.S.; Solov'ev, S.L. (OAO VNIIEhS, Moscow (RU)), E-mail:
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Raschetnyj analiz avarijnykh rezhimov reaktornoj ustanovki s ispol'zovaniem
teplogidravlichesкого koda KORSAR

p. 15-26

(RU)

2 refs., 8 figs.

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NUCLEAR POWER; REACTORS; NUCLEAR POWER PLANTS; LWGR TYPE
REACTORS; ACCIDENTS; SAFETY ANALYSIS; SIMULATION; THERMAL
HYDRAULICS

The result of this work is substantial upgrading of code KORSAR to enable the implementation of the safety analysis of the RBMK reactor facility. Received local and integral parameters indicate that the thermal hydraulic code KORSAR suitable for safety analysis of RBMK reactor facility

Результатом этой работы стала существенная модернизация кода КОРСАР для обеспечения возможности выполнения расчетного анализа реакторных установок РБМК. Полученные локальные и интегральные параметры свидетельствуют о том, что теплогидравлический расчетный код КОРСАР пригоден для анализа безопасности реакторных установок РБМК

The development of multi-point acoustic probe diagnostics system of two-phase coolant

S22

Mel'nikov, V.I.; Ivanov, V.V.; Teplyashin, I.A. (Nizhegorodskij Gosudarstvennyj Tekhnicheskij Univ. imeni R.E. Alekseeva, Nizhnij Novgorod (RU)), E-mail: melnikov@nntu.nnov.ru; Loginov, A.V.; Shmelev, D.I. (OAO Opytnoe Konstruktorskoe Byuro Mashinostroeniya Afrikantov, Nizhnij Novgorod (RU))

Razrabotka mnogotochechnoj akustozondovoj sistemy diagnostiki dvukhfaznogo teplonositelya p. 27-34

(RU)

5 refs., 7 figs.

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NUCLEAR POWER; NUCLEAR FACILITIES; REACTOR SAFETY; REACTOR SHUTDOWN; REACTOR COOLING SYSTEMS; ACOUSTIC MEASUREMENTS; ACOUSTIC DETECTION

An air-water two phase flow diagnostics system based on acoustic sounding method has been developed for passive system of emergency cooling. The system make possible to study the real volumetric gas content distribution in the cross section of rectangular channel 100x100 mm at different angles of slope from vertical

Разработана система диагностики двухфазного водовоздушного потока, основанная на методе акустического зондирования. Созданная система позволяет изучать распределение истинного объемного газосодержания по сечению прямоугольного канала 100x100 мм при различных углах его наклона от вертикали

Synthesis of nanostructural materials from metal fusions and prospect of their use in various areas of science and technics

S36

Askhadullin, R.Sh.; Osipov, A.A. (FGUP GNTs RF-FEhI im. A.I. Lejpunskogo, Obninsk (RU)), E-mail: raskhadullin@ippe.ru

Sintez nanostrukturnykh materialov iz metallicheskich rasplavov i perspektivy ikh ispol'zovaniya v razlichnykh oblastiakh nauki i tekhniki

p. 35-42

(RU)

3 refs., 5 figs.

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METALLURGY; METALS; LIQUID METALS; GALLIUM; BISMUTH; ALUMINIUM;
ZIRCONIUM; SILICON NITRIDES; SILICON CARBIDES; CERAMICS

The prospects of creating a liquid metal technology of nanomaterials synthesis of a wide range, which is managed by the selective oxidation of metal, dissolved in an inert with respect to the oxidant used liquid metal media have been presented in the paper

В работе рассмотрены перспективы создания жидкометаллической технологии синтеза широкого спектра наноматериалов, которая заключается в управляемом селективном окислении металла, растворенного в инертной по отношению к используемому окислителю жидкометаллической среде

Study of influence of nanostructured aluminum oxide aero-gel on the crystallization process and structure of clamp material based on the magnesium oxide

S36

Sukhonosov, V.Ya.; Chernov, V.A. (GNTs RF-FEhI. im. A.I. Lejpunskogo, Obninsk (RU)), E-mail: chernovlad@mail.ru; Bogdanov, N.Yu. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU))

Issledovaniya vliyaniya nanostrukturirovannogo aehrogelya oksida alyuminiya na protsessy kristallizatsii i strukturoobrazovaniya materiala-fiksatora na osnove oksida magniya

p. 43-50

(RU)

7 refs., 7 figs.

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METALLURGY; MATERIALS; MAGNESIUM OXIDES; ALUMINIUM PHOSPHATES;
ALUMINIUM OXIDES; NANOSTRUCTURES; SPENT FUELS

Study of influence of nanostructured aluminum oxide aero-gel on the crystallization process and structure of clamp material based on the magnesium oxide has been presented in the paper

В работе приведены исследования влияния наноструктурированного аэрогеля оксида алюминия на процессы кристаллизации и структурообразования материала-фиксатора на основе оксида магния

Calculation of isotopic composition and induced activity of irradiated materials in innovative accelerator-driven systems

S07

Korovin, Yu.A.; Maksimushkina, A.V. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)), E-mail: korovin@iate.obninsk.ru

Raschet izotopnogo sostava i navedennoj aktivnosti obluchennykh materialov innovatsionnykh ehlektroyadernykh ustanovok

p. 51-61

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16 refs., 7 figs., 2 tabs.

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ISOTOPES; ISOTOPE RATIO; ISOTOPE APPLICATIONS; SPENT FUEL ELEMENTS;
NUCLEAR REACTIONS; NEURAL NETWORKS

The calculation of isotopic composition and induced activity of irradiated materials in innovative accelerator-driven systems has been presented in the paper

В работе представлен расчет изотопного состава и наведенной активности облученных материалов инновационных электроядерных установок

Calculation code to estimate admissible thickness of NPP equipment components under flow-accelerated corrosion

S22;S36

Shcherbakov, A.V.; Gulina, O.M.; Sal'nikov, N.L. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)), E-mail: alex@sitetokey.net

Programmnyj kompleks rascheta dopuskaemykh tolshchin stenok ehlementov oborudovaniya AEhS v usloviyakh ehroziionno-korroziionnogo iznosa

p. 62-69

(RU)

3 refs., 3 figs., 3 tabs.

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NUCLEAR POWER; NUCLEAR FACILITIES; REACTOR SAFETY; PIPELINES; SERVICE LIFE; CORROSION; CORROSION FATIGUE; DEFORMATION

The paper presents a calculation code to estimate admissible wall thickness of NPP pipelines made of carbon steel to evaluate residual service life under flow accelerated corrosion

Авторы представляют разработанный программный комплекс по расчету допустимых толщин элементов трубопроводов атомных станций, изготовленных из углеродистых сталей и работающих в условиях эрозионно-коррозионного износа

Comparative analysis of MCNPX and GEANT4 for fast neutron radiation treatment planning

S62

Solov'ev, A.N.; Fedorov, V.V.; Kharlov, V.I.; Stepanova, U.A. (FGBU Meditsinskij Radiologicheskij Nauchnyj Tsentri Ministerstva Zdravookhraneniya Rossii, Obninsk (RU)), E-mail: salonf@googlemail.com

Sravnitel'nyj analiz programm MCNPX i GEANT4 dlya dozimetricheskogo planirovaniya terapii

bystryimi nejtronami

p. 70-80

(RU)

18 refs., 8 figs., 2 tabs.

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MEDICINE; NUCLEAR MEDICINE; NEUTRON THERAPY; NEUTRON TRANSPORT;
MONTE CARLO METHOD; FAST NEUTRONS

The paper presents a comparative analysis of the MCNPX and GEANT_4 simulation codes in radiation treatment planning tasks for fast neutron therapy.

Представлен сравнительный анализ средств моделирования программ MCNPX и GEANT_4 в задачах расчета транспорта быстрых нейтронов для дистанционной лучевой терапии

Studies validating a sodium purification system integrated in the reactor vessel **S22**

Kalyakin, S.G.; Sorokin, A.P.; Kozlov, F.A.; Alekseev, V.V.; Shcherbakov, S.I. (FGUP GNTs RF-FEhI im. A.I. Lejpunskogo, Obninsk (RU)), E-mail: kalyakin@ippe.ru

Issledovaniya v obosnovanie vstroennoj v bak reaktora sistemy ochistki natriya

p. 81-89

(RU)

4 refs., 3 figs., 1 tab.

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NUCLEAR POWER; FAST REACTORS; REACTOR VESSELS; PURIFICATION; COLD TRAPS; SODIUM; HYDROGEN

Authors present the studies to validate a sodium purification system integrated in the reactor vessel. Based on the analysis, the conclusion is made that the sodium-cooled trap is preferable because it allows gaining a more rational use of the space inside the trap

Авторы представляют исследования в обоснование встроеной в бак реактора системы очистки натрия. Сделан вывод о предпочтении варианта ловушки с натриевым охлаждением, позволяющим получить более экономичное использование объема внутри ловушки

GRS method to evaluate uncertainties in calculation parameters of an advanced fast reactor **S22**

Peregudov, A.A.; Andrianova, O.N.; Manturov, G.N.; Raskach, K.F.; Semenov, M.Yu.; Tsibulya, A.M. (FGUP GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU)), E-mail:

peregudov.abbn@inbox.ru

Ispol'zovanie metoda GRS dlya otsenki pogreshnosti nejtronno-fizicheskikh kharakteristik perspektivnogo bystrogo reaktora

p. 90-98

(RU)

9 refs., 4 figs., 3 tabs.

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NUCLEAR POWER; FAST REACTORS; REACTOR SAFETY; POWER DENSITY; DOSE RATES; DATA COVARIANCES

Authors describe the GRS technique applied to estimate the uncertainties of calculation parameters of an advanced sodium-cooled fast reactor, power density and stainless steel dose rate. The calculations have been performed using the diffusion nodal code TRIGEX and Monte Carlo code MMK

В статье описывается методика использования метода GRS для оценки погрешностей функционалов перспективного быстрого натриевого реактора большой мощности. Расчеты проводились по программам TRIGEX и MMKKENO

Verification of ABBN constants and CONSYST code in criticality calculations S22;S97

Golovko, Yu.E.; Koshcheev, V.N.; Lomakov, G.B.; Manturov, G.N.; Rozhikhin, E.V.; Semenov, M.Yu.; Tsibulya, A.M.; Yakunin, A.A. (FGUP GNTs RF-FEhI, Obninsk (RU)), E-mail: bnab@ippe.ru

Verifikatsiya sovremennoj versii konstant BNAB i programmy podgotovki CONSYST v raschetakh kritichnosti

p. 99-108

(RU)

10 refs., 7 figs., 1 tab.

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NUCLEAR FACILITIES; REACTOR SAFETY; CRITICALITY; MONTE CARLO METHOD; BENCHMARKS; VERIFICATION

Authors give a description of up-to-date version of BNAB neutron constants library which has been obtained from evaluated neutron data files RUSFOND-2010 and intended for perspective fast reactor models calculations in current paper materials

В работе дано описание состояния современной версии библиотеки групповых констант БНАБ, полученной на основе файлов оцененных нейтронных данных РОСФОНД-2010, предназначенной для расчетов перспективных моделей быстрых реакторов

Testing covariance matrices of uncertainties in the BNAB data system

S22;S97

Andrianova, O.N.; Golovko, Yu.E.; Zherdev, G.M.; Zadornov, D.V.; Koshcheev, V.N.; Manturov, G.N.; Peregudov, A.A.; Tsibulya, A.M. (FGUP GNTs RF-FEhI im. A.I. Lejpunskogo, Obninsk (RU)), E-mail: bnab@ippe.ru

Testirovanie kovariatsionnykh matrits pogreshnostej sistemy konstant BNAB

p. 109-117

(RU)

11 refs., 5 tabs.

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NUCLEAR FACILITIES; FAST REACTORS; BELOYARSK-3 REACTOR; BN-800 REACTOR; REACTOR SAFETY; CRITICALITY; MONTE CARLO METHOD

The present work is focused on testing covariance matrices of uncertainties which were derived from expert evaluation of the current version of the BNAB group constants library based on evaluated data files RUSFOND-2010

Работа посвящена тестированию библиотеки ковариационных матриц погрешностей, созданной на основании экспертных оценок для современной версии системы групповых констант БНАБ, в основе которой использована библиотека оцененных данных РОСФОНД-2010

Stochastic theory of zero power nuclear reactors. Part 2. Probability of degeneration for a branching process and some issues of estimating the probability of a nuclear accident**S97**

Volkov, Yu.V. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)), E-mail: yuvolkov@iate.obninsk.ru

Stokhasticheskaya teoriya yadernykh reaktorov nulevoj moshchnosti. Chast' 2. Veroyatnost' vyzhdeniya vetvyashchegosya protsessa i voprosy otsenki veroyatnosti vozniknoveniya yadernoj avarii

p. 118-127

(RU)

8 refs., 1 fig.

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MATHEMATICS; STOCHASTIC PROCESSES; MATHEMATICAL MODELS; STRUCTURAL MODELS; EQUATIONS; ZERO POWER REACTORS; REACTOR ACCIDENTS; CHAIN REACTIONS; NUCLEAR REACTIONS

The author has derived a formula to estimate the degeneration probability in which the conditional probability, that the fission process will not stop by the time t , is determined by using a system of non-linear differential equations

В работе выведена формула для оценки вероятности вырождения, в которой условная вероятность того, что процесс деления не закончится к моменту времени t , определяется из полученной в работе системы нелинейных дифференциальных уравнений

Effect of statistical characteristics of fuel pin bundle on evaluation of temperature in the core of sodium cooled fast reactor

S22

Tikhomirov, B.B.; Poplavskij, V.M. (FGUP GNTs RF-FEhI im. A.I. Lejpunskogo, Obninsk (RU)), E-mail: tibb@ippe.ru

Vliyanie statisticheskikh kharakteristik puchka tvehlov TVS na otsenku temperaturnogo rezhima aktivnoj zony bystrogo natrievogo reaktora

p. 128-139

(RU)

13 refs., 6 figs., 2 tabs.

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NUCLEAR POWER; FAST REACTORS; SRE REACTOR; REACTOR CORES; FUEL ELEMENTS; FUEL ELEMENT CLUSTERS; COOLANTS

The article is devoted to the optimization of analytical models used for the evaluation of fast reactor fuel element cladding temperature with the aim to increase the fidelity of the results obtained. Various hypothetical bundle models adopted for the calculation have been studied and their drawbacks are identified

Применительно к быстрым натриевым реакторам проанализированы различные модели пучка твэлов, используемые при расчетах температуры теплоносителя и оболочки топливных элементов внутри тепловыделяющей сборки (ТВС). Отмечены недостатки существующих моделей. Предложена модель пучка, основанная на экспериментальном исследовании реального размещения твэлов внутри чехла ТВС

Transmuting minor actinides with thermal reactor neutrons

S21;S22

Kazanskij, Yu.A. (Obninskij Inst. Atomnoj Ehnergetiki - IATEh NIYaU MIFI, Obninsk (RU)), E-mail: kazansky@iate.obninsk.ru; Romanov, M.I. (Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU))

Transmutatsiya malyx aktinidov v spektre nejtronov reaktora na teplovykh nejtronakh

p. 140-148

(RU)

11 refs., 4 figs.

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NUCLEAR FACILITIES; FAST REACTORS; THERMAL REACTORS; CLOSED-CYCLE SYSTEMS; TRANSMUTATION; ACTINIDES; SPENT FUELS

Authors discuss the feasibility of transmuting minor actinides with thermal reactor neutrons. Basic factor of transmutation feasibility - ratio of radioactivity levels with and without transmutation has been used

В данной работе рассмотрена целесообразность трансмутации малых актинидов в нейтронных спектрах реакторов на тепловых нейтронах. Использован основной фактор, влияющий на целесообразность трансмутации, отношение радиоактивностей с трансмутацией и без нее

Implementation of the experience in designing APCS systems for nuclear facilities based on UMIKON package in the educational process

S96;S97

Lebedev, V.O.; Tolokonskij, A.O.; Korolev, S.A.; Vlasov, V.A. (Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU)), E-mail: lebedev-vo@yandex.ru

Vnedrenie opyta sozdaniya ASUTP ob"ektov atomnoj otrasli na baze PTK UMIKON v uchebnyj protsess

p. 149-157

(RU)

10 refs., 4 figs.

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KNOWLEDGE MANAGEMENT; EDUCATION; LEARNING; EDUCATIONAL TOOLS; PERSONNEL; CONTROL SYSTEMS; COMPUTERIZED SIMULATION

The paper describes the experience in creating a training laboratory for designing an Automated Process Control System (APCS) for nuclear facilities based on the UMIKON hardware and software package which facilitates the engineering design process and helps to achieve more flexibility in upgrading APCS

В работе анализируется опыт создания учебного практикума по проектированию АСУТП объектов атомной отрасли на базе программно-технического комплекса УМИКОН, позволяющего обеспечить эффективность проектирования и гибкость модернизации АСУТП

Investigation of the SVRE influence on the safety of large size sodium fast reactor

S21;S22

Ashurko, Yu.M.; Andreeva, K.A.; Bur'evskij, I.V. (GNTs RF-FEhI im. A.I. Lejpunskogo, Obninsk (RU)), E-mail: ashurko@ippe.ru (and others)

Issledovanie vliyaniya natrievogo pustotnogo ehffekta reaktivnosti na bezопасnost' bystrogo natrievogo reaktora bol'shoj moshchnosti

p. 5-14

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4 refs., 8 figs., 2 tabs.

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FAST REACTORS; BN-800 REACTOR; REACTOR SAFETY; REACTOR ACCIDENTS;
FUELS; MIXED OXIDE FUELS

The article provides analysis of ULOF type BDBA for different designs of BN-1200 reactor core with MOX fuel. The design feature of the reactor core is the presence of sodium plenum above the fuel area, significantly reducing SVRE, similarly to what was done earlier in the BN-800 reactor

В работе проанализирован процесс протекания запроектной аварии ULOF для различных вариантов активной зоны реактора БН-1200 с МОКС-топливом. Показано, что во всех рассмотренных вариантах разрушения активной зоны не происходит, хотя возможны кипение теплоносителя и разгерметизация ТВЭЛОВ

Development of technology for continuous acoustic-emission monitoring of metal operational damaging of nuclear power plants main equipment

S22

Bakirov, M.B.; Levchuk, V.I. (OOO NSUTs TsMiR, Lyubertsy, Moskovskaya Obl. (RU));

Povarov, V.P.; Gromov, A.F. (Filial OAO Kontsern Rosehnergoatom Novovoronezhskaya Atomnaya Stantsiya, Novovoronezh (RU)), E-mail: info@expresstest.ru

Razrabotka tekhnologii nepreryvnogo akustiko-ehmissionnogo monitoringa ehkspluatatsionnoj povrezhdaemosti metalla otvetstvennogo oborudovaniya atomnykh stantsij

p. 15-24

(RU)

18 refs., 3 figs.

Izvestiya Vysshikh Uchebnykh Zavedenij. Yadernaya Ehnergetika

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NUCLEAR POWER; NUCLEAR POWER PLANTS; EQUIPMENT; STEAM GENERATORS;
TRANSDUCERS; ACOUSTIC EMISSION TESTING; ACOUSTIC MONITORING

The article presents the results of work on the development of a multi-parameter system for acoustic emission monitoring of nuclear power plant equipment damageability during power unit operation with due account of specific features of NPP control

В статье представлены результаты работы по разработке многопараметрической системы акустико-эмиссионного контроля повреждаемости оборудования атомной станции в процессе эксплуатации блока с учетом специфики контроля в условиях АЭС

Methodology of material testing for WWER fuel assemblies and fuel rods to support introduction of new fuel at NPPs

S22;S42

Pavlov, S.V. (OAO GNTs NIIAR, Dimitrovgrad (RU)), E-mail: pavlov@niiar.ru

Metodologiya materialovedcheskikh issledovaniy TVS i tvehlov VVEhR dlya operativnogo soprovozhdeniya vnedreniya novogo topliva na AEhS

p. 25-34

(RU)

14 refs., 6 figs.

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NUCLEAR POWER; WWER TYPE REACTORS; FUEL ASSEMBLIES; FUEL ELEMENTS; SPENT FUELS; MATERIALS TESTING

Authors propose a methodology for post-irradiation examination of WWER-reactor fuel assemblies and fuel rods to be used for effective engineering support of the introduction of new fuel at NPPs

Авторами предложена методология материаловедческих послереакторных исследований ТВС и твэлов реакторов ВВЭР с целью оперативного научно-технического сопровождения внедрения нового топлива на АЭС

Safety assessment for the MBIR reactor using the RELAP code

S21;S22

Nikulin, E.V.; Sobolev, A.V.; Volkov, Yu.V. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)), E-mail: jaki07@mail.ru

Otsenka pokazatelej bezopasnosti dlya reaktora tipa MBIR s pomoshch'yu raschetnogo koda RELAP

p. 35-43

(RU)

4 refs., 4 figs., 1 tab.

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NUCLEAR FACILITIES; FAST REACTORS; BOR-60 REACTOR; RESEARCH AND TEST REACTORS; COOLANTS; LIQUID METALS

The possibility of performing an analysis of an emergency situation at sodium-cooled reactors using the RELAP code is discussed. Authors underline the difficulty connected with the fact that RELAP does not take into account the liquid metal coolant

Рассмотрена возможность выполнения анализа аварийных ситуаций на реакторах с натриевым теплоносителем в среде расчетного комплекса RELAP. Авторы подчеркивают сложность, заключающуюся в отсутствии учета в RELAP жидкометаллического теплоносителя как такового

A method for statistical comparison of data sets and its uses in analysis of nuclear physics data

S97

Bitjukov, S.I.; Smirnova, V.V. (Inst. Fiziki Vysokikh Ehnergij, Protvino (RU)), E-mail: Serguei.Bitoukov@cern.ch; Krasnikov, N.V. (Inst. Yadernykh Issledovanij RAN, Moscow (RU)); Maksimushkina, A.V. (IATEh NIYaU MIFI, Obninsk (RU)); Nikitenko, A.N. (Imperial Kolledzh, London (GB))

Metod statisticheskogo sravneniya dannykh i ego primenenie dlya analiza ehksperimental'nykh yaderno-fizicheskikh dannykh

p. 44-51

(RU)

11 refs., 4 figs., 3 tabs.

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MATHEMATICS; MATHEMATICAL MODELS; DATA ANALYSIS; DATA PROCESSING; ALGORITHMS; STATISTICAL MODELS; MONTE CARLO METHOD

Authors propose a method for statistical comparison of two data sets. The method is based on the method of statistical comparison of histograms. As an estimator of quality of the decision made, it is proposed to use the value which it is possible to call the probability that the decision (data sets are various) is correct

В работе предлагается метод статистического сравнения данных для использования в задачах анализа экспериментальных и моделированных ядерно-физических данных, являющийся развитием метода статистического сравнения гистограмм. Показана возможность применения предлагаемого метода в задачах сравнения данных разных экспериментов или экспериментальных и смоделированных данных

Software for the radionuclide vertebroplasty

S62;S97

Levchenko, A.V. (Ehksperimental'nyj Nauchno-Issledovatel'skij i Metodicheskij Tsentr Modeliruyushchie Sistemy, Obninsk (RU)); Zabaryanskij, Yu.G. (GNTs RF-FEhI im. A.I. Lejpunskogo, Obninsk (RU)); Golovin, A.A.; Voznesenskij, N.K.; Kurachenko, Yu.A. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)), E-mail: artem.golovin1@gmail.com

Programmnoe obespechenie radionuklidnoj vertebroplastiki

p. 52-61

(RU)

8 refs., 5 figs.

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MEDICINE; NUCLEAR MEDICINE; RADIOLOGY; RADIOTHERAPY; RADIOISOTOPES; COMPUTER CODES; PROGRAMMING

The problem of program maintenance for radionuclide vertebroplasty is considered in the paper. Requirements for systems of preoperative preparation and postoperative analysis are described

Рассмотрена в целом задача программного сопровождения радионуклидной вертебропластики. Описаны требования к системам предоперационной подготовки и послеоперационного анализа

Thermohydraulic justification for the installation of block-containers with uranium-bearing material into experimental channel of WWR-c reactor **S22**

Kochnov, O.Yu. (Filial NIFKhI im. L.Ya. Karpova, Obninsk (RU)), E-mail:

kochnov2000@mail.ru; Levchenko, Yu.D. (GNTs RF-FEhI im. A.I. Lejpunskogo, Obninsk (RU));

Chusov, I.A. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU))

Теплогидравлическое обоснование возможности установки контейнеров с урансодержащим материалом в экспериментальный канал реактора ВВР-ц

р. 62-72

(RU)

5 refs., 8 figs., 6 tabs.

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POWER REACTORS; WWER TYPE REACTORS; REACTOR CORES; REACTOR CHANNELS; EXPERIMENTAL CHANNELS; FISSIONABLE MATERIALS; THERMAL HYDRAULICS

This paper presents the feasibility of block-containers with uranium-bearing fissionable material being installed into the experimental channels of the WWR-c reactor. Two versions of the block-container structure have been examined, namely, those with regular and enhanced loading of the fissionable material

На основании численных расчетов в работе показана возможность установки в один из экспериментальных каналов реактора ВВР-ц дополнительных контейнеров с урансодержащим делящимся материалом. Приводится описание методики расчета теплогидравлики экспериментального канала

Method of purification of liquid radioactive wastes and concentrates from organic impurities **S12**

S12

Legkikh, K.G.; Smykov, V.B. (GNTs RF-FEhI im. A.I. Lejpunskogo, Obninsk (RU)), E-mail:

Kristina-legkih@yandex.ru

Метод очистки жидких радиоактивных отходов и концентратов от органической составляющей

р. 73-81

(RU)

8 refs., 4 figs., 3 tabs.

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WASTE MANAGEMENT; RADIOACTIVE WASTE PROCESSING; LIQUID WASTES; PURIFICATION; SOLIDIFICATION; CEMENTING; DETERGENTS

A method for LRW and LRC purification by an inorganic sorbent, such as polianite, has been proposed by authors. The research demonstrates the efficiency of LRW purification from organic impurities. The principal tasks of further research into the solidification processes which occur during chemical processing of the sludge samples have been formulated on the basis of experimental data

В работе рассматривается метод очистки жидких радиоактивных отходов (ЖРО) и жидких радиоактивных концентратов с использованием неорганического сорбента. Проведенные исследования показали эффективность очистки ЖРО от органических примесей. На основании экспериментальных данных сформулированы основные задачи дальнейших исследований процессов отверждения при химической обработке образцов пульпы

Development of neutron spectrum analysis method to assess the content of fissile isotopes in SFA

S12

Mitskevich, A.V. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)), E-mail: Aleksandr_M@tut.by

Razrabotka metodiki nejtronnogo spektral'nogo analiza dlya otsenki sodержaniya delyashchikhsya izotopov v OTVS

p. 82-92

(RU)

15 refs., 5 figs., 7 tabs.

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WASTE MANAGEMENT; SPENT FUELS; CHEMICAL ANALYSIS; FISSION CHAMBERS; FUEL ASSEMBLIES; URANIUM 235; PLUTONIUM 239

The paper presents the integrated neutron spectrum analysis as a potential method for estimating the contents of fissile isotopes in spent fuel assemblies (SFA). Two method implementation variants are described: (1) measurement of SFA average transmission and (2) measurement of sample average transmission in the spectrum that has passed a SFA

Рассматривается интегральный нейтронный спектральный анализ в качестве возможной методики оценки содержания делящихся изотопов в отработавшей тепловыделяющей сборке (ОТВС). Представлены два варианта реализации методики - измерение среднего пропускания ОТВС и эталона на спектре, прошедшем ОТВС

ABBN-RF group constants library for nuclear reactor and shielding calculations

S22

Koshcheev, V.N.; Manturov, G.N.; Nikolaev, M.N.; Tsibulya, A.M. (FGUP GNTs RF-FEhI im. A.I. Lejpunskogo, Obninsk (RU)), E-mail: koscheev@ippe.ru

Biblioteka gruppovykh konstant BNAB-RF dlya raschetov reaktorov i zashchity

p. 93-101

(RU)

18 refs., 1 tab.

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NUCLEAR FACILITIES; FAST REACTORS; REACTOR SAFETY; BENCHMARKS;
VERIFICATION; RADIATION PROTECTION

The paper presents a description of a new ABBN-RF group constants library intended for calculations of different types of nuclear reactors and radiation shielding. The ABBN-RF library was constructed based on the national RUSFOND-2010 neutron data file library

В работе дано описание новой библиотеки групповых констант БНАБ-РФ, предназначенной для расчетов различных типов ядерных реакторов и радиационной защиты и полученной на основе файлов национальной библиотеки нейтронных данных РОСФОНД-2010

Stochastic theory of zero power nuclear reactors. Part 3. Stochastic differential equations of zero-dimensional reactor kinetics. Weak external neutron source. Analysis of the equivalent reactivity noise model

S97

Volkov, Yu.V. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)), E-mail: volkov@iate.obninsk.ru

Stokhasticheskaya teoriya yadernykh reaktorov nulevoj moshchnosti. Chast' 3. Stokhasticheskie differentsial'nye uravneniya tochechnoj kinetiki reaktora. Slabyj postoronnij istochnik nejtronov. Analiz modeli ehkvivalentnogo shuma reaktivnosti

p. 102-112

(RU)

8 refs., 1 fig.

Izvestiya Vysshikh Uchebnykh Zavedenij. Yadernaya Ehnergetika

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MATHEMATICS; STOCHASTIC PROCESSES; MATHEMATICAL MODELS;
STRUCTURAL MODELS; ZERO POWER REACTORS; REACTOR ACCIDENTS;
EQUATIONS; DIFFERENTIAL EQUATIONS; REACTOR KINETICS

Stochastic differential equations of zero-dimensional reactor kinetics have been derived assuming that the discrete-time and continuous-time branching processes should have moments of distribution which converge up to the second order, inclusive. The concept of the weak external neutron source has been clarified more precisely

Автором получены стохастические дифференциальные уравнения точечной кинетики ядерного реактора из условия, что дискретный ветвящийся процесс и его непрерывный аналог должны иметь совпадающие моменты распределения до второго порядка включительно. Уточнено понятие слабого постороннего источника нейтронов

Approaches to optimization of core reactivity coefficients for the MASTER heat supply reactor

S22

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Puti optimizatsii koehffitsientov reaktivnosti aktivnoj zony reaktora teplosnabzheniya MASTER p. 113-122

(RU)

12 refs., 7 figs., 2 tabs.

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NUCLEAR POWER; THERMAL REACTORS; REACTOR PHYSICS; REACTIVITY COEFFICIENTS; REACTOR CORES; NEUTRON ABSORBERS

Authors show approaches to optimization of core reactivity coefficients for the MASTER heat supply reactor. The WIMS-D4 neutron-physical code has been used as a calculation tool

Авторы демонстрируют пути оптимизации коэффициентов реактивности активной зоны реактора теплоснабжения МАСТЕР. В качестве расчетного инструмента в работе использовался нейтронно-физический код WIMS-D4

Implementation of expert continuous acoustic-emission monitoring technology for NPP critical equipment operational defectiveness assessment

S22

Bakirov, M.B.; Nikolaev, D.A.; Levchuk, V.I.; Gorokhov, S.M. (OOO Nauchno-Sertifikatsionnyj Uchebnyj Tsentr Materialovedeniya i Resursa Komponentov Yadernoj Tekhniki Tsentr Materialovedeniya i Resursa, Moscow (RU)); Povarov, V.P.; Gromov, A.F. (Filial OAO Kontsern Rosehnergoatom Novovoronezhskaya Atomnaya Stantsiya, Novovoronezh (RU)), E-mail: info@expresstest.ru

Vnedrenie tekhnologii ehkspertnogo nepreryvnogo akustiko-ehmissionnogo monitoringa dlya otsenki ehkspluatatsionnoj povrezhdaemosti metalla otvetstvennogo oborudovaniya atomnykh stantsij

p. 5-10

(RU)

7 refs., 3 figs.

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NUCLEAR POWER PLANTS; REACTOR OPERATION; STEAM GENERATORS; REACTOR SAFETY; ACOUSTIC MONITORING; NONDESTRUCTIVE TESTING; ACOUSTIC EMISSION TESTING

The results of the experimental implementation of the operational defectiveness by nondestructive testing technology based on the multi parameter acoustic emission continuous monitoring during the operation of power unit are presented in this article

В статье представлены результаты опытного внедрения технологии неразрушающего контроля эксплуатационной повреждаемости на основе использования многопараметрического акустико-эмиссионного непрерывного мониторинга на стадии эксплуатации блока атомной станции

NPP-2006 with WWER-1200 type reactor - a new approach to displaying information from technical diagnostics systems

S22

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AES 2006 s ehnergoblokami VVEhR-1200 - novyj podkhod k otobrazheniyu informatsii ot sistem tekhnicheskoy diagnostiki

p. 11-20

(RU)

4 refs., 4 figs.

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(no.4)

NUCLEAR POWER; WWER TYPE REACTORS; CONTROL EQUIPMENT; DIAGNOSTIC TECHNIQUES; DISPLAY DEVICES; DATA ACQUISITION

This article provides an analysis of the information from the systems of technical diagnostics, and attempts to formulate requirements for information output to operating personnel and to design a screen presentation of the information

В работе приводится анализ информации, поступающей с систем технической диагностики, выводимой на верхний уровень, описана разработка требований к экраным формам и варианты реализации

Loop tests of thermionic fuel elements in the AM reactor

S22

Yarygin, V.I. (GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU)), E-mail: ecs@ippe.ru

Petlevye ispytaniya termoehmisionnykh ehlektrogeneriruyushchikh kanalov v reaktore AM

p. 21-31

(RU)

9 refs., 8 figs.

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(no.4)

REACTORS; SPACE POWER REACTORS; TOPAZ REACTOR; THERMIONIC REACTORS; FUEL ELEMENTS; THERMIONIC FUEL ELEMENTS

The paper presents an overview of a thermionic power generating channels development results and test loop channels for space nuclear power plants in the first generation of AM (Atom Mirnyj) reactor

В работе представлен краткий обзор результатов разработки и испытаний петлевых каналов с термоэмиссионными электрогенерирующими каналами для космических ядерно-энергетических установок первого поколения в реакторе АМ (Атом Мирный)

Chemical decomposition of water into hydrogen in heterogeneous aluminum-containing compositions

S37

Milinchuk, V.K.; Belozarov, V.I.; Anan'eva, O.A.; Laricheva, T.E.; Kunitsyna, T.E. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)), E-mail: vkmilinchuk@mail.ru

Khimicheskoe razlozhenie vody na vodorod v geterogennykh alyuminijsoederzhashchikh kompozitsiyakh

p. 32-40

(RU)

9 refs., 4 figs., 3 tabs.

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(no.4)

CHEMISTRY; WATER; DECOMPOSITION; HYDROGEN; ALUMINIUM; ALUMINIUM OXIDES; CALCIUM OXIDES

Authors investigate the kinetic regularities and mechanism of chemical decomposition of water into hydrogen in heterogeneous hydro-reactive systems, containing aluminum, aluminum alloys, and such chemical compounds as liquid sodium glass or quicklime giving a water solution an alkaline character

В работе исследованы кинетические закономерности и механизм химического разложения воды на водород в гидрореакционных гетерогенных системах, содержащих алюминий, алюминиевые сплавы и химические соединения (жидкое натриевое стекло или негашеная известь), придающие водному раствору щелочной характер

Photoneutrons for neutron capture therapy

S62;S97

Kurachenko, Yu.A. (FGUP GNTs RF-FEhI im. A.I. Lejpunskogo, Obninsk (RU)), E-mail: ykurachenko@mail.ru

Fotonejtrony dlya nejtronzakhvatnoj terapii

p. 41-51

(RU)

20 refs., 4 figs., 2 tabs.

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MEDICINE; NUCLEAR MEDICINE; RADIOLOGY; RADIOTHERAPY; CAPTURE;
PHOTONEUTRONS; GALLIUM; MONTE CARLO METHOD

Possibilities of neutron capture therapy based on a powerful medium-energy electron accelerator have been analyzed. An accelerator with 35 MeV of electron energy and 4 mA of average beam current has been selected for the purpose

В работе проанализированы возможности организации нейтронозахватной терапии на базе мощного ускорителя электронов средних энергий с энергией электронов 35 МэВ и средним током 4 мА

Specifics of calculation of heat rate in oxide nuclear fuel during tests in reactor BOR-60

S22

Varivtsev, A.V.; Zhemkov, I.Yu. (OAO GNTs NIIAR, Dimitrovgrad (RU)), E-mail:

vav3@niiar.ru, ziu@niiar.ru

Osobennosti raschetnogo opredeleniya teplovydeleniya v oksidnom yadernom toplive pri ispytaniyakh v reaktore BOR-60

p. 52-59

(RU)

9 refs., 3 figs., 2 tabs.

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(no.4)

FAST REACTORS; BOR-60 REACTOR; NUCLEAR FUELS; FUEL ASSEMBLIES; FUEL ELEMENTS; FISSION PRODUCTS; HEAT RATE

Authors investigate into the specifics of calculation of heat rate in oxide nuclear fuel during tests in reactor BOR-60

В работе исследуются особенности расчетного определения тепловыделения в оксидном ядерном топливе при испытаниях в реакторе БОР-60

Combined numerical and experimental investigations of hydrodynamics and coolant flow mass transfer out of spacer grid in fuel assemblies of floating power unit

S22

Dmitirev, S.M.; Doronkov, D.V.; Pronin, A.N.; Solntsev, D.N.; Sorokin, V.D.; Khrobostov, A.E.

(Nizhegorodskij Gosudarstvennyj Tekhnicheskij Univ. imeni R.E. Alekseeva, Nizhnij Novgorod (RU)), E-mail: dmitriev@nntu.nnov.ru

Raschetno-ehksperimental'nye issledovaniya gidrodinamiki i massoobmena teplonositelya za distantsioniruyushchej reshetkoj teplovydelyayushchej sborki reaktora plavuchego ehnergobloka

p. 60-70

(RU)

8 refs., 8 figs.

Izvestiya Vysshikh Uchebnykh Zavedenij. Yadernaya Ehnergetika

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NUCLEAR FACILITIES; OFFSHORE NUCLEAR POWER PLANTS; REACTOR CORES; HYDRODYNAMICS; FUEL ASSEMBLIES; MASS TRANSFER; COOLANTS

The results of experimental investigations of local hydrodynamics and inter-cell mass transfer of coolant flow in specific zones of fuel assembly of KLT-40C reactor facility behind a plate spacer grid have been presented in the paper

В работе представлены результаты экспериментальных исследований локальной гидродинамики и межъячеечного массообмена потока теплоносителя в характерных зонах тепловыделяющей сборки реакторной установки КЛТ-40С за пластинчатой дистанционирующей решеткой

Calculational studies for security justification of SVBR-100 reactor fuel cycle based on reprocessed uranium

S22;S98

D'yachenko, A.I.; Balagurov, N.A.; Artisyuk, V.V. (Negosudarstvennoe Obrazovatel'noe Uchrezhdenie Dopolnitel'nogo Professional'nogo Obrazovaniya Tsentral'nyj Inst. Povysheniya Kvalifikatsii Goskorporatsii Rosatom, Obninsk (RU)), E-mail: dyachenkoai@mail.ru; Fedorov, M.I.; Solov'ev, S.V. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)), E-mail: fedorovmikhail@bk.ru

Raschetnye issledovaniya v obosnovanie zashchishchennosti toplivnogo tsikla reaktora SVBR-100 pri ispol'zovanii regenerirovannogo urana

p. 71-81

(RU)

13 refs., 5 figs., 5 tabs.

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REACTORS; FBR TYPE REACTORS; FUEL CYCLE; URANIUM RECYCLE; URANIUM 236; SAFEGUARDS; PROLIFERATION

Authors discuss the possible use of reprocessed uranium to form barriers against proliferation of fissile materials in SVBR-100 fuel cycle. Quantitative estimates of the required initial presence of ^{236}U isotope in order to reduce the attractiveness of plutonium from the view point of proliferation have also been made in the paper

В работе проводится анализ возможного использования регенерированного урана для формирования барьеров против несанкционированного распространения делящихся материалов топливного цикла реактора СВБР-100. Произведены количественные оценки требуемого начального присутствия изотопа ^{236}U для повышения защищенности плутония против несанкционированного распространения

Conditioning of high level reactor core graphite waste using self-propagating high temperature synthesis

S12

Konovalov, Eh.E.; Naumov, V.S.; Lastov, A.I. (GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU)), E-mail: vnaumov@ippe.ru

Konditsionirovanie vysokoaktivnykh otkhodov reaktornogo grafita s ispol'zovaniem samorasprostranyayushchegosya vysokotemperaturnogo sinteza

p. 82-91

(RU)

13 refs., 3 figs., 3 tabs.

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RADIOACTIVE WASTES; RADIOACTIVE WASTE PROCESSING; HIGH-LEVEL RADIOACTIVE WASTES; GRAPHITE; FISSION PRODUCTS; TRANSURANIUM ELEMENTS; TITANIUM CARBIDES; CORUNDUM

Issues of conditioning of high level reactor core graphite waste using self-propagating high temperature synthesis have been discussed in the paper

В работе рассмотрены технологические операции кондиционирования содержащих просыпь высокоактивных отходов графита уран-графитовых реакторов

Comparison of fuel cycles characteristics for nuclear energy systems based on WWER-TOI and BN-1200 reactors

S12;S22

Kagramanyan, V.S.; Kalashnikov, A.G.; Kapranova, Eh.N.; Puzakov, A.Yu. (FGUP GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU)), E-mail: kagramanyan@ippe.ru

Sravnenie kharakteristik toplivnykh tsiklov statsionarnoj yadernoj ehnergetiki na osnove reaktorov VVEhR-TOI i BN-1200

p. 92-101

(RU)

12 refs., 3 tabs.

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NUCLEAR POWER; WWER TYPE REACTORS; FUEL CYCLE; SPENT FUELS; PLUTONIUM RECYCLE; MIXED OXIDE FUELS; ACTINIDES

Authors determine the characteristics of the fuel cycle (FC) based on stationary nuclear power system based on WWER-TOI and BN-1200 reactors with fuel of different composition. Characteristics of reactor systems with partial or complete spent nuclear fuel reprocessing and recycling of plutonium are compared to those of the reference system consisting only of WWER-TOI with uranium oxide fuel, operating in an open FC

В работе определяются характеристики топливных циклов (ТЦ) стационарной ядерной энергетики на основе реакторов ВВЭР-ТОИ и БН-1200 с топливом различного состава. Характеристики систем реакторов с частичной или полной переработкой отработанного ядерного топлива и рециклом плутония сравниваются с характеристиками референтной системы, состоящей только из реакторов ВВЭР-ТОИ с урановым оксидным топливом, работающих в открытом ТЦ

Assessment of the increasing in ^{131}I production due to improved tellurium target in the WWR-C reactor core

S38

Kochnov, O.Yu.; Kolesov, V.V.; Fomin, R.V.; Zherdev, G.M. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)), E-mail: kochnov2000@mail.ru

Otsenka uvelicheniya proizvodstva ^{131}I pri ispol'zovanii tellurovykh mishenej usovershenstvovannoj konstruktsii na reaktore VVR-ts

p. 102-110

(RU)

5 refs., 5 figs., 5 tabs.

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RADIOISOTOPES; ISOTOPE PRODUCTION; IODINE 131; TELLURIUM; TARGETS; IRRADIATION DEVICES

Authors have investigated replacing aluminum alloy target shell with one made of stainless steel, and found a 20% decrease in the ^{131}I yield. The comparison between the calculated ^{131}I yield and the actual ^{131}I yield testifies that the cumulated ^{131}I yield can be significantly increased

В работе исследована замена алюминиевого сплава оболочки мишени на оболочку из нержавеющей стали и обнаружено 20%-ное снижение выхода ^{131}I . Сравнение рассчитанного выхода ^{131}I с реально извлекаемым показывает, что суммарный выход ^{131}I может быть существенно увеличен

Analysis of operational factors that can lead to fuel failure in WWER units

S22

Evdokimov, I.A.; Likhanskij, V.V.; Sorokin, A.A.; Zborovskij, V.G.; Kozhakin, A.N.; Chernetskij, M.G. (FGUP GNTs RF-TRINITI, Troitsk (RU)), E-mail: evdokimov@triniti.ru; Shestakov, Yu.M.; Semenovikh, A.S. (OAO VNIAEhS, Moscow (RU)), E-mail: shestakovym@mail.ru

Analiz ehkspluatatsionnykh faktorov, kotorye mogut privodit' k razgermetizatsii TVS VVEhR

p. 111-122

(RU)

8 refs., 7 figs.

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NUCLEAR POWER PLANTS; WWER TYPE REACTORS; FUEL RODS; FUEL ELEMENT FAILURE; LEAKS; POST-IRRADIATION EXAMINATION; ON-SITE INSPECTION

The present paper is focused on fuel failure in WWER units in the period of 2003 through 2013. The data of visual inspections and post-irradiation examinations of leaking fuel assemblies in hot cells are reported

В статье представлены результаты анализа данных по тепловыделяющим сборкам (ТВС) реакторов ВВЭР, в которых в период с 2003 по 2013 гг. разгерметизировались оболочки ТВЭЛОВ. Описаны результаты анализа данных визуального осмотра и послереакторных исследований ТВС с негерметичными ТВЭлами в горячих камерах

On the physical conditions for arising a controlled fusion chain reaction supported by neutrons in fusion facilities with magnetic plasma confinement **S70**

Shmelev, A.N.; Kulikov, G.G. (Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU)), E-mail: shmelan@mail.ru

O fizicheskikh usloviyakh dlya vozniknoveniya upravlyaeмой tseпnoj reaktsii sinteza s uchastiem nejtronov v termoyadernykh ustanovkakh s magnitnym uderzhaniem plazmy
p. 123-129

(RU)

8 refs., 1 fig., 1 tab.

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THERMONUCLEAR REACTIONS; SUBCRITICAL ASSEMBLIES; CHAIN REACTIONS; CROSS SECTIONS; NEUTRON FLUX; TRANSMUTATION; PLASMA CONFINEMENT

Authors define physical conditions for generating a controlled fusion chain reaction supported by neutrons in fusion facilities with magnetic plasma confinement. Such thermonuclear facilities could be used for fast transmutation of long-lived fission products with low neutron absorption cross-section, and perhaps even without their preliminary isotopic separation

Авторы определяют физические условия для возникновения управляемой цепной реакции синтеза с участием нейтронов в термоядерных установках с магнитным удержанием плазмы. Такие термоядерные установки можно будет использовать для быстрой трансмутации долгоживущих продуктов деления с малым сечением поглощения нейтронов и, возможно, даже без их предварительного изотопного разделения

Bioindication of the environment in the vicinity of the Far-Eastern Center for radioactive waste treatment

S12

Gorshkova, T.A. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)), E-mail: tgorshkova@yandex.ru; Udalova, A.A. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)); Vserossijskij Nauchno-Issledovatel'skij Inst. Radiologii i Agroehkologii, Obninsk (RU)); Geras'kin, S.A. (Vserossijskij Nauchno-Issledovatel'skij Inst. Radiologii i Agroehkologii,

Obninsk (RU); Kiselev, S.M.; Akhromeev, S.V. (FGBU GNTs Federal'nyj Meditsinskij Biofizicheskij Tsentr imeni A.I. Burnazyana, Moscow (RU))

Bioindikatsiya sostoyaniya prirodnoj sredy v rajone raspolozheniya Dal'nevostochnogo tsentra po obrashcheniyu s radioaktivnymi otkhodami

p. 130-139

(RU)

10 refs., 3 figs., 1 tab.

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RADIOACTIVE WASTE FACILITIES; RADIOACTIVE WASTE MANAGEMENT; ENVIRONMENTAL PROTECTION; ECOSYSTEMS; CONTAMINATION; RADIATION MONITORING

The present study aims at estimating the environmental conditions in the vicinity of the Far-Eastern Center on radioactive waste treatment using bioindication techniques. The findings obtained show that the bioindication approach could be considered as a promising method for primary screening-level assessment of the environmental well being under man-made (including radioactive) contamination

Целью работы являлась оценка состояния природной среды в зоне наблюдения пункта временного хранения радиоактивных отходов (РАО) Дальневосточного центра по обращению с РАО методами биоиндикации. Авторы показывают, что методы биоиндикации можно рассматривать в качестве перспективного приема первичной скрининговой оценки состояния окружающей среды в условиях техногенного, в том числе радиоактивного, загрязнения

Prospects of WWER-SKD in closed fuel cycle

S12;S22

Glebov, A.P.; Klushin, A.V.; Baranaev, Yu.D. (FGUP GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU)), E-mail: glebov@ippe.ru, baranaev@ippe.ru

Perspektivy ispol'zovaniya reaktora VVEhR-SKD v zamknutom toplivnom tsikle

p. 5-19

(RU)

18 refs., 5 figs., 2 tabs.

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NUCLEAR POWER; FAST REACTORS; WWER TYPE REACTORS; FUEL CYCLE; MIXED OXIDE FUELS; SPENT FUELS; ACTINIDES

Authors examines peculiarities of implementing closed fuel cycle in Russia, implementation stages and duration, associated problems and possible ways of dealing with them; the paper also proposes solutions for optimizing fuel cycle using SCWR reactor

Авторы рассматривают особенности реализации замкнутого топливного цикла (ЗТЦ) в России, этапы, сроки их выполнения, возникающие проблемы. Обосновывается использование реакторов SCWR с быстрым спектром нейтронов в системах с ЗТЦ

Nuclear power plants safety improvement by hydrogen technologies

S22

Aminov, R.Z.; Yurin, V.E. (Saratovskij Gosudarstvennyj Tekhnicheskij Univ. imeni Yu.A. Gagarina, Saratov (RU); Otdel Ehnergeticheskikh Problem Saratovskogo Nauchnogo Tsentra RAN, Saratov (RU)), E-mail: oepran@inbox.ru, urin1990777@bk.ru

Povyshenie bezopasnosti atomnykh ehlektricheskikh stantsij na osnove vodorodnykh tekhnologij p. 20-27

(RU)

7 refs., 4 figs., 2 tabs.

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(no.1)

NUCLEAR POWER PLANTS; REACTOR SAFETY; POWER SUPPLIES; ACCIDENTS; OUTAGES; HYDROGEN BURNING

Authors propose an effective application of hydrogen technologies in nuclear power plants; it improves the plant maneuverability at normal operation, and secures own electricity needs in case of station black-out

В работе предложен путь эффективного использования на атомных электростанциях водородных технологий, которые в штатном режиме дают возможность увеличить маневренность станции, а в случае полного обесточивания обеспечить электроснабжение собственных нужд АЭС

Analysis of the defectiveness of the welds of #Numero Sign#111 PGV-1000 and proposal on their elimination

S22

Povarov, V.P. (Filial OAO Kontsern Rosehnergoatom Novovoronezhskaya AEhS, Novovoronezh (RU)), E-mail: PovarovVP@vnpp1.rosenergoatom.ru

Analiz povrezhdennykh svarnykh shvov #Numero Sign#111 PGV-1000 i predlozheniya po ikh ustraneniyu

p. 28-38

(RU)

12 refs., 3 figs., 3 tabs.

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(no.1)

NUCLEAR POWER PLANTS; WWER TYPE REACTORS; REACTOR SAFETY; EQUIPMENT; STEAM GENERATORS; WELDED JOINTS

Author presents the results of investigating the reasons for the damage of metal in the zone of welded joint #Numero Sign#111 of hot collector to the branch pipe Du1200 of steam generator housing

В работе представлены результаты исследования причин повреждения металла в зоне сварного соединения #Numero Sign#111 приварки горячего коллектора к патрубку Ду1200 корпуса парогенератора

The software to test components of the integrated security systems of NPP of different developers on the integration

S22

Bezuglov, D.Yu.; Trofimov, M.A. (OOO ATEhKS, Obninsk (RU); Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)), E-mail: d1.bezuglov@gmail.com

Spetsializirovannoe programmnoe obespechenie dlya proverki komponentov integrirovannykh sistem bezopasnosti AEhS razlichnykh proizvoditelej na vozmozhnost' integratsii

p. 39-44

(RU)

7 refs., 2 figs.

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NUCLEAR POWER PLANTS; PHYSICAL PROTECTION; SECURITY; SAFETY STANDARDS; MECHANICAL TESTS; PROGRAMMING

Authors present the specialized software to test components of the integrated security systems of NPP of different manufacturers to be integrated

Авторы представляют специализированное программное обеспечение (ПО) для проверки компонентов интегрированных систем безопасности АЭС различных производителей на возможность интеграции. Описаны основополагающие функции разработанного ПО, механизм его работы и выходные данные

Self-organizing carbonitride coatings on steel under molten lead-magnesium eutectic

S36

Orlova, E.A.; Orlov, Yu.I.; Kryuchkov, E.A.; Komyshnyj, V.N.; Zhmurin, V.G.; Zagrebaev, S.A.; Kotovskij, N.A.; Dvortsevoj, V.G. (GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU)), E-mail: eorlova@ippe.ru

Samoorganizuyushcheesya karbonitridnoe pokrytie na stali iz rasplavlennoj ehvtektiki svinets-magnij

p. 45-55

(RU)

7 refs., 6 figs., 4 tabs.

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(no.1)

MATERIALS; ALLOYS; ZIRCONIUM NITRIDES; CARBON NITRIDES; LEAD; MAGNESIUM; CORROSION

Authors have developed a working site to study the formation of nitride and/or carbide cover from eutectic Pb-Mg alloy in the narrow gap between coaxial tubes. Trial tests are described in the paper

Авторами разработан и изготовлен рабочий участок для исследований по формированию нитридных и/или карбидных защитных покрытий из эвтектического расплава Pb-Mg в узком зазоре между коаксиально расположенными трубками. Описаны пробные испытания

Computational and experimental studies on the causes of crack network formation in the area of the heat exchanger tube sheet in the BN-600 reactor **S22**

Vilenskij, O.Yu.; Krylov, A.N.; Osipov, S.L.; Osetrov, D.L.; Rogozhkin, S.A. (OAO Opytnoe Konstruktorskoe Byuro Mashinostroeniya imeni I.I. Afrikantova, Nizhnij Novgorod (RU)); Margolin, B.Z.; Prokoshev, O.Yu.; Pozdnyakov, M.L.; Gulenko, A.G. (FGUP Tsentral'nyj Nauchno-Issledovatel'skij Inst. Konstruktsionnykh Materialov Prometej, Sankt-Peterburg (RU)), E-mail: silaev@okbm.nnov.ru, margolinbz@yandex.ru

Raschetno-eksperimental'nye issledovaniya prichin obrazovaniya setki treshchin v rajone verkhnej trubnoj doski teploobmennika reaktora BN-600

p. 56-65

(RU)

10 refs., 4 figs., 1 tab.

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(no.1)

FAST REACTORS; BELOYARSK-3 REACTOR; EQUIPMENT; REACTOR COMPONENTS; HEAT EXCHANGERS; FATIGUE; CREEP; CRACKS

Computational and experimental studies on the causes of crack network formation in the area of the heat exchanger tube sheet in the BN-600 reactor are presented in the paper. Authors prove that the presence of these cracks does not limit the possibility of extending the life of the PTO of the reactor facility BN-600 up to 45 years

В работе представлены расчетно-экспериментальные исследования причин образования сетки трещин в районе верхней трубной доски теплообменника реактора БН-600. Авторы доказывают, что наличие этих трещин не ограничивает возможность продления срока эксплуатации ПТО реакторной установки БН-600 до 45 лет

Swelling of guide tubes for safety rods in nonuniform fields of temperature and irradiation

S22;S60

Porollo, S.I.; Konobeev, Yu.V. (FGUP GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU)); Garner, F.A. (Firma Konsul'tatsii po Vliyaniyu Oblucheniya, Richland (US)), E-mail: porollo@ippe.ru, konobeev@ippe.ru, frank.garner@dslextreme.com

Raspukhanie napravlyayushchikh gil'z sterzhnej SUZ bystrykh reaktorov v neodnorodnykh temperaturnykh i radiatsionnykh polyakh

p. 66-75

(RU)

13 refs., 7 figs., 4 tabs.

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FAST REACTORS; BOR-60 REACTOR; REACTOR SAFETY; SWELLING; CREEP;
SCRAM RODS

The purpose of the present work is to investigate the shape changes and void swelling in safety rod guide tubes of the reactors BN-350 and BOR-60 after operation for the scheduled life time. Authors show that the change of safety rod guide tube sizes occurs due to a non-uniform swelling of the material along tube wall height, perimeter and thickness

В работе исследовались формоизменение и набухание направляющих гильз стержней управления и защиты (СУЗ) реакторов БН-350 и БОР-60 после отработки ими заданного ресурса. Авторами показано, что деформация направляющих гильз стержней СУЗ происходит в результате неоднородного набухания материала гильз по высоте, периметру и толщине стенки

The dielectric monitors of the high-dose and high-temperature reactor irradiation

S22;S36

Stepanov, V.A.; Isaev, E.I. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)); Kryukova, L.M.; Chernov, V.M. (Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU)); OAO VNIINM imeni Akademika A.A. Bochvara, Moscow (RU)); Plaksin, O.A. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)); FGUP GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU)); Stepanov, P.A. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)); Obninskoe Nauchno-Proizvodstvennoe Predpriyatie Tekhnologiya, Obninsk (RU)), E-mail: stepanov@iate.obninsk.ru

Dielektricheskie monitory uslovij vysokodoznogo i vysokotemperaturnogo reaktornogo oblucheniya

p. 76-86

(RU)

11 refs., 8 figs.

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REACTORS; MATERIALS; RADIOACTIVE MATERIALS; IRRADIATION; DIELECTRIC MATERIALS; RADIATION MONITORING; REACTOR CONTROL SYSTEMS

Radiation-induced changes of the structure of oxide and nitride dielectric materials and their properties are investigated in the paper. The use of high-temperature dielectric materials as temperature monitors has been proved

В работе исследованы радиационные изменения структуры и свойств оксидных и нитридных диэлектрических материалов. Обосновано применение высокотемпературных

Flexible distributed control and protection system for industrial objects power consumers

S29

Alontseva, E.N.; Belousov, P.A. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)), E-mail: alenika@iate.obninsk.ru

Gibkaya raspredelennaya sistema upravleniya i zashchity konechnykh potrebitelej ehlektroehnergii promyshlennykh ob"ektov

p. 87-91

(RU)

5 refs., 2 figs.

Izvestiya Vysshikh Uchebnykh Zavedenij. Yadernaya Ehnergetika

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(no.1)

NUCLEAR POWER PLANTS; ENERGY CONSUMPTION; ENERGY EFFICIENCY; ENERGY SPECTRA; RESOURCE MANAGEMENT; RESOURCE CONSERVATION

An approach to the construction of systems based on the concept of smart grids with a specific set of hardware and software to optimize the energy consumption and redistribute power, to quickly change the characteristics of the electrical network has been discussed in the paper

В статье рассмотрен подход к построению системы учета и управления энергоресурсами на базе концепции интеллектуальных сетей при помощи специализированного комплекса технических и программных средств, позволяющий оптимизировать энергозатраты, перераспределять электроэнергию, оперативно менять характеристики электрической сети

Combined numerical and experimental researches of temperature pulsations of collector unit fragment of heat exchanger clean-up and cooldown power nuclear unit systems

S22

Dmitriev, S.M.; Ryazapov, R.R.; Sobornov, A.E.; Kotin, A.V.; Mamaev, A.V. (Nizhegorodskij Gosudarstvennyj Tekhnicheskij Univ. imeni R.E. Alekseeva, Nizhnij Novgorod (RU)), E-mail: dmitriev@ntu.nnov.ru

Raschetno-ehksperimental'noe issledovanie termopul'satsij fragmenta kollektornogo uzla teploobmennika sistemy ochistki i raskholazhivaniya vodo-vodyanykh reaktorov

p. 92-102

(RU)

8 refs., 8 figs.

Izvestiya Vysshikh Uchebnykh Zavedenij. Yadernaya Ehnergetika

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(2015)

(no.1)

NUCLEAR POWER PLANTS; PWR TYPE REACTORS; HEAT EXCHANGERS; COOLANT CLEANUP SYSTEMS; REACTOR SHUTDOWN; CLEANING

An experimental model of collector unit fragment of heat-exchanger clean-up and cooldown power nuclear unit systems has been designed and manufactured. Experimental researches of temperature state have been carried out with the use of a given experimental model. Authors have proposed solutions focused on degradation of intensity of temperature pulsations

Разработана и изготовлена экспериментальная модель фрагмента коллекторного узла теплообменника системы очистки и расхолаживания водо-водяных реакторов. С использованием данной модели проведены экспериментальные исследования температурного состояния. Предложены решения, направленные на снижение интенсивности термопульсаций

Study of functional characteristics for safety system check valve using scaled model

S21;S22

Baluev, D.E.; Gusev, D.V.; Meshkov, S.I.; Nikanorov, O.L.; Osipov, S.L.; Rogozhkin, S.A.; Rukhlin, S.V.; Shepelev, S.F. (OAO OKBM Afrikantov, Nizhny Novgorod (RU)), E-mail: nikanorov@okbm.nnov.ru

Issledovanie funktsional'nykh kharakteristik obratnogo klapana sistemy bezopasnosti na masshtabnoj modeli

p. 103-110

(RU)

7 refs., 5 figs.

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(no.1)

NUCLEAR POWER PLANTS; LIQUID METAL COOLED REACTORS; TEST FACILITIES; REACTOR SAFETY; RELIEF VALVES; THERMODYNAMIC MODEL

Authors demonstrate results of experimental scale-model studies on functional characteristics of a check valve used in the safety system of a liquid-metal-cooled reactor. Check valve model fluid dynamic characteristics obtained in the course of experimental studies are shown in diagrams as functions of some design and operation parameters of the model

Авторами представлены результаты экспериментальных исследований функциональных характеристик обратного клапана, применяемого в системе безопасности реактора с жидкометаллическим теплоносителем на масштабной модели. Гидродинамические характеристики модели клапана, полученные в ходе экспериментальных исследований, представлены на графиках в виде зависимостей от некоторых конструктивных и эксплуатационных параметров модели

On high level waste immobilization in Y-Al garnet-based cermet matrix under SHS conditions

S12

Konovalov, Eh.E.; Lastov, A.I.; Nerozin, N.A. (FGUP GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU)), E-mail: eekonovalev@mail.ru

К вопросу иммобилизации высокоактивных отходов в керметную матрицу на основе Y-Al

granata v rezhime SVS

p. 111-118

(RU)

9 refs., 4 figs., 1 tab.

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RADIOACTIVE WASTES; RADIOACTIVE WASTE PROCESSING; HIGH-LEVEL RADIOACTIVE WASTES; SOLIDIFICATION; MATRICES; MATRIX MATERIALS; ALUMINIUM OXIDES; YTTRIUM COMPOUNDS

Authors recommend an advanced method of HLRAW immobilization for developing a technology of ultimate processing of spent nuclear fuels of nuclear power plants

Авторы предлагают метод иммобилизации высокоактивных отходов, характеризующийся новизной, достойными технико-экономическими и экологическими показателями, который может стать основой технологического процесса завершающего передела переработки отработавшего ядерного топлива

Influence of inorganic ligands on effectiveness of radioactive waste ion-exchange processing

S12

Chugunov, A.S.; Rumyantsev, A.V.; Vinnitskij, V.A.; Nechaev, A.F. (Sankt-Peterburgskij Gosudarstvennyj Tekhnologicheskij Inst. (TU), Sankt-Peterburg (RU)), E-mail: tchugunov@mail.ru

Vliyanie neorganicheskikh ligandov na ehffektivnost' ionoobmennoj pererabotki radioaktivnykh otkhodov

p. 119-127

(RU)

16 refs., 4 figs.

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(no.1)

RADIOACTIVE WASTES; RADIOACTIVE WASTE PROCESSING; LIQUID WASTES; LIGANDS; ION EXCHANGE; MAGNESIUM IONS; COBALT IONS; CATIONS

Some practical methods to minimize the impact of inorganic ligands on the ion-exchange effectiveness have been proposed in the paper. It is proved that the ligands of mineral acids can have a noticeable effect on the cation exchange efficiency

В работе предложены практические методы минимизации влияния неорганических лигандов на коэффициент ионообменной дезактивации радиоактивных растворов. Доказано, что лиганды минеральных кислот могут оказывать заметное влияние на эффективность катионообменной очистки растворов

Forming a supply of protected nuclear fuel based on RepU for countries- recipients of Russian nuclear technologies

S22;S12

Fedorov, M.I. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)); D'yachenko, A.I.; Balagurov, N.A.; Artisyuk, V.V. (Negosudarstvennoe Obrazovatel'noe Uchrezhdenie Dopolnitel'nogo Professional'nogo Obrazovaniya Tsentral'nyj Inst. Povysheniya Kvalifikatsii Goskorporatsii Rosatom, Obninsk (RU)), E-mail: fedorovmikhail@bk.ru

Formirovanie postavok zashchishchennogo yadernogo topliva na osnove regenerirovannogo urana dlya stran-retsipientov rossijskikh yadernykh tekhnologij

p. 128-135

(RU)

7 refs., 7 figs.

Izvestiya Vysshikh Uchebnykh Zavedenij. Yadernaya Ehnergetika

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(no.1)

REACTORS; WWER TYPE REACTORS; NUCLEAR FUELS; BURNUP; SPENT FUELS; URANIUM RECYCLE; FUEL SUPPLIES

This paper presents the estimation of the time required for the formation of a protected fuel loading of the WWER-1000 reactor based on reprocessed uranium recovered from spent fuel of reactors of the same type cleared of minor actinides and fission products

В работе рассматриваются различные стратегии перевода реакторов ВВЭР-1000 на регенерированное урановое топливо для оценки необходимого времени формирования защищенной топливной загрузки реактора ВВЭР- 1000 на основе урана, выделенного из отработанного топлива реактора того же типа, очищенного от минорных актинидов и продуктов деления

The CORNER neutronic calculation code

S97

Bereznev, V.P.; Seleznev, E.F.; Asatryan, D.S. (Inst. Problem Bezopasnogo Razvitiya Atomnoj Ehnergetiki RAN, Moscow (RU)), E-mail: bvp@ibrae.ac.ru

Nejtronno-fizicheskij raschetnyj kod CORNER

p. 136-143

(RU)

8 refs., 3 figs., 1 tab.

Izvestiya Vysshikh Uchebnykh Zavedenij. Yadernaya Ehnergetika

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(no.1)

CALCULATION METHODS; DISCRETE ORDINATE METHOD; NEUTRON TRANSPORT THEORY; HEXAGONAL CONFIGURATION; DIFFERENTIAL GEOMETRY; FAST REACTORS

The paper presents calculations of a BN-800 core problem verified against the MMK Monte Carlo code. Discretization of the angular variable is carried out by introducing the angular quadrature set. There is an option of specifying a set of user's quadratures

В работе представлены результаты кросс-верификации с расчетным кодом ММК, основанным на методе Монте-Карло, на модели активной зоны реактора БН-800. Энергетическая зависимость представлена многогрупповым приближением. Дискретизация угловой переменной осуществляется за счет введения угловых квадратур

Neutronics of BRIG-300 breeder reactor

S22

Samokhin, D.S.; Znak, I.M.; Terekhova, A.M. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)), E-mail: samohin@iate.obninsk.ru

Fizicheskie osobennosti proekta reaktornoj ustanovki-bridera BRIG-300

p. 144-149

(RU)

5 refs., 1 fig., 3 tabs.

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(2015)

(no.1)

NUCLEAR ENERGY; NUCLEAR POWER PLANTS; FAST REACTORS; BREEDER REACTORS; COOLANTS; REACTOR CORES

Authors give the description of some characteristics of the BRIG-300 reactor design. Evaluation of neutronics characteristics of a breeder reactor BRIG-300 are made in the software package TRIGEX. The results of the calculation of the effective multiplication factor, and the balance of neutrons in the core of the reactor plant are also presented

В работе приводится описание некоторых характеристик быстрого реактора проекта БРИГ-300. Даны оценки нейтронно-физических характеристик реактора-бридера, выполненные с использованием программного комплекса TRIGEX, также приводятся результаты расчета эффективного коэффициента размножения и баланса нейтронов в активной зоне реакторной установки

Radioactive decay data uncertainties library of isotopes for ABBN constant system

S22

Barabanova, D.S. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)); Zherdev, G.M. (FGUP GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU)), E-mail: dassha20081@rambler.ru

Biblioteka pogreshnostej kharakteristik raspada radionuklidov v sisteme konstant BNAB

p. 150-156

(RU)

5 refs., 1 fig., 4 tabs.

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(2015)

(no.1)

NUCLEAR FACILITIES; REACTOR SAFETY; BENCHMARKS; COMPUTER CODES; VERIFICATION; MANUALS; DECAY

This article contains the creation results of uncertainty data library of decay data at the ABBN-format. The analyses of different evaluations are presented, as well calculation results of these uncertainties on the test task

В статье представлены результаты работы по созданию библиотеки погрешностей данных о радиоактивных распадах в формате БНАБ. Сравнены разные оценки погрешностей и на примере тестовой задачи определено их влияние на результат расчета остаточного энерговыделения

Neutron leakage spectra from Pb-Li sphere with central ^{252}Cf - and 14 MeV-neutron sources and verification of evaluated neutron data

S73

Blokhin, A.I.; Zhuravlev, B.V.; Talalaev, V.A.; Sipachev, I.V. (GNTs RF-Fiziko-Ehnergeticheskij Inst. imeni A.I. Lejpunskogo, Obninsk (RU)), E-mail: blokhin@obninsk.ru

Spektry nejtronov utechki iz Pb-Li-sfery s ^{252}Cf - i 14 MeV-istochnikami nejtronov v tsentre i testirovka otsenennykh nejtronnykh dannyykh

p. 157-162

(RU)

5 refs., 3 figs.

Izvestiya Vysshikh Uchebnykh Zavedenij. Yadernaya Ehnergetika

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(2015)

(no.1)

NEUTRON SOURCES; NEUTRON SPECTRA; NEUTRON LEAKAGE; IONIZATION CHAMBERS; NEUTRON DETECTORS; SCINTILLATION COUNTERS

The performed measurements with the use of a specially designed fast ionization chamber, providing the stop pulses for the time-of-flight technique as well as total number of ^{252}Cf disintegrations during the experiment, have been described in the paper. The measured data have been compared with the MCNP-4 Monte-Carlo code calculations with nuclear data processed from the ENDF/B-VII.1 and BROND-3 libraries

В статье описан процесс выполнения измерений с использованием специально спроектированной быстрой ионизационной камеры, обеспечивающей одновременно стоповые импульсы для времяпролетной техники и фиксирование полного количества распадов ^{252}Cf за время эксперимента. Дано сравнение результатов обоих измерений с расчетами по программе Монте-Карло MCNP-4B с библиотеками нейтронных данных ENDF/B-VII.1 и БРОНД-3

Nuclear power as a basis for future electricity generation

S21

Pioro, I.L. (Univ. of Ontario Inst. of Technology, Ontario (CA)), E-mail: Igor.Pioro@uoit.ca; Kirillov, P.L. (State Scientific Centre of the Russian Federation - Inst. for Physics and Power Engineering n.a. A.I. Lejpunsky (SSC RF-IPPE), Obninsk (RU))

p. 5-23

(EN)

21 refs., 6 figs., 10 tabs.

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(no.2)

NUCLEAR POWER; NUCLEAR POWER PLANTS; NUCLEAR INDUSTRY; POWER GENERATION; PWR TYPE REACTORS; THERMAL REACTORS; LWGR TYPE REACTORS

The paper presents the current status of nuclear power industry in the world, and a comparison of nuclear energy systems to other energy systems

В статье кратко излагается современное состояние атомной энергетики и дается ее сравнение с другими источниками энергии

Prediction own energy consumption nuclear power plants using data mining methods

S21

Kleshcheva, M.A.; Nakhabov, A.V. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)), E-mail: kleshcheva_ma@mail.ru

Predskazanie sobstvennogo ehnergopotrebleniya AEhS s ispol'zovaniem metodov intellektual'nogo analiza dannykh

p. 24-31

(RU)

6 refs., 5 figs., 3 tabs.

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(2015)

(no.2)

NUCLEAR POWER; NUCLEAR POWER PLANTS; ENERGY CONSUMPTION; ENERGY EFFICIENCY; PWR TYPE REACTORS; POWER GENERATION

This article discusses the use of various methods of data analysis and data mining to predict the energy consumption for own needs of nuclear power plant using actual operational data

В статье рассматривается применение различных методов анализа данных для прогнозирования собственного энергопотребления АЭС с использованием фактических данных, сравнение этих методов как между собой, так и с используемым в настоящее время на АЭС

Investigation of the dependence testimony sector control system leak fuel cladding BN-600 reactor on the operational parameters

S22

Albutova, O.I.; Luk'yanov, D.A (AO GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU)), E-mail: oalbutova@ippe.ru

Issledovanie zavisimosti pokazanij sektornoj sistemy kontrolya germetichnosti obolochek tvehlov reaktora BN-600 ot ehkspluatatsionnykh parametrov

p. 32-38

(RU)

7 refs., 4 figs., 1 tab.

Izvestiya Vysshikh Uchebnykh Zavedenij. Yadernaya Ehnergetika

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(2015)

(no.2)

NUCLEAR POWER; FAST REACTORS; BELOYARSK-3 REACTOR; FUEL ASSEMBLIES;
LEAKS; REACTOR MONITORING SYSTEMS

The problem of determining the background of indications of measuring channels of the sector control tightness claddings (SSKGO), which significantly depend on the location of the control points and the operational parameters of the reactor, relevant for fast reactors, has been discussed in the paper

Цель работы - исследование зависимости показаний измерительных каналов секторной системы контроля герметичности оболочек твэлов (ССКГО) реактора БН-600 от эксплуатационных параметров и разработка регрессионной модели предсказания фоновых показаний ССКГО

Evaluation of instability of the metrological characteristics of research reactors measuring channels

S22

Bereznoj, D.A.; Kravtsova, S.E.; Malovik, K.N. (Sevastopol'skij Gosudarstvennyj Univ., Sevastopol' (RU)), E-mail: bereznoyd@mail.ru

Otsenivanie nestabil'nosti metrologicheskikh kharakteristik izmeritel'nykh kanalov issledovatel'skikh reaktorov

p. 39-48

(RU)

8 refs., 6 figs.

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(no.2)

NUCLEAR POWER; RESEARCH AND TEST REACTORS; REACTOR CHANNELS;
PARAMETRIC INSTABILITIES; ERRORS

The article identifies factors affecting the instability of error measuring channel neutron power and areas of preventive control an instability of error of measuring channels neutron power. The use of methods of the Taguchi theory for assessing the instability of metrological characteristics of measuring channels has been proposed by the authors

В работе рассмотрены факторы, воздействующие на нестабильность погрешности измерительных каналов (ИК). Определена область упреждающего контроля нестабильности погрешности ИК нейтронной мощности. Предложено применение подходов теории Тагучи для оценки нестабильности метрологических характеристик ИК

Stand-alone hydrogen generator based on the chemical decomposition of water

by aluminum

S36

Milinchuk, V.K.; Klinshpont, Eh.R.; Belozarov, V.I. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)), E-mail: milinchuk@iate.obninsk.ru

Avtonomnyj generator vodoroda na osnove khimicheskogo razlozheniya vody alyuminiem
p. 49-59

(RU)

12 refs., 7 figs.

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(no.2)

MATERIALS; HYDROGEN; ALUMINIUM; ALUMINIUM OXIDES; WATER; HYDROGEN GENERATORS; DECOMPOSITION

Authors present a stand-alone hydrogen generator based on the chemical decomposition of water in heterogeneous compositions containing finely dispersed powder aluminum and crystalline sodium metasilicate

Авторы представляют разработанный автономный генератор водорода на основе химического разложения воды в гетерогенных композициях, содержащих высокодисперсный порошок алюминия и кристаллогидраты метасиликата натрия

Testing of the system code designed for simulation of hypothetical beyond design-basis accident on fast breeder reactor

S22

Zajtsev, A.A.; Kazantsev, A.A.; Luk'yanov, A.A.; Supotnitskaya, O.V. (OAO GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU)), E-mail: zajtsev@ippe.ru; Semenov, V.N.; Fokin, A.L.; Tsaun, S.V. (Inst. Problem Bezopasnogo Razvitiya Atomnoj Ehnergetiki RAN, Moscow (RU)); Filippov, M.F. (Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU))

Testirovanie integrirovannogo programmnoho sredstva po modelirovaniyu gipoteticheskikh avarij na RU BN

p. 60-70

(RU)

12 refs., 4 figs., 2 tabs.

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(2015)

(no.2)

NUCLEAR POWER; NUCLEAR POWER PLANTS; BREEDER REACTORS; REACTOR ACCIDENTS; LEAKS; FISSION PRODUCTS; REACTOR SAFETY

The paper is devoted to the solution of an actual problem of development of a system code for analysis of hypothetical beyond design-basis accidents for fast breeder reactor. The presented results of the solution of the test task confirm operability of the system code providing modeling all specified chain of processes

Цель работы - создание интегрированного программного средства для проведения сквозного самосогласованного расчета всего цикла жизни продуктов деления (ПД) на

АЭС с реакторной установкой на быстрых нейтронах от накопления ПД в активной зоне до выхода их в окружающую среду и распространение за промплощадку АЭС

Modeling of direct conversion of uranium fission fragments kinetic energy to laser radiation energy in argon-xenon dusty plasma containing uranium nanoparticles

S30

Slyunyaev, M.N.; Budnik, A.P.; Sipachev, A.V. (AO GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU)), E-mail: max.my.net@gmail.com

Modelirovanie pryamogo preobrazovaniya kineticheskoy ehnergii oskolkov deleniya urana v ehnergiyu lazernogo izlucheniya v argon-ksenonovoj pylevoj plazme s nanochastitsami urana p. 71-80

(RU)

11 refs., 5 figs., 1 tab.

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(no.2)

ENERGY CONVERSION; DIRECT ENERGY CONVERSION; KINETIC ENERGY; LASER RADIATION; FISSION; COMPUTERIZED SIMULATION; MATHEMATICAL MODELS

Authors investigate the process of direct conversion of uranium fission fragments kinetic energy to laser radiation energy in the moving argon-xenon laser- active gas medium containing uranium nanoparticles. The model of the space-time evolution of concentration distribution of the uranium nanoparticles injected into the cylindrical dust laser-active element and a method of model's numerical solution have been developed in the course of the study

В работе исследуется процесс прямого преобразования кинетической энергии осколков деления урана в энергию лазерного излучения в движущейся аргон-ксеноновой лазерно-активной газовой среде, содержащей наночастицы урана. Разработаны модель и метод численного решения уравнений модели прямого преобразования кинетической энергии осколков деления урана в энергию лазерного излучения в такой среде

Concerning hydrogen production on the base of nuclear technologies

S08

Khorasanov, G.L.; Kolesov, V.V.; Korobejnikov, V.V. (Obninskij Inst. Atomnoj Ehnergetiki IATEh NIYaU MIFI, Obninsk (RU)), E-mail: khorasanow@yandex.ru

К вопросу polucheniya vodoroda na baze yadernykh tekhnologij

p. 81-87

(RU)

8 refs.

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(no.2)

HYDROGEN; HYDROGEN PRODUCTION; HYDROGEN FUELS; METHANE; FAST REACTORS; BELOYARSK-3 REACTOR

The paper is dedicated to assuming a possibility of application a sodium fast reactor technology for steam methane reforming into hydrogen. Existence of three independent power loops in the Russian reactor BN-600 allows using a steam generator of one of these loops as a source of pressurized steam for preparation a steam-gas mixture

В работе рассматривается возможность использования технологии быстрого натриевого реактора для паровой конверсии метана в водород. Наличие трех независимых энергетических петель в российском быстром реакторе БН-600 позволяет использовать парогенератор одной из петель как источник водяного пара

Experience of computer technology usage within university training for future specialists of nuclear power plants under #Left-Pointing Double Angle Quotation Mark#the university-enterprise#Right-Pointing Double Angle Quotation Mark# program

S97

Semenov, V.K.; Vol'man, M.A. (FGBOU VPO Ivanovskij Gosudarstvennyj Ehnergeticheskij Univ. imeni V.I. Lenina, Ivanovo (RU)), E-mail: semenov_vk@mail.ru; Zhuravleva, V.S. (Filial OAO Kontsern Rosehnergoatom Kalininskaya Atomnaya Stantsiya, Udomlya (RU))

Опыт применения компьютерных технологий в вузовской подготовке будущих специалистов для АЭС по программе #Left-Pointing Double Angle Quotation Mark#vuz-predpriyatie#Right-Pointing Double Angle Quotation Mark#

р. 88-95

(RU)

10 refs., 1 fig.

Izvestiya Vysshikh Uchebnykh Zavedenij. Yadernaya Ehnergetika

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(no.2)

MATHEMATICS; COMPUTERS; COMPUTERIZED SIMULATION; MATHEMATICAL MODELS; REACTOR SIMULATORS; TRAINING; EDUCATION

The article deals with the aspects of training program for future specialists of nuclear power plants. This program is realized at NPP Department of Ivanovo State University and Kalinin NPP. The usage of computer and simulation modeling at the university are the main components of this concept

В работе рассмотрены аспекты разработанной на кафедре АЭС Ивановского государственного энергетического университета совместно с Калининской АЭС программы подготовки будущих специалистов для АЭС, составными элементами которой являются использование компьютерного и имитационного моделирования в вузе

Experimental study of thermal and hydraulic characteristics of VK-300 reactor in solitary uptake tube model

S22

Serdun', N.P.; Ignatenko, V.I.; Kotikov, G.S. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)), E-mail: snp@iate.obninsk.ru

Ehksperimental'nye issledovaniya teplogidravlicheskich kharakteristik reaktora VK-300 na

modeli odinochnoj tyagovoj trubyy

p. 96-107

(RU)

8 refs., 6 figs.

Izvestiya Vysshikh Uchebnykh Zavedenij. Yadernaya Ehnergetika

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(no.2)

NUCLEAR POWER; FAST REACTORS; BWR TYPE REACTORS; THERMAL HYDRAULICS; NATURAL CONVECTION; STEAM SEPARATORS; MIXTURES

Authors present the realization of experimental works of thermohydraulic reactor characterization on the model of single draft tube. They show that there is a possible ingress of steam into the ring slit, modeling inter tubular space and its penetration into the nuclear reactor core inlet

В работе представлены экспериментальные исследования теплогидравлических характеристик реактора ВК-300 на модели одиночной тяговой трубы. Показана возможность попадания пара в кольцевую щель, моделирующую межтрубное пространство, и его проникновение на вход активной зоны

The influence of axial-burnup distribution on K_{eff} for casks with spent fuel S12;S22

Vnukov, V.S.; Chkuaseli, L.I. (AO GNTs RF-FEHI imeni A.I. Lejpunskogo, Obninsk (RU));

Kulikov, V.I. (AO Atomproekt, Sankt-Peterburg (RU)), E-mail: vkulikov@givnpiet.ru

Vliyaniye aksial'nogo raspredeleniya glubiny vygoraniya OTVS na znachenie K_{eff} kontejnerov s otrabotavshim yadernym toplivom

p. 108-116

(RU)

6 refs., 6 figs., 2 tabs.

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NUCLEAR POWER; WWER TYPE REACTORS; RADIATION PROTECTION; SPENT FUELS; BURNUP; SPENT FUEL CASKS; WASTE TRANSPORTATION

This article presents axial burnup data and evaluates the effects of axial-burnup on K_{eff} for American cask GBC-32 storage and transportation spent fuel PWR and Russian cask TUK-6 spent fuel WWER-440

Цель работы - определение минимально допустимых глубин выгорания топлива, позволяющих компоновать безопасные загрузки транспортного упаковочного комплекта ТУК-6 для транспортирования отработавшего топлива реакторов ВВЭР-440

Improvement the value of sodium void reactivity effect of the fast neutron reactor by the instrumentality of the Monte Carlo code

S22

Maslov, P.A.; Matveev, V.I.; Malysheva, I.V. (AO GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU)), E-mail: pmaslov@ippe.ru

Utochnenie velichiny natrievogo pustotnogo ehffekta reaktivnosti v bystrykh natrievykh reaktorakh s pomoshch'yu programm Monte-Karlo

p. 117-125

(RU)

9 refs., 1 fig., 7 tabs.

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NUCLEAR POWER; FAST REACTORS; MIXED OXIDE FUELS; MIXED NITRIDE FUELS; REACTIVITY COEFFICIENTS; BURNUP; MONTE CARLO METHOD

The article considers transitional mode operation of the reactor from the start state to the steady-state overload, where core passes various stages of fuel burn-up

В статье рассмотрен переходный режим работы реактора от стартового состояния до установившегося режима перегрузок, в котором активная зона проходит последовательно разные стадии выгорания топлива

Sensitivity coefficients of the neutron and physical reactor parameters to the fuel inventory parameters

S21;S22

Demeneva, I.V.; Eliseev, V.A.; Korobejnikova, L.V. (AO GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU)), E-mail: idemeneva@ippe.ru

Koeffitsienty chuvstvitel'nosti K_{eff} i KV k parametram toplivnoj zagruzki

p. 126-131

(RU)

5 refs., 3 tabs.

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NUCLEAR POWER; FAST REACTORS; REACTOR CORES; MIXED NITRIDE FUELS; SPENT FUELS; MULTIPLICATION FACTORS; BREEDING RATIO; MAYAK PLANT

Numerical illustrations of sensitivity coefficients to the plutonium isotope composition and other fuel inventory parameters for the reactor BREST-300 are presented in the paper

В работе рассмотрены два подхода к расчету чувствительностей параметров реактора к изменению изотопного состава плутония. Представлены численные иллюстрации применительно к реактору типа БРЕСТ-300

Physical characteristics of the large size sodium cooled fast reactors with advanced nitride and metal fuel

S21;S22

Matveev, V.I.; Malysheva, I.V.; Bur'evskij, I.V. (AO GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU)), E-mail: matveev@ippe.ru

Fizicheskie kharakteristiki bystrykh natrievykh reaktorov bol'shoj moshchnosti na perspektivnykh vidakh topliva - nitridnom i metallicheskom
p. 132-142

(RU)

13 refs., 2 figs., 6 tabs.

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NUCLEAR POWER; FAST REACTORS; MIXED NITRIDE FUELS; LIQUID METAL FUELS; BURNUP; BREEDING

The features of metallic fuel and the results of calculation studies of the use of metallic fuel in large fast sodium reactors with comparison to nitride fuel are presented in the paper

В статье приводятся особенности перспективных видов топлива и результаты расчетных исследований их применения в быстрых натриевых реакторах большой мощности в сравнении с нитридным топливом

Application of zonality conceptual model of chronic effects of ionizing radiation for studying the behavior of radiostrontium in terrestrial ecosystems**S29**

Shoshina, R.R.; Synzynys, B.I. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)); Lavrent'eva, G.V. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)); Kaluzhskij Filial MGTU imeni N.Eh. Baumana, Kaluga (RU)), E-mail: Lavrentyeva_G@list.ru

Primenenie kontseptual'noj modeli zonal'nosti khronicheskogo dejstviya ioniziruyushchej radiatsii pri izuchenii povedeniya radiostrontsiya v sukhoputnykh ehkosistemakh

p. 143-151

(RU)

21 refs., 3 figs.

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ENVIRONMENT; ENVIRONMENTAL IMPACTS; ECOSYSTEMS; TERRESTRIAL ECOSYSTEMS; ECOLOGY; STRONTIUM 90; DOSE RATES

Authors present the application of zonality conceptual model of chronic effects of ionizing radiation for studying the behavior of radiostrontium in terrestrial ecosystems

Авторы рассказывают о применении концептуальной модели зональности хронического действия ионизирующей радиации при изучении поведения радиостронция в сухопутных экосистемах

The 10-th anniversary Russian scientific conference Radiation protection and radiation safety in nuclear technologies. Book of abstracts S61;S54;S22

GK Rosatom, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); GNTs RF - FEhI, Obninsk (RU); NOU DPO TsIPK Rosatoma, Obninsk (RU); IBRAEh RAN, Moscow (RU)

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2015

348 p.

(RU)

LEADING ABSTRACT; MEETINGS; RADIATION PROTECTION; DECOMMISSIONING; COMPUTER CODES; NUCLEAR FACILITIES; OPERATION; NUCLEAR MEDICINE; RADIOECOLOGY; RADIONUCLIDE MIGRATION; PERSONNEL DOSIMETRY

There are the abstracts of the works from the 10-th anniversary Russian scientific conference #Left-Pointing Double Angle Quotation Mark#Radiation protection and radiation safety in nuclear technologies#Right-Pointing Double Angle Quotation Mark# (22-25 September, 2015, Moscow - Obninsk). The problems of ionizing radiation transport in substance and radioactive substances migration in nuclear facility components, site and environment are under consideration. Many works deal with the research in the field of radiation protection and radiation safety, safety of designed, operating and shutdown radiation-dangerous facilities. The great attention is paid to radiation safety of nuclear facilities under decommissioning, software of nuclear and radiation dangerous facilities decommissioning safety justification. The problems of radioactive wastes and nuclear materials handling, radiation technologies, radiation medicine and ecology are considered

Представлены тезисы докладов 10-й юбилейной Российской научной конференции #Left-Pointing Double Angle Quotation Mark#Радиационная защита и радиационная безопасность в ядерных технологиях#Right-Pointing Double Angle Quotation Mark#, проходившей 22-25 сентября 2015 г. (Москва - Обнинск). Рассмотрены вопросы переноса ионизирующих излучений в веществе и миграции радиоактивных веществ в элементах ядерных установок, на промышленной площадке и в окружающей среде. Многие работы касаются исследований в области радиационной защиты и радиационной безопасности, обеспечения безопасности проектируемых, действующих и остановленных радиационно-опасных объектов. Большое внимание уделено радиационной безопасности при выводе ядерно-технических установок из эксплуатации, программным средствам обоснования безопасности вывода из эксплуатации ядерно- и радиационно-опасных объектов. Обсуждаются вопросы обращения с радиоактивными отходами и ядерными материалами, радиационных технологий, радиационной медицины и радиационной экологии

Hybrid neutronic calculation of radiation protection of liquid heavy metal-cooled reactor

S21

Gostev, A.L.; Zemskov, E.A.; Mel'nikov, K.G.; Suslov, I.R.; Tormyshev, I.V. (AO GNTs RF - FEhI, Obninsk (RU))

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Gibridnyj nejtronno-fizicheskij raschet radiatsionnoj zashchity yadernogo reaktora s TZhMT

p. 8-9

(RU)

3 refs., 1 fig.

GK Rosatom, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); GNTs RF - FEhI, Obninsk (RU); NOU DPO TsIPK Rosatoma, Obninsk (RU); IBRAEh RAN, Moscow (RU)

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348 p.

LIQUID METAL COOLED REACTORS; COOLANTS; RADIATION PROTECTION; MATHEMATICAL MODELS; MONTE CARLO METHOD; LEAD ALLOYS; BISMUTH ALLOYS; FAST NEUTRONS; NEUTRON FLUX; CROSS SECTIONS

Calculation of radiation fields by #Left-Pointing Double Angle Quotation Mark#weight window#Right-Pointing Double Angle Quotation Mark# iteration procedure in ASMM 10/100 kW project

S97

Chernov, S.V.; Son'ko, A.V.; Khoromskij, V.A. (GNTs RF - FEhI, Obninsk (RU))

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Raschet polej izluchenij metodom iteratsij #Left-Pointing Double Angle Quotation

Mark#vesovykh okon#Right-Pointing Double Angle Quotation Mark# v proekte ASMM 10/100 kBt

p. 9-10

(RU)

2 refs.

GK Rosatom, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); GNTs RF - FEhI, Obninsk (RU); NOU DPO TsIPK Rosatoma, Obninsk (RU); IBRAEh RAN, Moscow (RU)

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RADIATION PROTECTION; MONTE CARLO METHOD; MATHEMATICAL MODELS;
COMPUTER CODES; NUCLEAR POWER PLANTS; POWER RANGE 10-100 KW

Development and realization of data conversion algorithms from CADS-models of objects into TDMCC program format

S97

Vaneev, Yu.E.; Blokhin, P.A.; Dimitriev, A.S. (IBRAEh RAN, Moscow (RU)), e-mail: yuvan@ibrae.ac.ru

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Razrabotka i realizatsiya algoritmov konvertatsii dannykh iz SAPR-modelej ob'ektov v format programmy TDMCC

p. 11

(RU)

3 refs.

GK Rosatom, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); GNTs RF - FEhI, Obninsk (RU); NOU DPO TsIPK Rosatoma, Obninsk (RU); IBRAEh RAN, Moscow (RU)

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348 p.

COMPUTER-AIDED DESIGN; MATHEMATICAL MODELS; ALGORITHMS; DATA PROCESSING; T CODES; COMPUTERIZED SIMULATION

Verification of TDMCC program in respect to nuclear legacy facilities

S97

Vaneev, Yu.E.; Blokhin, P.A. (IBRAEh RAN, Moscow (RU)), e-mail: yuvan@ibrae.ac.ru

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radiatsionnaya bezopasnost' v yadernykh tekhnologiyakh
Verifikatsiya programmy TDMCC primenitel'no k ob'ektam yadernogo naslediya

p. 12

(RU)

5 refs.

GK Rosatom, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); GNTs RF - FEhI, Obninsk (RU); NOU DPO TsIPK Rosatoma, Obninsk (RU); IBRAEh RAN, Moscow (RU)

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348 p.

T CODES; VERIFICATION; BENCHMARKS; EQUIVALENT RADIATION DOSES

Modification of GAMOS software package for solving the problem of space nuclear power facilities radiation protection

S97

Ekhlakov, I.A.; Pyshko, A.P. (AO GNTs RF - FEhI im. A.I. Lejpunskogo, Obninsk (RU)), e-mail: firefly1248@mail.com, pyshko@ippe.ru

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Modifikatsiya programmnoogo kompleksa GAMOS dlya resheniya zadach radiatsionnoj zashchity kosmicheskikh YaEhU

p. 13

(RU)

GK Rosatom, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); GNTs RF - FEhI, Obninsk (RU); NOU DPO TsIPK Rosatoma, Obninsk (RU); IBRAEh RAN, Moscow (RU)

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348 p.

NUCLEAR FACILITIES; SPACE VEHICLES; G CODES; RADIATION PROTECTION; NEUTRON FLUX; REACTOR CORES; COMPARATIVE EVALUATIONS

Automated search technique of radiation shielding optimal arrangement

S97

Ekhlakov, I.A.; Pyshko, A.P. (AO GNTs RF - FEhI im. A.I. Lejpunskogo, Obninsk (RU)), e-mail:

firefly1248@mail.com, pyshko@ippe.ru

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Avtomatizirovannaya metodika poiska optimal'noj komponovki radiatsionnoj zashchity p. 14-15

(RU)

GK Rosatom, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); GNTs RF - FEhI, Obninsk (RU); NOU DPO TsIPK Rosatoma, Obninsk (RU); IBRAEh RAN, Moscow (RU)

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348 p.

RADIATION PROTECTION; COMPUTER CODES; SPACE VEHICLES; RADIATIONS; ALGORITHMS; ACCURACY; MATHEMATICAL MODELS

Calculational technique of contribution current using #Left-Pointing Double Angle Quotation Mark#black body#Right-Pointing Double Angle Quotation Mark# in the problems of space nuclear power units radiation protection S97

Ekhlakov, I.A.; Pyshko, A.P. (AO GNTS RF - FEhI im. A.I. Lejpunskogo, Obninsk (RU)), e-mail: firefly1248@mail.com, pyshko@ippe.ru

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Metod rascheta toka kontributonov s ispol'zovaniem #Left-Pointing Double Angle Quotation Mark#chernogo tela#Right-Pointing Double Angle Quotation Mark# v zadachakh radiatsionnoj zashchity KYaEhU

p. 15-16

(RU)

GK Rosatom, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); GNTs RF - FEhI, Obninsk (RU); NOU DPO TsIPK Rosatoma, Obninsk (RU); IBRAEh RAN, Moscow (RU)

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MATHEMATICAL MODELS; ELECTRIC CURRENTS; NEUTRON FLUX; SPACE VEHICLES; REACTORS; RADIATION PROTECTION

Solution of radiation protection problems on nonstructured tetrahedral grids
S97

Bass, L.P.; Kokonkov, N.I.; Nikolaeva, O.V. (IPM im. M.V. Keldysha RAN, Moscow (RU)), e-mail: bass@kiam.ru, kknkoff@gmail.com, nika@kiam.ru; Kuznetsov, V.S. (NITs Kurchatovskij Inst., Moscow (RU)), e-mail: lri@yandex.ru

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Reshenie zadach radiatsionnoj zashchity na nestrukturovannykh tetraedricheskikh setkakh
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Voloshchenko, A.M.; Russkov, A.A. (IPM im. M.V. Keldysha RAN, Moscow (RU)), e-mail: volosch@kiam.ru, russkov@inbox.ru

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Grabezhnoj, V.A.; Lomakov, G.B.; Popov, Eh.P.; Tykleeva, K.V. (GNTs RF - FEhI, Obninsk (RU)), e-mail: bnaab@ippe.ru

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Vasyukhno, V.P.; Dikareva, O.F.; Trusova, E.A. (AO NIKIEhT, Moscow (RU)), e-mail: trusova_ea@nikiet.ru

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Andreev, N.G.; Vavilkin, V.N. (Opytnoe Konstruktorskoe Byuro Mashinostroeniya im. I.I. Afrikantova, Nizhnij Novgorod (RU)), e-mail: okbm@okbm.nnov.ru

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Vlaskin, G.N.; Khomyakov, Yu.S. (ITTsP PRORYV, Moscow (RU)), e-mail: vgn@proryv2020.ru; Bulanenko, V.I. (GNTs RF - FEhI, Obninsk (RU))

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Vlaskin, G.N.; Khomyakov, Yu.S. (ITTsP PRORYV, Moscow (RU)), e-mail: vgn@proryv2020.ru

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Degtyarev, I.I.; Novoskol'tsev, F.N.; Lyashenko, O.A.; Gulina, E.V.; Morozova, L.V. (FGBU GNTs RF - IFVEh, Protvino (RU)), e-mail: Igor.Degtyarev@ihep.ru

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Filippov, M.F.; Semenov, V.N.; Tsaun, S.V. (IBRAEh RAN, Moscow (RU)), e-mail: philippov@ibrae.ac.ru

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Bezlepkin, V.V.; Karaseva, M.A.; Frolov, A.S.; Kharchenko, E.V. (AO Atomproekt, Sankt-Peterburg (RU)), e-mail: EVKHarchenko@atomporekt.com

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Dzama, D.V.; Semenov, V.N.; Sorokovikova, O.S. (IBRAEh RAN, Moscow (RU)), e-mail: diman_sw@mail.ru

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Arutyunyan, R.V.; Semenov, V.N.; Sorokovikova, O.S.; Pripachkin, D.A.; Dzama, D.V.; Nikiforov, V.S. (IBRAEh RAN, Moscow (RU)); Rubinshtejn, K.G.; Ignatov, R.Yu.; Smirnova, M.M.; Gubenko, I.M. (IBRAEh RAN, Moscow (RU); Gidromettsentr Rossii, Moscow (RU)), e-mail: prad@ibrae.ac.ru

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Serebraykov, B.E. (FGBU GNTs FMBTs im. A.I. Burnazyana FMBA Rossii, Moscow (RU)); Ivanov, E.A.; Pyrkov, I.V.; Korotkov, A.S. (OAO VNIIEhS, Moscow (RU)), e-mail: bserebr@yandex.ru

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Doil'nitsyna, V.V.; Sobko, A.A. (OAO RAOPROEKT, Sankt-Peterburg (RU)), e-mail: vvdoinitcvna@raoproekt.ru

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Anokhina, K.A. (MFTI, Dolgoprudnyj (RU)); Matveev, L.V. (MFTI, Dolgoprudnyj (RU)); IBRAEh RAN, Moscow (RU)), e-mail: matweev@ibrae.ac.ru

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Kuznetsov, M. (MFTI, Dolgoprudnyj (RU)); Matveev, L. (MFTI, Dolgoprudnyj (RU)); IBRAEh RAN, Moscow (RU)), e-mail: Matweev@ibrae.ac.ru

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Avetisyan, A.R.; Arutyunyan, R.V.; Kondratenko, P.S.; Matveev, L.V. (IBRAEh RAN, Moscow (RU); MFTI, Dolgoprudnyj (RU)), e-mail: avetis@ibrae.ac.ru

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Shandala, N.K.; Seregin, V.A.; Semenova, M.P.; Filonova, A.A.; Kiselev, S.M. (FGBU Gosudarstvennyj Nauchnyj Tsentr Rossijskoj Federatsii - Federal'nyj Meditsinskij Biofizicheskij Tsentr im. A.I. Burnazyana FMBA Rossii, Moscow (RU)), e-mail: anfl@mail.ru; Sneve, M. (Gosudarstvennoe Upravlenie Norvegii po Yaderno-Radiatsionnoj Bezopasnosti, Oslo (NO))

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Karev, A.E.; Tsov'yanov, A.G.; Shinkarev, S.M. (FGBU GNTs FMBTs im. A.I. Burnazyana FMBA Rossii, Moscow (RU)); Fertman, D.E. (OAO SNIIP, Moscow (RU))

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Karev, A.E.; Tsov'yanov, A.G.; Shinkarev, S.M. (FGBU GNTs FMBTs im. A.I. Burnazyana FMBA Rossii, Moscow (RU)); Rizin, A.I.; Fertman, D.E.; Tsoj, N.V. (OAO SNIIP, Moscow (RU)); Chebyshev, S.B. (OAO Atomenergomash, Moscow (RU))

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Lunev, A.S. (FGBU GNTs FMBTs im. A.I. Burnazyana FMBA Rossii, Moscow (RU); FGBOU VPO MGAVMiB im. K.I. Skryabina, Moscow (RU)); Klement'eva, O.E.; Kodina, G.E. (FGBU GNTs FMBTs im. A.I. Burnazyana FMBA Rossii, Moscow (RU)); Lysenko, N.P. (FGBOU VPO MGAVMiB im. K.I. Skryabina, Moscow (RU)), e-mail: l5h33@rambler.ru

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Rozhdestvenskij, L.M.; Mikhajlov, V.F.; Osipov, A.N. (FGBU GNTs FMBTs im. A.I. Burnazyana FMBA Rossii, Moscow (RU)), e-mail: lemrod@mail.ru

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Rybin, A.A.; Ryaskova, M.V.; Serebryakov, V.V. (AO GNTs NIAR, Dimitrovgrad (RU))

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Madeev, V.G.; Uksusov, E.I.; Cheshigin, I.V.; Badretdinov, T.M.; Degtyarev, A.S.; Kozhevnikov, A.N. (NITs Kurchatovskij Inst., Moscow (RU)), e-mail: madeev_vg@nrcki.ru

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Gantsovskij, P.P.; Tsov'yanov, A.G.; Karev, A.E. (FGBU GNTs FMBTs im. A.I. Burnazyana, Moscow (RU)); Alekseev, A.G. (OOO API, Protvino (RU))

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Britvich, G.I.; Kostin, M.Yu.; Sukhikh, A.V.; Chernichenko, S.K.; Yanovich, A.A. (FGBU GNTs IFVEh NITs Kurchatovskij Inst., Protvino (RU)), e-mail: britvich@ihep.ru

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Pavlyuk, A.O.; Bospala, E.V.; Kotlyarevskij, S.G.; Izmet'ev, A.M.; Markov, S.A. (AO ODTs UGR, Seversk (RU)), e-mail: bospala_evgeny@mail.ru

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Avaev, V.N.; Vasyukhno, V.P.; Vasyukhno, N.V.; Os'kin, A.V.; Khacheresov, G.A. (AO NIKIEhT, Moscow (RU)), e-mail: vasyukhno@nikiet.ru

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Mazur, I.K.; Kudrin, I.D.; Kryuchkov, V.P.; Chizhov, K.A.; Simakov, A.V.; Tsov'yanov, A.G. (FGBU GNTs FMBTs im. A.I. Burnazyana FMBA Rossii, Moscow (RU)), e-mail:

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Chernyshev, O.V. (FGUP PO Mayak, Ozersk (RU))

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Romanov, V.V. (Federal'noe Mediko-Biologicheskoe Agentstvo, Moscow (RU)); Shandala, N.K.; Kiselev, S.M.; Titov, A.V. (FGBU Gosudarstvennyj Nauchnyj Tsentr Rossijskoj Federatsii - Federal'nyj Meditsinskij Biofizicheskij Tsentr im. A.I. Burnazyana, Moscow (RU)), e-mail: titov_fmcb@mail.ru

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Gavrilov, P.M.; Merkulov, I.A.; Kravchenko, V.A.; Matselya, V.I.; Seelev, I.N. (FGUP Gorno-Khimicheskij Kombinat, Zheleznogorsk (RU))

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*Chernov, V.A.; Nikolaev, S.A.; Volkov, V.S.; Masterov, A.V.; Isaev, S.G.; Kuzin, N.V. (GNTs RF
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*Semenov, S.G.; Volkovich, A.G.; Potapov, V.N.; Stepanov, V.E.; Smirnov, S.V.; Chesnokov, A.V.
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Ermakov, A.I.; Semenovikh, S.V. (AO FTsYaRB, Moscow (RU))

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Blokhin, P.A.; Vaneev, Yu.E. (IBRAEh, Moscow (RU)), e-mail: blokhin@ibrae.ac.ru

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Snigirev, E.V.; Kurachenko, Yu.A. (GNTs RF - FEhI, Obninsk (RU)), e-mail:

evgsnigirev@yandex.ru; Aleksandrova, O.P. (IATEh NIYaU MIFI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk (RU))

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Fedorova, A.V.; Nikitin, A.I. (FGBU NPO Tajfun, Obninsk (RU)), e-mail: fedorovaav@bk.ru; Matushevich, E.S. (IAEh NIYaU MIFI, Obninsk (RU))

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Latynova, N.E.; Latynov, V.L. (NOU DPO TsIPK Rosatoma, Obninsk (RU)), e-mail:

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Latynov, V.L.; Kritskij, R.O.; Artisyuk, V.V. (NOU DPO TsIPK Rosatoma, Obninsk (RU)), e-mail: VLLatynov@rosatom-cipk.ru, kritskiy.roman@scicet.ru, VVArtisyuk@rosatom-cipk.ru; Ulanov, D.V. (Goskorporatsiya Rosatom, Moscow (RU)), e-mail: DVUlanov@rosatom.ru

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Shandala, N.K.; Titov, A.V.; Seregin, V.A.; Kiselev, S.M.; Metlyaev, E.G.; Isaev, D.V.; Filonova, A.A.; Starinskij, V.G. (Federal'nyj Meditsinskij Biofizicheskij Tsentri im. A.I. Burnazyana, Moscow (RU)); Khokhlova, E.A. (Regional'noe Upravlenie nom. 107 FMBA Rossii, Krasnokamensk (RU)), e-mail: sergbio@gmail.com

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Isaev, D.V.; Shandala, N.K.; Titov, A.V.; Novikova, N.Ya.; Kiselev, S.M.; Seregin, V.A. (FGBU Gosudarstvennyj Nauchnyj Tsentri Rossijskoj Federatsii - Federal'nyj Meditsinskij Biofizicheskij Tsentri im. A.I. Burnazyana FMBA Rossii, Moscow (RU)); Zolotukhina, S.B.; Zhuravleva, L.A. (Tsentri Gigieny i Ehpideologii nom. 107 FMBA Rossii, Krasnokamensk (RU)); Marennij, A.M. (FGUP Nauchno-Tekhnicheskij Tsentri Radiatsionno-Khimicheskoy Bezopasnosti i Gigieny FMBA Rossii, Moscow (RU)), e-mail: strategik@bk.ru

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Kuchin, N.L.; Lajkin, A.I.; Mikheev, Yu.V. (Krylovskij Gosudarstvennyj Nauchnyj Tsentri, Sankt-Peterburg (RU)), e-mail: yurym59@mail.ru

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Polyudin, A.L. (FGUP RFYaTs - VNIITF im. akad. E.I. Zababakhina, Snezhinsk (RU))

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Epimakhov, V.G. (FGBNU VNII Radiologii i Agroekologii, Obninsk (RU)), e-mail: epimakhov.vg@gmail.com

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Koz'min, G.V.; Sanzharova, N.I. (FGBNU VNIIRAEh, Obninsk (RU)), e-mail:

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Vazina, A.A. (FGBUN Inst. Teoreticheskoy i Eksperimental'noj Biofiziki RAN, Pushchino (RU)), e-mail: vazina@mail.ru

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Loj, N.N.; Sanzharova, N.I. (FGBNU VNIIRAEh, Obninsk (RU)); Kuznetsov, A.A.; Molin, A.A. (OOO Tsent Atommed, Moscow (RU)); Vinokurov, V.A. (FGBOU VPO RGUNG im. I.M. Gubkina, Moscow (RU)), e-mail: loy.nad@yandex.ru

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Tikhonov, A.V. (FGBNU VNII Radiologii i Agroehkologii RASKhN, Obninsk (RU)), e-mail: ti_@list.ru

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*Zyakun, A.M. (FGBUN IBFM im. G.K. Skryabina RAN, Pushchino (RU)); Koz'min, G.V.
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*Sarukhanov, V.Ya.; Koz'min, G.V.; Kobyalko, V.O.; Frolova, N.A. (FGBNU VNIIRAEh, Obninsk (RU)); Lykov, I.N. (KGU im. K.Eh. Tsiolkovskogo, Kaluga (RU)), e-mail:
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Komarova, L.N.; Lyapunova, E.R.; Amosova, N.V. (IATEh NIYaU MIFI, Obninsk (RU)), e-mail: komarova_1411@mail.ru

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Pavlova, N.N.; Kylina, N.S. (IATEh NIYaU MIFI, Obninsk(RU)), e-mail: nadpavl@yandex.ru

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Biotestirovanie vodnykh prob iz kontrol'nykh skvazhin khranilishcha radioaktivnykh otkhodov g. Obninska

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Vasil'ev, D.V.; Kuz'menkov, A.G.; Dikareva, N.S.; Geras'kin, S.A. (FGBNU VNIIRAEh, Obninsk (RU)), e-mail: treworqwert@mail.ru

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Kuchin, N.L.; Lajkin, A.I.; Mikheev, Yu.V. (Krylovskij Gosudarstvennyj Nauchnyj Tsentr, Sankt-Peterburg (RU)), e-mail: yurym59@mail.ru

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Assessment of reparative status of specialists deal with different radiations at delayed terms

S63

Nikanorova, E.A.; Nagiba, V.I.; Ivanov, K.Yu.; Medvedev, Ya.I. (FGUP Rossijskij Federal'nyj Yadernyj Tsentr - Vserossijskij NII Ehksperimental'noj Fiziki, Sarov (RU)), e-mail: gane@orb2.vniief.ru

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DNA REPAIR; BLOOD; DELAYED RADIATION EFFECTS; PERSONNEL; GAMMA RADIATION; BETA PARTICLES; RADIATION DOSES; TIME DEPENDENCE; HUMAN POPULATIONS; COMPARATIVE EVALUATIONS; CORRELATIONS; CHROMOSOMAL ABERRATIONS

Numerical simulation and parametrization of absorbed dose fields at dosimetric planning of liver malignant neoplasms brachytherapy by microspheres with ⁹⁰Y

S62

Egorov, V.V. (OOO NPP, Moscow (RU)); Snigirev, E.V. (GNTs RF - FEhI, Obninsk (RU)); Aleksandrova, O.P.; Matusевич, E.S. (IATEh NIYaU MIFI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk (RU)), e-mail: vlad.v.egorov@gmail.com

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S62

Egorov, V.V. (OOO NPP, Moscow (RU)); Snigirev, E.V. (GNTs RF - FEhI, Obninsk (RU)); Aleksandrova, O.P. (IATEh NIYaU MIFI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk (RU)), e-mail: vlad.v.egorov@gmail.com

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DOSES

Voxel anthropomorphic phantom for radiological survey tasks

S61

Moiseenko, D.N. (NPO TAJFUN, Obninsk (RU)); Kurachenko, Yu.A. (GNTs RF - FEhI, Obninsk (RU)), e-mail: forallusers@mail.ru

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SPECT/CT and planar scintigraphic and dosimetric studies during clinical investigations of radiopharmaceutical ^{18}Re in bone metastases therapy

S62

Aleksandrova, O.P.; Kurachenko, Yu.A. (IATEh NIYaU MIFI, Obninsk (RU)); Klepov, A.N.

(NPP DIONIS, Obninsk(RU)); Smolyarchuk, M.A. (OOO NPP, Moscow (RU)); Kochetova, T.Yu.; Krylov, V.V.; Spichenkova, O.N.; Davydov, G.A. (MRNTs im. A.F. Tsyba, Obninsk (RU)); Kanygin, V.V. (FMBA Minzdrava RF, Moscow (RU)), e-mail: oksana-dolya@mail.ru

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OFEhKT/KT i planarnye stsintigraficheskie i dozimetricheskie issledovaniya v protsesse klinicheskikh ispytaniy RFP #Left-Pointing Double Angle Quotation Mark#Fosforen, "1"8"8Re#Right-Pointing Double Angle Quotation Mark# v terapii kostnykh metastazov p. 283-284

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Physico-mathematical simulation and dosimetric analysis of liver tumor brachytherapy by nanostructured microspheres with ^{223}Rn

S62

Kon'kov, A.V.; Androsenko, P.A.; Aleksandrova, O.P. (IATEh NIYaU MIFI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk (RU)), e-mail: oksana-dolya@mail.ru

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Fiziko-matematicheskoe modelirovanie i dozimetricheskij analiz brakhiterapii opukholej pecheni nanostrukturirovannymi mikrosferami s ^{223}Rn

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RADIOPHARMACEUTICALS; PHOSPHORUS 32; DOSIMETRY; RADIATION DOSES;
SPATIAL DISTRIBUTION; MATHEMATICAL MODELS; MONTE CARLO METHOD;
RADIOSENSITIVITY

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S62

Aleksandrova, O.P. (IATEh NIYaU MIFI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk (RU)); Petriev, V.M.; Skvortsov, V.G.; Shiryayeva, V.K. (MRNTs im. A.F. Tsyba, Obninsk (RU)); Kanygin, V.V. (FMBA Minzdrava RF, Moscow (RU)), e-mail: oksana-dolya@mail.ru

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ANIMAL TISSUES; COMPUTER CODES; RADIONUCLIDE KINETICS

Dosimetric analysis of radioiodine therapy of patients with thyroid differential carcinoma with lung metastasis on the base of compartment simulation of ^{131}I pharmacokinetics

S62

Aleksandrova, O.P. (IATEh NIYaU MIFI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk

(RU)); Rodichev, A.A. (MRNTs im. A.F. Tsyba, Obninsk (RU)), e-mail: oksana-dolya@mail.ru

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Dozimetriceskij analiz radiojodterapii bol'nykh differentsirovannym rakom shchitovidnoj zhelezy s metastazami v legkie na baze kompartmentnogo modelirovaniya farmakokinetiki ^{131}I

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S62

Aleksandrova, O.P. (IATEh NIYaU MIFI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk (RU)), e-mail: oksana-dolya@mail.ru

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DOSIMETRY; MATHEMATICAL MODELS; NONLINEAR PROGRAMMING;
EFFICIENCY; PATIENTS

External irradiation of personnel and wider public during and after brachytherapy process of prostate cancer by ^{125}I microspheres

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Kurachenko, Yu.A. (GNTs RF - FEhI, Obninsk (RU)); Aleksandrova, O.P. (IATEh NIYaU MIFI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk (RU)), e-mail: yukurachenko@ippe.ru

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Kurachenko, Yu.A. (GNTs RF - FEhI, Obninsk (RU)); Aleksandrova, O.P. (IATEh NIYaU MIFI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk (RU)), e-mail: yukurachenko@ippe.ru

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Kurachenko, Yu.A. (GNTs RF - FEhI, Obninsk (RU)); Aleksandrova, O.P.; Matusevich, E.S. (IATEh NIYaU MIFI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk (RU)), e-mail: yukurachenko@ippe.ru

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Kurachenko, Yu.A.; Grudzevich, O.T. (GNTs RF - FEhI, Obninsk (RU)); Matushevich, E.S. (IATEh NIYaU MIFI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk (RU)); Voznesenskij, N.K. (GKB nom. 8, Obninsk (RU)), e-mail: yukurachenko@ippe.ru

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Kurachenko, Yu.A. (GNTs RF - FEhI, Obninsk (RU)); Matusевич, E.S. (IATEh NIYaU MIFI, Obninsk (RU)), e-mail: yukurachenko@ippe.ru

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Kurachenko, Yu.A. (GNTs RF - FEhI, Obninsk (RU)); Fadeev, M.N. (IATEh NIYaU MIFI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk (RU)), e-mail: yukurachenko@ippe.ru

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RADIATION DOSES; NEUTRON FLUENCE; MATHEMATICAL MODELS

Assessment and analysis of energy balance of radiations during brachytherapy by ^{125}I microspheres

S62

Kurachenko, Yu.A. (GNTs RF - FEhI, Obninsk (RU)); Aleksandrova, O.P. (IATEh NIYaU MIFI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk (RU)), e-mail: yukurachenko@ippe.ru

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ENERGY BALANCE

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S62

Kurachenko, Yu.A. (GNTs RF - FEhI, Obninsk (RU)); Aleksandrova, O.P. (IATEh NIYaU MIFI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk (RU)), e-mail: yukurachenko@ippe.ru

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S61

Kurachenko, Yu.A. (GNTs RF - FEhI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk (RU)); Aleksandrova, O.P. (IATEh NIYaU MIFI, Obninsk (RU)); Kalistaya, M.S. (FMBA Minzdrava RF, Moscow (RU)), e-mail: yukurachenko@ippe.ru

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S62

Kurachenko, Yu.A. (GNTs RF - FEhI, Obninsk (RU)); Aleksandrova, O.P. (IATEh NIYaU MIFI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk (RU)), e-mail: yukurachenko@ippe.ru

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BRACHYTHERAPY; DOSIMETRY; PLANNING; NEOPLASMS; MATHEMATICAL SOLUTIONS

Optimization of external radiation protection parameters of PET-center on the base of cyclotron with integrated biological shield

S61

Kurachenko, Yu.A. (GNTs RF - FEhI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk (RU)); Aleksandrova, O.P. (IATEh NIYaU MIFI, Obninsk (RU)); Kalistaya, M.S.; Dubinkin, D.O. (FMBA Minzdrava RF, Moscow (RU)), e-mail: yukurachenko@ippe.ru

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Kurachenko, Yu.A. (GNTs RF - FEhI, Obninsk (RU)); Aleksandrova, O.P. (IATEh NIYaU MIFI, Obninsk (RU)); Klepov, A.N. (NPP DIONIS, Obninsk (RU)); Popov, S.V. (OOO NPP, Moscow (RU)), e-mail: yukurachenko@ippe.ru

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Quantitative characteristics of PO Mayak personnel descendants cohort S63

Sosnina, S.F.; Sokol'nikov, M.Eh.; Okatenko, P.V. (FGUP YuUrIBF, Ozersk (RU)), e-mail: sosnina@subi.su

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MAYAK PLANT; PERSONNEL; DELAYED RADIATION EFFECTS; GENETIC RADIATION EFFECTS; AGE GROUPS; IRRADIATION; RADIATION DOSES; NEOPLASMS; CORRELATIONS

Radiation doses of personnel of organizations and population of the territories serviced by FMBA of Russia

S61

Kosterev, V.V.; Tsov'yanov, A.G.; Sivenkov, A.G.; Bragin, Yu.N.; Kriminskij, A.A. (FGBU GNTs FMBTs im. A.I. Burnazyana FMBA Rossii, Moscow (RU)), e-mail: vvkosterev@mephi.ru, atsovyan@mail.ru

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Radiation safety in radiotherapy with the use electron accelerators

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Nurlybaev, K.; Bushmanov, A.Yu.; Tsov'yanov, A.G. (FGBU GNTs FMBTs im. A.I. Burnazyana FMBA Rossii, Moscow (RU)); Martynyuk, Yu.N.; Karakash, A.I.; Sinnikov, L.L. (OOO NPP Doza, Moscow (RU)); Loginova, S.V. (RMAPO, Moscow (RU)), e-mail: kubesh@doza.ru

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Positron annihilation temporal spectroscopy technique as a rapid method of carcinogens recognition

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Pivtsaev, A.A.; Razov, V.I. (Dal'nevostochnyj Federal'nyj Univ., Vladivostok (RU)), e-mail: aleksey_pivtsaev@mail.ru

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LEADING ABSTRACT; MEETINGS; METALLURGY; PHYSICAL PROPERTIES; CHEMICAL PROPERTIES; MATERIALS WORKING; NANOSTRUCTURES; NANOTECHNOLOGY; COMPOSITE MATERIALS; FILMS; COATINGS

The conference covers the whole range of issues of metallurgy and materials, including nanomaterials and nanotechnologies, advanced materials, resource-saving processes and technologies. At the conference the work of sections for oral and poster presentations on the following topics was conducted: 1. Structure and properties of high strength and nanostructured non-metallic and composite materials; 2. The development of methods for studying and modeling the structure and properties of materials and nanomaterials; 3. Functional ceramic and composite nanomaterials; 4. Development of physicochemical bases and technologies of new processes for production and molding of powder materials and nanomaterials; 5. Physicochemical basis for obtaining and processing of advanced inorganic materials; 6. Physical chemistry and technologies of energy-, resource-saving and environmentally safe processes for production of ferrous, non-ferrous and rare metals; 7. Prospective composite coatings and nanostructured films of functional

purpose; 8. Physical and chemical bases of new molding processes and pressure treatment of materials and nanomaterials

Конференция затрагивает весь комплекс вопросов металлургии и материаловедения, включая наноматериалы и нанотехнологии, перспективные материалы, ресурсосберегающие процессы и технологии. В рамках конференции проведена работа секций для устных и стендовых докладов по следующим тематикам: 1. Структура и свойства высокопрочных и наноструктурных металлических и композиционных материалов; 2. Развитие методов исследования и моделирования структуры и свойств материалов и наноматериалов; 3. Функциональные керамические и композиционные наноматериалы; 4. Развитие физико-химических основ и технологий новых процессов получения и формования порошковых материалов и наноматериалов; 5. Физико-химические основы получения и обработки перспективных неорганических материалов; 6. Физико-химия и технологии энерго-, ресурсосберегающих и экологически безопасных процессов получения черных, цветных и редких металлов; 7. Перспективные композиционные покрытия и наноструктурные пленки функционального назначения; 8. Физико-химические основы новых процессов формообразования и обработки давлением материалов и наноматериалов

Investigating effect of properties of electrode powder wire thermite filler on the weld quality

S36

Abashkin, E.E. (FGBUN Inst. Mashinovedeniya i Metallurgii DVO RAN, Komsomol'sk-na-Amure (RU)), e-mail: abashkine@mail.ru

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Issledovanie vliyaniya svojstv termitnogo napolnitelya ehlektrodoj poroshkovej provoloki na kachestvo svarnogo shva

p. 21-22

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The effect of nanoparticles small additions on the structure, density and heat resistance of alumino composites

S37

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The increase in heat resistance and thermal stability of alloys based on #gamma#-TiAl at a temperature of 900#Degree Sign# C

S36

Arnautov, D.O. (NITU MISiS, Moscow (RU)), e-mail: dao93@bk.ru

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Baikin, A.S. (IMET RAN, Moscow (RU)), e-mail: baikinas@mail.ru

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Gerov, M.V. (IMET RAN, Moscow (RU)), e-mail: zodom@mail.ru

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BINARY ALLOY SYSTEMS; ALUMINIUM BASE ALLOYS; SILICON ALLOYS; CHEMICAL COMPOSITION; PRESSURE RANGE MEGA PA; TEMPERATURE RANGE 0273-0400 K; MICROSTRUCTURE; CRYSTAL LATTICES

The structure and properties of mild low alloyed steel after thermomechanical processing

S36

Del'gado Rejna, S.Yu. (Inst. Fiziki Metallov UrO RAN, Ekaterinburg (RU)), e-mail: delgadosvetlana@gmail.com

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1 tab.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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CHROMIUM-NICKEL-MOLYBDENUM STEELS; CHEMICAL COMPOSITION;
THERMOMECHANICAL TREATMENTS; NANOSTRUCTURES; TEMPERING; CRYSTAL
STRUCTURE; YIELD STRENGTH

Effect of diffusion processes on the corrosion resistance to local types of corrosion of multilayer materials

S36

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1 fig.

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STEEL-CR18NI10TI; MANGANESE ALLOYS; EXPLOSION WELDING; ANNEALING;
HARDNESS; BIMETALS; TEMPERING; TEMPERATURE RANGE 0400-1000 K;
TEMPERATURE RANGE 1000-4000 K; DEFORMATION

Structural aspects of the quality of the new generation steels for building structures

S36

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AUSTENITIC STEELS; GRAIN ORIENTATION; SCANNING ELECTRON MICROSCOPY; GRAIN SIZE; NANOSTRUCTURES; GRAIN BOUNDARIES

About structural changes of cast synthetic mineral alloys when exposed to static and dynamic loads

S36

Ignatova, A.M. (Permskij Natsional'nyj Issledovatel'skij Politekhnikeskij Univ., Perm' (RU)), e-mail: iampstu@gmail.com

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11 refs.

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MELTING; CASTING; ALLOYS; SYNTHETIC MATERIALS; ROCKS; SOLID WASTES;
DYNAMIC LOADS; STATIC LOADS; DISLOCATIONS; FAILURES

The study of fatigue properties of TiNi alloy

S36

Kaplan, M.A. (IMET RAN, Moscow (RU)), e-mail: Misha279@yandex.ru

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NICKEL ALLOYS; TITANIUM ALLOYS; WEAR; ANNEALING; TEMPERATURE RANGE
0400-1000 K; THERMAL CYCLING; THERMAL FATIGUE

Production technology of composite materials based on aluminum, dispersion-strengthened by nanoparticles SiO₂ and W in the non-stationary field

S77

Kaplanskiy, Yu.Yu. (NITU MISiS, Kafedra PMiFP, Moscow (RU)), e-mail: ykaplansky@mail.ru

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Fiziko-khimiya i tekhnologiya neorganicheskikh materialov (s mezhdunarodnym uchastiem)
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dispersnymi nanochastitsami SiO₂ i W v nestatsionarnom pole

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MATRIX MATERIALS; ALUMINIUM; NANOPARTICLES; NANOTECHNOLOGY

Compound of steel with aluminum and aluminum alloys in the liquid phase (review)

S36

Kovalev, V.V. (MGU im. N.Eh. Baumana, Moscow (RU)), e-mail: vvkovalev90@gmail.com

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(RU)

7 refs.

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Effect of gold impurities on the melting point and the processes of formation of the internal structure of copper nanoclusters

S36

Komzolakov, A.V. (Khakasskij Gosudarstvennyj Univ. im. N.F. Katanova, Abakan (RU)), e-mail: alekseikomzolakov@mail.ru

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5 refs., 2 figs.

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IMPURITIES; MELTING POINTS; NANOSTRUCTURES; NANOMATERIALS; METALLURGICAL EFFECTS; COMPUTERIZED SIMULATION; ATOMIC CLUSTERS; COPPER ALLOYS; GOLD ALLOYS

Regression analysis of experimental data

S36

Kondrashina, A.V. (Moskovskij Gosudarstvennyj Mashinostroitel'nyj Univ. (MAMI), Moscow (RU)), e-mail: solnishkomoe83@mail.ru

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Regressionnyj analiz ehksperimental'nykh dannyxh

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Getting biocompatible Ti-Nb alloys with low-gas-forming impurities

S36

Konopatskij, A.S. (NITU MISiS, Moscow (RU)), e-mail: ankonopatsky@gmail.com

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Poluchenie biosovmestimykh Ti-Nb splavov s nuzkim soderzhaniem gazoobrazuyushchikh primesej

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1 fig., 2 tabs.

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The study of the mechanical properties of nitinol

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NICKEL ALLOYS; TITANIUM ALLOYS; NANOMATERIALS; STATIC LOADS; TEMPERATURE RANGE 0400-1000 K; PLASTICITY; ANNEALING; FRACTURE PROPERTIES

Effect of the shock-wave loading on the mechanical and thermomechanical characteristics of Ti-Ni-Nb and Ti-Ni-Nb-Zr alloys with shape memory

S36

Lar'kina, Yu.A. (FGUP RFYaTs-VNIIEhF, Sarov (RU)), e-mail: popov@astra.vniief.ru

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Vliyanie udarno-volnovogo nagruzheniya na mekhanicheskie i termomekhanicheskie kharakteristiki splavov sistemy Ti-Ni-Nb i Ti-Ni-Nb-Zr s pamyat'yu formy

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2 refs., 2 tabs.

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SHAPE MEMORY EFFECT; TITANIUM ALLOYS; NICKEL ALLOYS; NIOBIUM ALLOYS; ZIRCONIUM ALLOYS; DYNAMIC LOADS; DEFORMATION; MECHANICAL TESTS; THERMOMECHANICAL TREATMENTS; ANNEALING

**The study of dielectric characteristics of radio absorbing composite material
#Left-Pointing Double Angle Quotation Mark#SiC-steel#Right-Pointing Double
Angle Quotation Mark#**

S36

Leonov, A.V. (IMET RAN, Moscow (RU)), e-mail: madflame@list.ru

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Issledovanie dielektricheskikh kharakteristik radiopogloshchayushshego kompozitsionnogo materiala #Left-Pointing Double Angle Quotation Mark#SiC-stal'#Right-Pointing Double Angle Quotation Mark#

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DIELECTRIC PROPERTIES; COMPOSITE MATERIALS; SILICON CARBIDES;
CHROMIUM-NICKEL STEELS; POWDERS; ANNEALING; HOT PRESSING;
ELECTROMAGNETIC RADIATION

Investigation into composition radio absorbing material #Left-Pointing Double Angle Quotation Mark#SiC-steel#Right-Pointing Double Angle Quotation Mark#

S36

Leonova, Yu.O. (IMET RAN, Moscow (RU)), e-mail: Bastka.imet@list.ru

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(RU)

1 tab.

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COMPOSITE MATERIALS; SILICON CARBIDES; ELECTROMAGNETIC RADIATION; STEELS; TEMPERATURE RANGE 1000-4000 K; TIME DEPENDENCE; FLEXURAL STRENGTH; POROSITY; DENSITY

Modification of weld metal by nano-sized refractory particles

S77

Linnik, A.A. (Moskovskij Gosudarstvennyj Tekhnicheskij Univ. im. N.Eh. Baumana, Moscow (RU)), e-mail: lurc@list.ru

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Modifitsirovanie nanorazmernymi tugoplavkimi chastitsami metalla shva

p. 67-68

(RU)

1 ref.

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NANOPARTICLES; COMPOSITE MATERIALS; MODIFICATIONS; FRACTURE PROPERTIES; WELDED JOINTS; POWDERS; METALLURGICAL EFFECTS; TEMPERATURE RANGE 0065-0273 K

Analysis of the reinforcing phase distribution in aluminum-matrix composites S36

Lopatina, Yu.A. (MGTU im. N.Eh. Baumana, Moscow (RU)), e-mail: lopatina.julia@yandex.ru

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Analiz raspredeleniya armiruyushchej fazy v alyumomatrichnykh kompozitsionnykh materialakh p. 68-69

(RU)

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Effect of heat treatment and plastic deformation on the structure and mechanical properties of nitrogen-containing steel 04Ni9Cr2N

S36

Lukin, E.I. (IMET RAN, Moscow (RU)), e-mail: flattop@yandex.ru

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p. 69-71

(RU)

2 tabs.

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NICKEL STEELS; CHROMIUM ALLOYS; NITROGEN ADDITIONS; CHEMICAL COMPOSITION; STRAINS; DEFORMATION; PLASTICITY; QUENCH HARDENING; TEMPERATURE RANGE 0273-0400 K; TEMPERATURE RANGE 1000-4000 K

The mechanical properties of alloys Mg-Sm-Tb-Zr at high temperatures

S36

Luk'yanova, E.A. (IMET RAN, Moscow (RU)), e-mail: pavlodarskaia@yandex.ru

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(RU)

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MAGNESIUM ALLOYS; SAMARIUM ALLOYS; TERBIUM ALLOYS; ZIRCONIUM ADDITIONS; CHEMICAL COMPOSITION; MICROSTRUCTURE; HOT PRESSING; TEMPERATURE RANGE 0273-0400 K; TEMPERATURE RANGE 0400-1000 K

Effect of the shock-compressed gas thermodynamic parameters on formation of compound in bimetal #Left Double Quotation Mark#steel-superalloy#Right Double Quotation Mark# by explosion welding

S36

Malakhov, A.Yu. (ISMAN, Chernogolovka (RU)), e-mail: malahov-aspirant@yandex.ru

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Vliyanie termodinamicheskikh parametrov udarno-szhatogo gaza na obrazovanie soedineniya v bimetalle #Left Double Quotation Mark#stal' - zharoprochnyj splav#Right Double Quotation Mark# svarkoj vzryvom

p. 72-73

(RU)

4 refs.

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EXPLOSION WELDING; BIMETALS; HEAT RESISTING ALLOYS; NIOBIUM ALLOYS;
NICKEL ALLOYS; COBALT ALLOYS; TENSILE PROPERTIES; NICKEL STEELS;
THERMODYNAMIC PROPERTIES

The recovery processes in alloys Fe-3%Si and Fe-50%Ni upon annealing at high magnetic fields

S36

Milyutin, V.A. (IFM UrO RAN, Ekaterinburg (RU)), e-mail: v.a.milutin@gmail.com

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p. 73-75

(RU)

5 refs., 1 fig., 1 tab.

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MAGNETIC FIELDS; IRON BASE ALLOYS; SILICON ALLOYS; NICKEL ALLOYS;
ANNEALING; RECRYSTALLIZATION; TEMPERATURE RANGE 0400-1000 K; GRAIN
ORIENTATION; MAGNETIC PROPERTIES; MICROHARDNESS

Effect of nanoscale carbon fibers on the structure and properties of the composite material from powder steel 35MnSi

S36

Mikheev, G.V. (NITU MISiS, Moscow (RU)), e-mail: gera_jrich@mail.ru

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kompozitsionnogo materiala iz poroshkovej stali 35GS

p. 75-76

(RU)

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MANGANESE ALLOYS; CARBON FIBERS; COMPOSITE MATERIALS; POWDERS; AUSTENITIC STEELS; POROSITY; NANOCOMPOSITES; CRYSTAL-PHASE TRANSFORMATIONS; SINTERING

Study of low cycle fatigue in heat-resistant martensitic steel

10Cr10Co3W2MoVNbB

S36

Mishnev, R.V. (Belgorodskij Gosudarstvennyj Natsional'nyj Issledovatel'skij Univ. (NIU BelGU), Belgorod (RU)), e-mail: mishnev91@mail.ru

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Izuchenie malotsiklovoj ustalosti zharoprochnoj martensitnoj stali 10Kh10K3V2MFBR

p. 76-77

(RU)

1 ref., 2 figs.

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MARTENSITIC STEELS; TUNGSTEN ALLOYS; VANADIUM ALLOYS; MOLYBDENUM ALLOYS; BORON ADDITIONS; COBALT ALLOYS; TEMPERATURE RANGE 0400-1000 K; DEFORMATION; FATIGUE; CRACK PROPAGATION

Features of a nitrogen-containing austenitic steel 05Cr22Mn15Ni8MoVBe solid solution quench aging

S36

Muradyan, S.O. (IMET RAN, Moscow (RU)), e-mail: muradianso@gmail.com

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p. 80-81

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2 refs., 1 fig., 1 tab.

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AUSTENITIC STEELS; CHEMICAL COMPOSITION; MICROSTRUCTURE; QUENCH AGING; SOLID SOLUTIONS; TEMPERATURE RANGE 1000-4000 K; MICROHARDNESS; TIME DEPENDENCE; WATER; AIR

Effect of heat treatment parameters on the mechanical properties of powder steels, nanosized additives alloyed

S36

Nitkin, N.M. (Moskovskij Gosudarstvennyj Mashinostroitel'nyj Univ. MAMI, Moscow (RU))

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Vliyaniye parametrov termicheskoy obrabotki na mekhanicheskie svoystva poroshkovykh stalej,
legirovannykh nanorazmernymi dobavkami

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(RU)

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POWDERS; STEELS; HEAT TREATMENTS; NANOCOMPOSITES; METALLURGICAL
EFFECTS; APPROPRIATE TECHNOLOGY; NANOSTRUCTURES

Magnetic properties of nanostructured alloys R₂Fe₁₋₄B

S36

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Fiziko-khimiya i tekhnologiya neorganicheskikh materialov (s mezhdunarodnym uchastiem)

Magnitnye svoystva nanostrukturirovannykh splavov R₂Fe₁₋₄B

p. 82-83

(RU)

1 fig.

R₂Fe₁₋₄B (R=Nd, Ho)

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MAGNETIC PROPERTIES; NANOSTRUCTURES; NEODYMIUM ALLOYS; HOLMIUM ALLOYS; BORON ADDITIONS; HYSTERESIS; QUENCH HARDENING; DEFORMATION; GRAIN SIZE

Prospective magnetocaloric compounds: influence of substitutional and interstitial atoms

S36

Politova, G.A. (IMET RAN, Moscow (RU)), e-mail: gpolitova@gmail.ru

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MAGNETIC MATERIALS; GADOLINIUM; MAGNETIC FIELDS; INTERMETALLIC COMPOUNDS; ARC WELDING; ANNEALING; HYDROGENATION; LAVES PHASES

The impact of the structural state of the titanium surface layer on the fatigue durability

S36

Popkova, Yu.F. (Inst. Fiziki Prochnosti i Materialovedeniya SO RAN, Tomsk (RU)), e-mail: yusik_p@mail.ru

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Vliyanie strukturnogo sostoyaniya poverkhnostnogo sloya na ustalostnuyu dolgovechnost' titana
p. 84-85

(RU)

1 tab.

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TITANIUM ALLOYS; NANOCOMPOSITES; NANOSTRUCTURES; HYDROGENATION; FATIGUE; STRAIN HARDENING; MICROSTRUCTURE; POROSITY

The tribological properties of wear-resistant metal composite materials, in volume reinforced by superelastic solid carbon

S36

Potapova, I.N. (IMET RAN, Moscow (RU)), e-mail: ino1985@yandex.ru

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Tribologicheskie svoystva iznosostojkikh metallicheskikh kompozitsionnykh materialov, ob"emno armirovannykh sverkhuprugim tverdym uglerodom

p. 85-86

(RU)

2 refs., 2 figs., 1 tab.

Metals: Cu, Ag, Ni, Co, Fe, Ti

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619 p.

COMPOSITE MATERIALS; REINFORCED MATERIALS; WEAR RESISTANCE;
FULLERENES; CHEMICAL COMPOSITION; YOUNG MODULUS; TRIBOLOGY

Effect of explosion welding mode on the quality of the thermostatic bimetals L63-36N near weld zone

S36

Sajkov, I.V. (Uchrezhdenie Rossijskoj Akademii Nauk Inst. Strukturnoj Makrokinetiki i Problem Materialovedeniya RAN, Chernogolovka (RU)), e-mail: revan84@mail.ru

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1 fig.

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619 p.

CLADDING; THERMAL STRESSES; TEMPERATURE CONTROL; CHEMICAL COMPOSITION; INVAR; BRASS; MECHANICAL PROPERTIES; EXPLOSION WELDING

The study of the mechanical properties of the composite material

S36

Sevost'yanov, M.A. (IMET RAN, Moscow (RU)), e-mail: cmakp@mail.ru

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(RU)

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619 p.

COMPOSITE MATERIALS; NICKEL ALLOYS; TITANIUM ALLOYS; TANTALUM ALLOYS; SPUTTERING; YIELD STRENGTH; TENSILE PROPERTIES

Technology of producing corrosion resistant NiTiTa alloy for the production of solid-state heat engines

S36

Sergienko, K.V. (IMET RAN, Moscow (RU)), e-mail: shulf@yandex.ni

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5 refs., 1 fig.

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619 p.

NICKEL ALLOYS; TITANIUM ALLOYS; TANTALUM ALLOYS; CORROSION RESISTANT ALLOYS; CORROSION RESISTANCE; NANOSTRUCTURES; CHEMICAL COMPOSITION; DRAWING

Mechanical properties of the carbon-carbon fiber composite material EPAN reinforced by nanosized fibers

S36

Slyuta, D.A. (Moskovskij Gosudarstvennyj Mashinostroitel'nyj Univ. MAMI, Moscow (RU))

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p. 90-91

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Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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619 p.

CARBON FIBERS; COMPOSITE MATERIALS; NANOSTRUCTURES; REINFORCED MATERIALS; HEAT RESISTANT MATERIALS

The structure of the titanium alloy VT6 after electroexplosive yttrium doping and subsequent electron-beam treatment

S36

Sosnin, K.V. (Sibirskij Gosudarstvennyj Industrial'nyj Univ., Novokuznetsk (RU)), e-mail: gromov@physics.sibsiu.ru

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619 p.

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The atomic and electronic structure of the boundaries Ta(Mo)/TiNi (001) and (110)

S36

Tarasov, K.Yu. (Natsional'nyj Issledovatel'skij Tomskij Gosudarstvennyj Univ., Tomsk (RU)), e-mail: konjtar@mail.ru

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Features of structural and magnetic properties of the fainmet alloy microwires obtained by Ulitovsky-Taylor method

S36

Umnova, N.V. (IMET RAN, Moscow (RU)), e-mail: molokano@imet.ac.ru

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Alloy: Fe₇₃Si₁₃B₉Nb₃Cu₁

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619 p.

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BENDING

Features of formation of a composite structure in steel R9

S36

Chernoivanenko, E.A. (Natsional'naya Metallurgicheskaya Akademiya Ukrainy, Dnepropetrovsk (UA)), e-mail: ekmovchan@yandex.ru

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5 refs., 2 figs.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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Properties of nanostructured chromo-siliconized powder materials

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Sharipzyanova, G.Kh. (Moskovskij Gosudarstvennyj Mashinostroitel'nyj Univ., Moscow (RU)), e-mail: guzel@mtw.ru

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CHROMIUM COMPOUNDS; SILICON COMPOUNDS; CHEMICAL COMPOSITION;
DIFFUSION

The structure and properties of the hot formed multicomponent composites based on aluminum

S36

Shishkina, Yu.A. (Inst. Problem Materialovedeniya im. I.N. Frantsevicha NAN Ukrainy, Kiev (UA)), e-mail: juli-unona@ukr.net

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The dependence of the powder material strength characteristics from its porous microstructure

S36

Shustov, V.S. (IMET RAN, Moscow (RU)), e-mail: Nemo_73@mail.ru

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2 refs., 2 tabs.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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619 p.

POWDERS; TITANIUM CARBIDES; MICROSTRUCTURE; POROSITY; BENDING; FLEXURAL STRENGTH; HARDNESS; TEMPERATURE RANGE 1000-4000 K; DENSITY

The structure, phase composition and mechanical properties of synthesized nanocomposites based on Fe-NbC

S36

Yazovskikh, K.A. (Fiziko-Tekhnicheskij Inst. Ural'skogo Otdeleniya Rossijskoj Akademii Nauk, Izhevsk (RU)), e-mail: yazovskikh_pti@mail.ru

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3 refs., 1 fig.

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619 p.

NANOCOMPOSITES; NIOBIUM CARBIDES; POWDERS; COMPACTING;
COMPACTORS; NANOSTRUCTURES; IRON CARBIDES

**Determination of aluminum content in the iron-based alloys by AES with ICP
S36**

Andreeva, N.A. (IMET RAN, Moscow (RU)), e-mail: andreeva150388@mail.ru

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1 tab.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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IRON BASE ALLOYS; ALUMINIUM ALLOYS; ICP MASS SPECTROSCOPY; EMISSION SPECTROSCOPY; QUANTITATIVE CHEMICAL ANALYSIS; NANOSTRUCTURES; CORUNDUM; DISSOLUTION; SOLUTIONS

**The study of mechanical properties of materials by indentation using
microhardness tester**

S36

Bargan, D.V. (Yuzhno-Ukrainskij Natsional'nyj Pedagogicheskij Univ. im. K.L. Ushinskogo, Odessa (UA)), e-mail: bargan_90@mail.ru

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4 refs., 1 fig., 1 tab.

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Borovik, A.V. (Inst. Problem Prochnosti im. G.S. Pisarenko NAN Ukrainy, Kiev (UA)), e-mail: borovyk.oleksandra@gmail.com

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Breki, A.D. (Sankt-Peterburgskij Gosudarstvennyj Politehnicheskij Univ., Sankt-Peterburg (RU)), e-mail: albreki@yandex.ru

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Goloven'ko, Zh.V. (Khakasskij Gosudarstvennyj Univ. im. N.F. Katanova, Abakan (RU)), e-mail: gg@khsu.ru

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Dragomeretskaya, E.A. (Yuzhno-Ukrainskij Natsional'nyj Pedagogicheskij Univ. im. K.D. Ushinskogo, Odessa (UA)), e-mail: drag_8181@mail.ru

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The system of a unified authorization for the information resources on the properties of inorganic substances in Institute of Metallurgy and Material Science RAS

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Dudarev, V.A. (IMET RAN, Moscow (RU)), e-mail: vic@imet.ac.ru

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Zamotin, K.Yu. (FGBOU VPO SPbGPU, Sankt-Peterburg (RU)), e-mail: kirart@gmail.com

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Zelepugin, A.S. (Tomskij Nauchnyj Tsentr SO RAN, Tomsk (RU); Natsional'nyj Issledovatel'skij Tomskij Gosudarstvennyj Univ., Tomsk (RU)), e-mail: Zel84@yandex.ru

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Zuev, S.M. (Univ. Mashinostroeniya, Moscow (RU)), e-mail: SkvortsovAA2009@yandex.ru

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TEMPERATURE RANGE 0400-1000 K

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Il'kiv, B.I. (Inst. Problem Materialovedeniya im. I.N. Frantsevicha NAN Ukrainy, Kiev (UA)), e-mail: b_ilkiv@ukr.net

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Kaverinskij, V.V. (Inst. Problem Materialovedeniya NAN Ukrainy, Kiev (UA)), e-mail: hisie@ukr.net

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Kartamyshev, A.I. (Belgorodskij Gosudarstvennyj Natsional'nyj Issledovatel'skij Univ., Nauchno-Obrazovatel'nyj i Innovatsionnyj Tsentri Nanostrukturnye Materialy i Nanotekhnologii, Belgorod (RU)), e-mail: firewitcher@mail.ru

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Kolotova, L.N. (Ob''edinennyj Inst. Vysokikh Temperatur RAN, Moscow (RU)), e-mail: lada.kolotova@gmail.com

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Marchenkov, A.Yu. (NIU MEhI, Moscow (RU)), e-mail: art-marchenkov@yandex.ru

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Sbornik materialov
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The verification of the Einstein relation in the percolation model

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Najdanova, V.A. (Khakasskij Gosudarstvennyj Univ. im. N.F. Katanova, Abakan (RU)), e-mail: n-varvara91@yandex.ru

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DIFFUSION; COMPUTERIZED SIMULATION; TRANSPORT THEORY; KINETIC EQUATIONS; STATISTICAL MECHANICS; PARTICLE MOBILITY

Fine-tuning technologies of acoustic emission method application for diagnosing technical products of alloy AMg6

S36

Pankin, D.A. (Voennaya Akademiya RVSN im. Petra Velikogo, Moscow (RU)), e-mail: pandian@rambler.ru

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S97

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Investigation into transition zone structure of the spall crack under dynamic strain localization

S36

Petrov, E.V. (FGBUN Inst. Strukturnoj Makrokinetiki i Problem Materialovedeniya RAN, Chernogolovka (RU)), e-mail: petrowewgeni@mail.ru

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Two-temperature atomistic model of heavy ion tracks formation in UO₂

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Pisarev, V.V. (Ob''edinennyj Inst. Vysokikh Temperatur RAN, Moscow (RU)), e-mail:

pisarevvv@gmail.com

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Research on selection of titanium hydrides in the titanium matrix within the model of dilute solid solutions associates

S36

Poletaev, D.O. (Belgorodskij Gosudarstvennyj Natsional'nyj Issledovatel'skij Univ., Nauchno-Obrazovatel'nyj i Innovatsionnyj Tsentr Nanostrukturnye Materialy i Nanotekhnologii, Belgorod (RU)), e-mail: darken_rhal@mail.ru

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Pushkarev, S.S. (Inst. SVCh Poluprovodnikovoj Ehlektroniki RAN, Moscow (RU); Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU)), e-mail: s_s_e_r_p@mail.ru

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Rakhmanova, M.I. (FGBUN Inst. Neorganicheskoy Khimii im. A.V. Nikolaeva SO RAN, Novosibirsk (RU)), e-mail: rakhmanova_m@mail.ru

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Presence forms analysis of oxygen in titanium carbide nanopowders by fractional gas analysis

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Rumyantsev, B.A. (IMET RAN, Moscow (RU)), e-mail: rumin@mail.ru

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NANOSTRUCTURES; POWDERS; OXYGEN; TITANIUM CARBIDES; GAS ANALYSIS;
QUANTITATIVE CHEMICAL ANALYSIS; OXYCARBIDES; TEMPERATURE
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Savel'ev, V.N. (Belgorodskij Gosudarstvennyj Natsional'nyj Issledovatel'skij Univ., Nauchno-Obrazovatel'nyj i Innovatsionnyj Tsentri Nanostrukturnye Materialy i Nanotekhnologii, Belgorod (RU)), e-mail: firesnake21@mail.ru

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San'kova, S.V. (Yuzhno-Ural'skij Natsional'nyj Pedagogicheskij Univ. im. K.D. Ushinskogo, Odessa (UA)), e-mail: sankova_1986@mail.ru

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Severova, V.M. (IMET RAN, Moscow (RU)), e-mail: lerka91@gmail.com

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BASE ALLOYS; EMISSION SPECTROSCOPY; WAVELENGTHS; DISSOLUTION

Discrete breathers in #alpha#-Ti

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Semenov, A.S. (Politekhnikeskij Inst. (Filial) FGAOU VPO Severo-Vostochnyj Federal'nyj Univ. imeni M.K. Ammosova v g. Mirnom, (RU)), e-mail: sash-alex@yandex.ru

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DISCRETE ORDINATE METHOD; PHONONS; EXCITATION

The use of interatomic potentials for self-diffusion study in #gamma#-U and in alloys U-Mo

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URANIUM-MOLYBDENUM FUELS; CRYSTALS; URANIUM; SELF-DIFFUSION; MOLECULAR DYNAMICS METHOD; BCC LATTICES; CRYSTAL DEFECTS

Damaged material nondestructive testing methods in the conditions of mixed types of load

S36

Sodatenkov, A.P. (IMET RAN, Moscow (RU)), e-mail: alexxx.sodatenkov@yandex.ru

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619 p.
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STRESSES; NONDESTRUCTIVE TESTING

Construction and analysis of microdamages percolation cluster in loaded metallic materials

S36

Stepanova, V.Eh. (Tul'skij Gosudarstvennyj Univ., Tula (RU)), e-mail: valentina_step@mail.ru

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4 refs., 2 figs.

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Development of nitrogen express fractional analysis in steels

S36

Tabakov, Ya.I. (IMET RAN, Moscow (RU)), e-mail: ytabakov@gmail.com

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Investigation into the spinodal decomposition kinetics in amorphous alloys

S36

Terekhova, Yu.V. (Donetskij Fiziko-Tekhnicheskij Inst. im. A.A. Galkina, Donetsk (UA)), e-mail: tmv.dn@yandex.ru

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Investigation of phase transitions in uranium nitride by atomistic simulations
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S36

Cherednichenko, A.V. (Kuzbasskij Gosudarstvennyj Tekhnicheskij Univ., Kemerovo (RU)), e-mail: allacherednichenk@rambler.ru

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Cherneva, E.S. (Yuzhno-Ukrainskij Natsional'nyj Pedagogicheskij Univ. im. K.D. Ushinskogo, Odessa (UA)), e-mail: cherneva_1986@mail.ru

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Calculating the parameters of martensitic transformation by atomistic simulations methods

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Chirkov, P.V. (Yuzhno-Ural'skij Gosudarstvennyj Univ., Chelyabinsk (RU)), e-mail: p.chirkov@physics.susu.ac.ru

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Shabunina, E.V. (KhGU im. N.F. Katanova, Abakan (RU)), e-mail: galichinaev@mail.ru

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NANOSTRUCTURES; COMPUTERIZED SIMULATION; FERROMAGNETIC MATERIALS; ONE-DIMENSIONAL CALCULATIONS; PHASE DIAGRAMS; ISING MODEL

Study of optical properties of gallium-silicate fiber lightguides with a core containing β -Ga₂O₃:Cr³⁺

S36

Abramov, A.N. (IKhVV RAN, Nizhnij Novgorod (UA)), e-mail: abramovan84@mail.ru

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SPECTRA

Effect of silicon lattice orientation on the character of etching the monocrystal during the formation of nanoporous silicon layers

S36

Abramova, E.N. (Moskovskij Gosudarstvennyj Univ. Tonkikh Khimicheskikh Tekhnologij im. M.V. Lomonosova, Moscow (RU)), e-mail: overmind11@rambler.ru

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Preparation and research of functional nanomaterials on the basis of barium titanate for purposes of electrical calorimetry

S77

Abrashova, E.V. (FGBOU Sankt-Peterburgskij Gosudarstvennyj Ehlektrotekhnicheskij Univ. im. V.I. Ul'yanova (Lenina) LEhTI, Sankt-Peterburg (RU)), e-mail: katabr@mail.ru

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Influence of ways of obtaining on the dissipative characteristics of solid solutions of sodium - lithium niobates

S37

Abubakarov, A.G. (Yuzhnyj Federal'nyj Univ., Nauchno-Issledovatel'skij Inst., Rostov-na-Donu (RU)), e-mail: abubakarov12@mail.ru

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SOLID SOLUTIONS; NIOBATES; SODIUM COMPOUNDS; LITHIUM COMPOUNDS;
CHEMICAL COMPOSITION; CURIE POINT; ABSORPTION SPECTRA

Thermo-EMF of graphite and mixtures of graphite with zirconium oxide at pressures from 6 to 44 GPa

S36

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(RU)

2 refs., 2 figs.

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619 p.

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Research of influence of gel formers and the temperature synthesis of powders on the properties of ceramics based on lanthanum chromite

S36

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na osnove khromita lantana

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LANTHANUM COMPOUNDS; CHROMITES; CERAMICS; TEMPERATURE RANGE

1000-4000 K; HOT PRESSING; PVA; THERMAL ANALYSIS; PVP; GELATION;

SINTERING

Dielectric characteristics of pure and Li-modified ceramics

PbFe_{0.5}Nb_{0.5}O₃ in the temperature range (20-300) K

S36

Boldyrev, N.A. (Yuzhnyj Federal'nyj Univ., Nauchno-Issledovatel'skij Inst. Fiziki, Rostov-na-Donu (RU)), e-mail: huckwrench@gmail.com

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MODIFICATIONS

Production of composite materials such as carbon graphite-aluminum alloy

S36

Golovinov, P.S. (Volgogradskij Gosudarstvennyj Tekhnicheskij Univ., Volgograd (RU)), e-mail: pashabanshik@mail.ru

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The study of hydrogen permeability of the membrane from a palladium alloy with lead before and after work in gas media

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Gorbunov, S.V. (IMET RAN, Moscow (RU)), e-mail: Merciles@mail.ru

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Armour ceramic material based on silicon carbide

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Zhitnyuk, S.V. (FGBOU VPO RKhTU im. D.I. Mendeleeva, Moscow (RU)), e-mail: feanor42@ya.ru

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MODULUS; VICKERS HARDNESS

New catalysts based on cerium dioxide for use in reactions involving oxygen
S77

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The membranes based on nanoporous aluminum oxide for chemical gas sensors
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Zakhlebaeva, A.I. (Belorusskij Gosudarstvennyj Univ. Informatiki i Radioelektroniki, Minsk (BY)), e-mail: zahlebaeva@gmail.com

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Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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Study of the microstructure of rutile prepared using oxidative design approach S36

Kovalev, I.A. (IMET RAN, Moscow (RU)), e-mail: vankovalskij@mail.ru

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Composite nanoporous material and process for its preparation

S77

Kozlov, A.Yu. (Tol'yattinskij Gosudarstvennyj Univ., Tol'yatti (RU)), e-mail:

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Studying the properties of Ni/Fe nanotubes

S77

Kozlovskij, A.L. (ENU im. L.N. Gumileva, Astana (KZ)), e-mail: artem88sddt@mail.ru

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NANOMATERIALS; MEMBRANES; LATTICE PARAMETERS

Issues of creation gradient-porous substrates for hydrogen separating layers based on palladium

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Kolchina, A.S. (Federal'noe Gosudarstvennoe Byudzhethoe Uchrezhdenie Nauki Inst. Problem Tekhnologii Mikroelektroniki i Osobochistykh Materialov RAN, Chernogolovka (RU)), e-mail: a.kolchina@mail.ru

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Study of interphase boundaries in the heterostructure GaN/Al₂O₃/Si
S36

Kondrat'eva, O.N. (Federal'noe Gosudarstvennoe Uchrezhdenie Nauki Inst. Obshchej i Neorganicheskoj Khimii im. N.S. Kurnakova RAN, Moscow (RU)), e-mail: ol.kondratieva@gmail.com

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(RU)

4 refs., 1 fig., 1 tab.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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GALLIUM NITRIDES; ALUMINIUM OXIDES; SILICON; SEMICONDUCTOR MATERIALS; INTERFACES; FREE ENTHALPY; TEMPERATURE RANGE 0273-0400 K; TEMPERATURE RANGE 0400-1000 K; TEMPERATURE RANGE 1000-4000 K; TEMPERATURE DEPENDENCE; CHEMICAL REACTIONS

Electrical conductivity of glasses 8(Na₂O+Fe₂O₃) - 22B₂O₃ - 70SiO₂
S36

Konon, M.Yu. (Inst. Khimii Silikatov im. I.V. Grebenshchikova RAN (IKhS RAN), Sankt-Peterburg (RU)), e-mail: marina-konon@mail.ru

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7 refs., 2 figs., 1 tab.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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The use of point structures to create high-performance thermoelectric generators based on samarium sulfide

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Kostyaeva, A.V. (NITs Kosmicheskoe Materialovedenie FIK RAN, Kaluga (RU)), e-mail: kostikova89@mail.ru

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(RU)

4 refs., 1 fig.

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SAMARIUM SULFIDES; ELECTROMOTIVE FORCE; TEMPERATURE DEPENDENCE;
TEMPERATURE RANGE 0065-0273 K; TEMPERATURE RANGE 0273-0400 K;

Oxygen permeability of the composite membrane materials NiO - Ag - melts based on Bi₂O₃ and In₂O₃ - Ag - melts based on Bi₂O₃

S36

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(RU)

2 refs.

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CERMETS; NICKEL OXIDES; SILVER; BISMUTH OXIDES; INDIUM OXIDES; IONIC CONDUCTIVITY; PERMITTIVITY; CHEMICAL COMPOSITION; MEMBRANES; MICROSTRUCTURE

The binding energies between the atoms in the metal-graphene interface

S36

Kurbanova, Eh.D. (Inst. Metallurgii UrO RAN, Ekaterinburg (RU)), e-mail: kurbellya@mail.ru

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GRAPHENE; BINDING ENERGY; NICKEL; PALLADIUM; COPPER; INTERFACES; DENSITY FUNCTIONAL METHOD; POTENTIAL ENERGY; TEMPERATURE RANGE 1000-4000 K; TEMPERATURE DEPENDENCE

Synthesis and electrochemical properties of cathode materials based on $\text{Li}_{1-x}\text{Mn}_x\text{Ni}_x\text{O}_2$ for lithium-ion batteries

S36

Kurilenko, K.A. (MGU im. M.V. Lomonosova, Moscow (RU)), e-mail: kostik_msu@mail.ru

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Sintez i ehlektrokhimicheskie svoystva katodnykh materialov na osnove

$\text{Li}_{1-x}\text{Mn}_x\text{Ni}_x\text{O}_2$ dlya litij-ionnykh akkumulyatorov

p. 225

(RU)

2 refs.

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CLATHRATES; LITHIUM OXIDES; MANGANESE OXIDES; NICKEL OXIDES;
MICROSTRUCTURE; MORPHOLOGY; CHEMICAL COMPOSITION;
NANOSTRUCTURES; ANNEALING; TEMPERATURE RANGE 0400-1000 K

**Studying the properties of ytterbium-doped fibers on the basis of
multicomponent matrices Al₂O₃-P₂O₅-SiO₂ and B₂O₃-Al₂O₃-
P₂O₅-SiO₂**

S36

Lipatov, D.S. (IKhVV RAN, Nizhnij Novgorod (RU)), e-mail: lipatovds@mail.ru

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(RU)

4 figs.

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ALUMINIUM OXIDES; PHOSPHORUS OXIDES; SILICON OXIDES; BORON OXIDES; YTTERBIUM OXIDES; OPTICAL FIBERS; LUMINESCENCE; REFLECTION

Development of ceramic membranes modified by carbon nanotubes

S36

Luchikhina, V.S. (Inst. Kataliza SO RAN, Novosibirsk (RU)), e-mail: Torry133@gmail.com

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CERAMICS; MEMBRANES; MODIFICATIONS; CARBON NANOTUBES; PALLADIUM NITRATES; SYNTHESIS; SILICON OXIDES; SODIUM OXIDES; ALUMINIUM OXIDES; CALCIUM OXIDES; PERMITTIVITY

The chemical stability of $BaCe_{0.8-x}Zr_xY_{0.2}O_{3-\delta}$ based material

S36

Lyagaeva, Yu.G. (Inst. Vysokotemperaturnoj Ehlektrokhimii UrO RAN, Ekaterinburg (RU)), e-mail: yulia.lyagaeva@ya.ru

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Khimicheskaya stabil'nost' materialov na osnove $BaCe_{0.8-x}Zr_xY_{0.2}O_{3-\delta}$
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2 refs., 4 figs.

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Advanced ceramic materials based on high-temperature multiferroics

S36

Markov, A.V. (Nauchno-Issledovatel'skij Inst. Fiziki Yuzhnogo Federal'nogo Univ., Rostov-na-Donu (RU)), e-mail: ihatethuh@gmail.com

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8 refs., 1 fig.

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619 p.

CERAMICS; FERROELECTRIC MATERIALS; SOLID SOLUTIONS; BISMUTH OXIDES; LEAD OXIDES; NIOBIUM OXIDES; IRON OXIDES; HYSTERESIS; ABUNDANCE

Microstructure, dielectric and magnetodielectric characteristics of ceramics

Bi_{0.6}La_{0.4}MnO₃

S36

Makhaboroda, A.V. (Nauchno-Issledovatel'skij Inst. Fiziki Yuzhnogo Federal'nogo Univ., Rostov-na-Donu (RU)), e-mail: kanemaster@maul.ru

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Bi_{0.6}La_{0.4}MnO₃

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1 ref., 2 figs.

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Materials for spintronics: electronic and transport properties of ZGNR/h-BN(0001) heterostructures

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Nguyen, V.Ch. (Donskoj Gosudarstvennyj Tekhnicheskij Univ., Rostov-na-Donu (RU)), e-mail: chuongnguyen11@gmail.com

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BORON NITRIDES; DENSITY FUNCTIONAL METHOD; GRAPHENE;
NANOSTRUCTURES; ELECTRONIC STRUCTURE; VAN DER WAALS FORCES;
HEXAGONAL LATTICES

Matrix of dispersed nuclear fuel based on Al-Be-alloys

S36

Nikitin, S.N. (Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU)), e-mail: mephi200809@yandex.ru

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(RU)

8 refs.

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ALUMINIUM ALLOYS; BERYLLIUM ALLOYS; PROTECTIVE COATINGS; DISPERSION NUCLEAR FUELS; MATRIX MATERIALS; THERMAL CONDUCTIVITY

Research in wear resistance of powder composite electrode-tools when electroerosion firmware

S36

Ogleznev, N.D. (Permskij Natsional'nyj Issledovatel'skij Politekhnikeskij Univ., Perm' (RU)), e-mail: fastrex@mail.ru

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ehlektroehrozionnoj proshivke

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WEAR RESISTANCE; MECHANICAL TESTS; ELECTRODES; COPPER BASE ALLOYS; ELECTRIC CONDUCTIVITY; COMPOSITE MATERIALS; ALUMINIUM TUNGSTATES; CHROMIUM ALLOYS; TITANIUM CARBIDES; POROSITY

The efficiency of absorption of laser radiation by silver nanoparticles in a matrix of PETN

S77

Odintsova, O.V. (FGBOU VPO Kemerovskij Gosudarstvennyj Univ., Kemerovo (RU)), e-mail: kriger@kemsu.ru

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Ehffektivnost' pogloshcheniya lazernogo izlucheniya nanochastitsami serebra v matritse tehna p. 238-239

(RU)

1 ref., 2 figs.

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NANOSTRUCTURES; LASER RADIATION; ALUMINIUM; COMPOSITE MATERIALS;
SILVER; PETN; PARTICLE SIZE; WAVELENGTHS

Dielectric spectroscopy of solid solutions based on ferroelectromagnets

BeFeO₃/REE

S37

Pavelko, A.A. (NII Fiziki Yuzhnogo Federal'nogo Univ., Rostov-na-Donu (RU)), e-mail: aapavelko@sfnedu.ru

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BiFeO₃/RZeh

p. 239-241

(RU)

5 refs., 2 figs.

Rare earth: Tb, Dy, Ho, Er, Tm, Yb, Lu

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SOLID SOLUTIONS; FERRITES; BISMUTH COMPOUNDS; CHEMICAL COMPOSITION;
CERAMICS; TEMPERATURE RANGE 0400-1000 K; TEMPERATURE DEPENDENCE;
FERROELECTRIC MATERIALS

Dielectric spectroscopy of ceramics Ba_{0.5}Sr_{0.5}Nb₂O₆ in the temperature range of 20-300 #Degree Sign# C

S37

Pavlenko, A.V. (NII Fiziki Yuzhnogo Federal'nogo Univ., Rostov-na-Donu (RU)), e-mail: tolik_260686@mail.ru

XI Russian annual conference of young researchers and graduate students Physico-chemistry and technology of inorganic materials (with international participation)

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XI Rossijskaya ezhegodnaya konferentsiya molodykh nauchnykh sotrudnikov i aspirantov Fiziko-khimiya i tekhnologiya neorganicheskikh materialov (s mezhdunarodnym uchastiem) Dielektricheskaya spektroskopiya keramiki Ba_{0.5}Sr_{0.5}Nb₂O₆ v oblasti temperatur 20-300 #Degree Sign# C

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(RU)

2 figs.

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CERAMICS; SOLID SOLUTIONS; NIOBATES; BARIUM COMPOUNDS; STRONTIUM COMPOUNDS; CHEMICAL COMPOSITION; PERMITTIVITY; TEMPERATURE RANGE 0273-0400 K; TEMPERATURE RANGE 0400-1000 K; TEMPERATURE DEPENDENCE; FERROELECTRIC MATERIALS

Effect of the structural state of chromium on the oxidation resistance of the iron aluminide based composite powder and the Al₂O₃ inclusions reinforced S36

Parnitskij, N.M. (Inst. Poroshkovej Metallurgii, Minsk (BY)), e-mail: skeyone@rambler.ru

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Vliyanie strukturnogo sostoyaniya khroma na stojkost' k okisleniyu kompozitsionnogo poroshka na osnove alyuminida zheleza, uprochnennogo vklyucheniyami Al₂O₃

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2 figs.

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POWDERS; MICROHARDNESS; CHROMIUM OXIDES; OXIDATION;
METALLURGICAL EFFECTS

Structural ceramics based on B₄C obtained by liquid phase sintering method **S36**

Perevislov, S.N. (OAO TsNIIM, Sankt-Peterburg (RU)), e-mail: perevislov@mail.ru

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(RU)

1 ref., 1 fig., 2 tabs.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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DENSITY; YOUNG MODULUS; MICROSTRUCTURE

Studying the electronic and spatial structure of magnetic materials based on chromium-copper disulfide

S36

Peregudova, N.N. (FBGUN Inst. Neorganicheskoy Khimii im. A.V. Nikolaeva SO RAN, Novosibirsk (RU)), e-mail: pe-nat@mail.ru

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1 ref.

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The elastic properties of single-walled carbon boron nitride and silicon carbide nanotubes: ionization effect

S36

Petrushenko, I.K. (Irkutskij Gosudarstvennyj Tekhnicheskij Univ., Irkutsk (RU)), e-mail: igor.petrushenko@istu.edu

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(RU)

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Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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NANOSTRUCTURES; ATOMIC CLUSTERS; NICKEL; ORDER-DISORDER MODEL; COMPUTERIZED SIMULATION; NANOPARTICLES; CRYSTALLIZATION; FCC LATTICES

Metal-containing materials based on chitosan: synthesis, structure and

properties

S75

Rubina, M.S. (*Inst. Ehlementoorganicheskikh Soedinenij im. A.N. Nesmeyanova RAN, Moscow (RU)*), e-mail: margorubina@yandex.ru

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MOESSBAUER EFFECT; SPECTRA; NANOPARTICLES; NANOCOMPOSITES; COLLAGEN; IRON; GOLD; SILVER; POLYMERS

Physical and mechanical properties of highly porous ceramics based on diatomite

S36

Skvortsov, P.A. (*Univ. Mashinostroeniya, Moscow (RU)*), e-mail: SkvortsovAA2009@yandex.ru

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Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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K

Low-temperature ceramics based on zirconium dioxide

S36

Smirnov, S.V. (IMET RAN, Moscow (RU)), e-mail: serega_smirnov92@mail.ru

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Fiziko-khimiya i tekhnologiya neorganicheskikh materialov (s mezhdunarodnym uchastiem)
Nizkotemperaturnaya keramika na osnove dioksida tsirkoniya

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(RU)

2 refs., 1 fig.

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FRACTURE PROPERTIES

Effect of gas molecules chemisorption on the dielectric properties of the highly dispersed material

S37

Sokolov, A.A. (FGOU VPO Gosuniversitet - UNPK, Orel (RU)), e-mail: ispleener@gmail.com

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(RU)

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Study of surface properties of sorbents based on zirconium and aluminum oxyhydrates

S37

Sorokina, E.A. (Gosudarstvennoe Vysshee Uchebnoe Zavedenie Gosudarstvennyj Khimiko-Tekhnologicheskij Univ., Dnepropetrovsk (UA)), e-mail: kate_3110@mail.ru

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2 figs.

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SURFACE PROPERTIES; HYDRATES; ZIRCONIUM OXIDES; ALUMINIUM OXIDES;
ADSORBENTS; NANOSTRUCTURES; POROSITY; SOL-GEL PROCESS

Suppression of ferroelectric state, caused by the barium modification of ceramics based on ferroelectrics-relaxor

S36

Talanov, M.V. (NII Fiziki Yuzhnogo Federal'nogo Univ., Rostov-na-Donu (RU)), e-mail: mvtalanov@sfnu.ru

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Podavlenie segnetoehlektricheskogo sostoyaniya, vyzvanoe modifitsirovaniem bariem keramik na osnove segnetoehlektrikov-relaksorov

p. 262-263

(RU)

3 refs., 1 fig.

Ceramics: $Pb_{(1-z)}$

$Ba_z(Mg_{1/3}Nb_{2/3})_m(Zn_{1/3}Nb_{2/3})_y(Ni_{1/3}Nb_{2/3})_nTi_xO_3$ ($x=0$
#En Dash# 0.15, $m=0.4541$, $y=0.0982$, $n=0.1477$, $z=0.3$)

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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Synthesis and study of the characteristics of expanded graphite

S36

Timofeeva, A.A. (Novosibirskij Gosudarstvennyj Tekhnicheskij Univ., Novosibirsk (RU)), e-mail: timofeeva_anastasia@mail.ru

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(RU)

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The mechanism of transfer of oxygen through the composite ion transport membranes ZnO-Bi₂O₃ with a liquid channel grain boundary structure S36

Fedorov, S.V. (IMET RAN, Moscow (RU)), e-mail: fedserv@rambler.ru

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COMPOSITE MATERIALS; MEMBRANES; ION EXCHANGE MATERIALS; ZINC OXIDES; BISMUTH OXIDES; GRAIN BOUNDARIES; IONIC CONDUCTIVITY; PERMEABILITY

Thermal properties of solid solutions BiFeO_3/Tb , Dy, Ho, Er, Tm, Yb, Lu S36

Khasbulatov, S.V. (NII Fiziki Yuzhnogo Federal'nogo Univ., Rostov-na-Donu (RU)), e-mail: said_vahaevich@mail.ru

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7 refs., 2 figs.

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SOLID SOLUTIONS; FERRITES; BISMUTH COMPOUNDS; CRYSTAL DOPING; ION IMPLANTATION; CERAMICS; RARE EARTH ALLOYS; SINTERING; TEMPERATURE DEPENDENCE; TEMPERATURE RANGE 0273-0400 K; TEMPERATURE RANGE 0400-1000 K; ELECTRIC CONDUCTIVITY

Effect of sintering temperature of ceramics based on silicon carbide under a high pressure on its density, microhardness and thermal conductivity

S36

Kholdeev, K.I. (GO NPTs NANB po Materialovedeniyu, Minsk (BY)), e-mail: HKI1990@tut.by

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3 refs., 2 figs.

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SILICON CARBIDES; SINTERING; CERAMICS; DENSITY; MICROHARDNESS; THERMAL CONDUCTIVITY; HOT PRESSING; PRESSURE RANGE GIGA PA; TEMPERATURE RANGE 1000-4000 K; TEMPERATURE DEPENDENCE; MORPHOLOGY

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Ce_{0.09}Zr_{0.91}O₂/MgO-Al₂O₃

S36

Khrushcheva, A.A. (IMET RAN, Moscow (RU)), e-mail: anastasiya.xr@gmail.com

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Effect of the structural state on cold-storage capacity of gadolinium

S37

Chzhan, V.B. (IMET RAN, Moscow (RU)), e-mail: lemuriform@gmail.com

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Study of Pd-In-Ru membrane performance in hydrocarbon gas mixtures

S37

Chistova, T.V. (IMET RAN, Moscow (RU)), e-mail: tatyana_chistova@list.ru

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Iron-based oxide composites obtained by using the approach of oxidative constructing

S36

Shashkeev, K.A. (IMET RAN, Moscow (RU)), e-mail: shashkon@yandex.ru

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3 refs., 2 figs.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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IRON; IRON OXIDES; OXIDATION; MORPHOLOGY; CERAMICS; COMPOSITE MATERIALS; CRYSTALLOGRAPHY; TEMPERATURE RANGE 1000-4000 K

The microstructure of titanium dioxide produced at a temperature of 1300#Degree Sign# C

S36

Shevtsov, S.V. (IMET RAN, Moscow (RU)), e-mail: shevtsov_sv@mail.ru

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1 ref., 3 figs.

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TITANIUM OXIDES; OXIDATION; TEMPERATURE RANGE 1000-4000 K; TETRAGONAL LATTICES; CERAMICS; MICROSTRUCTURE; NANOSTRUCTURES

Preparation and model nitride nuclear fuel study

S37

Shornikov, D.P. (Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU))

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S36

Shuba, A.V. (Voronezhskij Gosudarstvennyj Tekhnicheskij Univ., Voronezh (RU)), e-mail: shandvit@rambler.ru

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LANDAU FLUCTUATIONS; CURIE POINT; PHASE TRANSFORMATIONS; FREE
ENTHALPY; NANOSTRUCTURES

Investigation into the process of titanium diboride synthesis using nanofibrous carbon

S36

Antonova, E.V. (Novosibirskij Gosudarstvennyj Tekhnicheskij Univ., Novosibirsk (RU)), e-mail: antonova@corp.nstu.ru

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6 refs., 2 figs.

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TITANIUM BORIDES; CARBON FIBERS; CERMETS; CHEMICAL PREPARATION;
TEMPERATURE RANGE 1000-4000 K; TITANIUM OXIDES; BORON CARBIDES

Distribution properties of the deposited nanopowder layer at height of reactor when plasma-chemical synthesis

S77

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NANOSTRUCTURES; SYNTHESIS; PLASMA JETS; POWDERS; PLASMATRONS;
ELECTRIC ARCS; DENSITY

SHS-extrusion materials based on Ti-Al-C

S36

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TITANIUM CARBIDES; ALUMINIUM CARBIDES; MICROSTRUCTURE; EXTRUSION;
SYNTHESIS; TEMPERATURE RANGE 1000-4000 K; CHEMICAL COMPOSITION;
TITANIUM BASE ALLOYS; ALUMINIUM ALLOYS; CERAMICS

Synthesis and crystal structure of nanostructured powders of system Ni-Cd **S77**

*Val'nyukova, A.S. (FGBOU VPO Kemerovskij Gosudarstvennyj Univ., Kemerovo (RU)), e-mail:
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NANOSTRUCTURES; INTERMETALLIC COMPOUNDS; NICKEL BASE ALLOYS;
CADMIUM ALLOYS; SYNTHESIS; SOLID SOLUTIONS; FCC LATTICES; POWDERS

The study of ceramics manufacturing processes based on the bonded aluminium- yttrium composition

S36

*Varfolomeev, M.S. (MATI-RGTU im. K.Eh. Tsiolkovskogo, Moscow (RU)), e-mail:
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(RU)
2 refs.

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Gold nanoparticles synthesis using the contact nonequilibrium low-temperature plasma

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Vorob'eva, M.I. (Ukrainskij Gosudarstvennyj Khimiko-Tekhnologicheskij Univ., Dnepropetrovsk (UA)), e-mail: Rita-Vorobyova@yandex.ru

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S77

Garbuzova, A.K. (FGBOU VPO Sibirskij Gosudarstvennyj Industrial'nyj Univ., Novokuznetsk (RU)), e-mail: kafcmet@sibsiu.ru

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K; TEMPERATURE RANGE OVER 4000 K

Nanostructured powders formation by ultrasonic spray pyrolysis for ceramics

S77

Gasimi, M.S. (IMET RAN, Moscow (RU)), e-mail: msgasimi@gmail.com

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Complex analysis of iron oxide nanopowders synthesized by chemical methods S77

Grebennikov, I.S. (NITU MISiS, Moscow (RU)), e-mail: da_prorabinho@mail.ru

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Datij, K.A. (Inst. Uglekhemii i Khimicheskogo Materialovedeniya SO RAN, Kemerovo (RU)), e-mail: zaharov@kemsu.ru

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Preparation of composites based on ultrafine molybdenum and tungsten oxides and nanocarbon

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Kotsareva, K.V. (IMET RAN, Moscow (RU)), e-mail: kotsareva.k@gmail.com

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Kryzhanov, M.V. (Federal'noe Gosudarstvennoe Byudzhethoe Uchrezhdenie Nauki Inst. Khimii i Tekhnologii Redkikh Ehlementov i Mineral'nogo Syr'ya im. I.V. Tananaeva Kol'skogo Nauchnogo Tsentra RAN - IKhTREhMS KNTs RAN, Apatity (RU)), e-mail:

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Kuznetsova, V.V. (Novosibirskij Gosudarstvennyj Tekhnicheskij Univ., Novosibirsk (RU)), e-mail: Kuznecova_VV@inbox.ru

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5 refs., 2 figs.

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Development of copper-based alloys for impregnation of carbon graphite materials

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Leshchinskaya, K.V. (IMET RAN, Moscow (RU)), e-mail: kirochka87@list.ru

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3 refs.

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S77

Litvin, V.A. (Cherkasskij Natsional'nyj Univ. imeni Bogdana Khmel'nitskogo, Cherkassy (UA)), e-mail: litvin_valentina@ukr.net

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1 ref., 1 fig.

Metals: Fe, Co, Ni

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SYNTHESIS; NANOCOMPOSITES; FULVIC ACIDS; PYROLYSIS; NANOPARTICLES;
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Lysenkov, A.S. (IMET RAN, Moscow (RU)), e-mail: toxa55@bk.ru

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Influence of the type of nickel containing catalyst on carbon nanotubes synthesis by catalytic pyrolysis of ethanol

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Mal'tsev, I.A. (Permskij Natsional'nyj Issledovatel'skij Politekhnikeskij Univ., Perm' (RU)), e-mail: keramik@pm.pstu.ac.ru

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CARBON NANOTUBES; NICKEL OXIDES; NICKEL HYDROXIDES; NICKEL NITRATES; NICKEL; CATALYSTS; SYNTHESIS; PYROLYSIS; ETHANOL

Spark plasma sintering of the composite material Cu60-Ta40

S36

Malyutina, Yu.N. (Novosibirskij Gosudarstvennyj Tekhnicheskij Univ., Novosibirsk (RU)), e-mail: iuliamaliutina@gmail.com

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COPPER BASE ALLOYS; TANTALUM ALLOYS; SINTERING; ELECTRIC CURRENTS;
ELECTRIC SPARKS; COMPOSITE MATERIALS; MICROSTRUCTURE;
NANOSTRUCTURES

**Cast composite materials in the system: Al₂O₃-Cr₂O₃ x ZrO₂ obtained
by SHS-metallurgy**

S36

Miloserdov, P.A. (Federal'noe Gosudarstvennoe Byudzhethnoe Uchrezhdenie Nauki Inst. Strukturnoj Makrokinetiki i Problem Materialovedeniya RAN, Chernogolovka (RU)), e-mail: yu_group@ism.ac.ru

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Effect of titanium additions on the rheological properties of feedstock billet and formability of material based on molybdenum disilicide

S36

Mikheev, M.V. (Federal'noe Gosudarstvennoe Byudzhethnoe Uchrezhdenie Nauki Inst. Strukturnoj Makrokinetiki i Problem Materialovedeniya RAN, Chernogolovka (RU)), e-mail: mixeev777@rambler.ru

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Preparation of powder alloys based on RuAl from the processed powder mixtures Ru+Al in an attritor

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Morozov, A.E. (IMET RAN, Moscow (RU)), e-mail: ae_morozov@list.ru

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SINTERING; GRANULATION; MIXING; TEMPERATURE RANGE 0400-1000 K

The study of diffusion processes in normal and plasma spark sintering of systems with dispersed powders

S77

Portalov, M.N. (Permskij Natsional'nyj Issledovatel'skij Politehnicheskij Univ., Perm' (RU)), e-mail: maksimpo1989@mail.ru

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Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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SINTERING; ELECTRIC SPARKS; ELECTRIC CURRENTS; MICROHARDNESS;
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Shock-wave synthesis of titanium carbide and diboride in cylindrical saving ampoules

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Sajkov, I.V. (Uchrezhdenie Rossijskoj Akademii Nauk Inst. Strukturnoj Makrokinetiki i Problem Materialovedeniya RAN, Chernogolovka (RU)), e-mail: revan.84@mail.ru

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TITANIUM CARBIDES; TITANIUM BORIDES; SHOCK WAVES; SYNTHESIS; PRESSING; EXPLOSION WELDING; POWDERS

Extend characteristics of the dispersed composition of nanopowders W, Cu, Al₂O₃ and TiO₂ formed in a plasma reactor according to the mechanism of vapor-liquid-crystal

S77

Sinajskij, M.A. (IMET RAN, Moscow (RU)), e-mail: sinaisky@imet.ac.ru

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Development of carbon structures production by thermolysis of plastic wastes

S36

Sychenko, D.V. (Novosibirskij Gosudarstvennyj Tekhnicheskij Univ., Novosibirsk (RU)), e-mail: sychenko.diana@mail.ru

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619 p.

CARBON NANOTUBES; CHEMICAL WASTES; PLASTICS; TEMPERATURE RANGE 0400-1000 K; HEAT TREATMENTS; THERMAL DEGRADATION

Alloying and sintering effects on the porosity and magnetic hysteresis properties of the Fe-26Cr-16Co powder alloy

S36

Ustyukhin, A.S. (IMET RAN, Moscow (RU)), e-mail: fcbneo@yandex.ru

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IRON BASE ALLOYS; CHROMIUM ALLOYS; COBALT ALLOYS; POWDER METALLURGY; TUNGSTEN ALLOYS; MOLYBDENUM ALLOYS; SINTERING; TEMPERATURE RANGE 1000-4000 K; COERCIVE FORCE; MAGNETIC PROPERTIES

Copper nanopowder production by plasma chemical method

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NANOPARTICLES; PARTICLE SIZE; SPHERICAL CONFIGURATION; COPPER

Magnetic iron oxides preparation by reduction of various types of carbon **S77**

Fedotov, M.A. (IMET RAN, Moscow (RU)), e-mail: mikle_fed@mail.ru

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Development of gas-free static technology for composite production **S36**

Chernichkin, E.E. (Volgogradskij Gosudarstvennyj Tekhnicheskij Univ., Volgograd (RU)), e-mail: john-qwerty@mail.ru

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COMPOSITE MATERIALS; IMPREGNATION; TEMPERATURE DEPENDENCE; ALUMINIUM BASE ALLOYS; GRAPHITE; THERMAL EXPANSION; TEMPERATURE RANGE 0400-1000 K

Production of ceramic electrode nanomaterials by a SHS-extrusion and their use as electric-spark coatings

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Chizhikov, A.P. (ISMAN RAN, Chernogolovka (RU)), e-mail: chiz_an_pz@mail.ru

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Improved melting and performance properties of strontium-aluminum-silicate pyroceramic glass by mechanical activation of charge

S36

Shelaeva, T.B. (OAO Nauchno-Issledovatel'skij Inst. Tekhnicheskogo Stekla, Moscow (RU)), e-mail: shelaeva_tatyana@mail.ru

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5 refs., 1 fig.

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COMMUNITION; MELTING; TEMPERATURE RANGE 1000-4000 K;
AGGLOMERATION; MIXTURES

The study of the phase transition in the powder TiO₂ prepared by the sol-gel technology

S36

Shokov, V.O. (FGBOU VPO Permskij Natsional'nyj Issledovatel'skij Politekhnikeskij Univ., Perm' (RU)), e-mail: keramik@pm.pstu.ac.ru

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Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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TITANIUM OXIDES; POWDERS; SOL-GEL PROCESS; SYNTHESIS; CALCINATION; TEMPERATURE RANGE 1000-4000 K; PHASE STUDIES; DIFFERENTIAL THERMAL ANALYSIS

Investigation into the process of grinding of hematite (α -Fe₂O₃) nanoparticles in gelatin

S37

Baluyan, T.G. (Moskovskij Gosudarstvennyj Univ. imeni M.V. Lomonosova, Moscow (RU)), e-mail: tarinax@yandex.ru

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Issledovanie protsessa izmel'cheniya nanochastits gematita (α -Fe₂O₃) v zhelatine
p. 359-360

(RU)

6 refs.

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COBALT COMPLEXES; STEARATES; CHEMICAL COMPOSITION; LIGANDS;
NICOTINIC ACID; COORDINATION NUMBER; CHEMICAL PREPARATION;
QUANTITATIVE CHEMICAL ANALYSIS; MOLECULAR STRUCTURE

Investigation into the influence of synthesis parameters on the rheological properties of the gels ZnO

S77

Blinov, A.V. (FGAOU VPO Severo-Kavkazskij Federal'nyj Univ., Stavropol' (RU)), e-mail: blinov.a@mail.ru

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4 refs.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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ZINC OXIDES; RHEOLOGY; GELS; NANOSTRUCTURES; SYNTHESIS; VISCOSITY;
TEMPERATURE RANGE 0273-0400 K; PH VALUE; CONVERSION RATIO

Choosing the synthesis optimal parameters of silver nanoparticles

S77

Blinov, A.V. (FGAOU VPO Severo-Kavkazskij Federal'nyj Univ., Stavropol' (RU)), e-mail: blinov.a@mail.ru

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SILVER; NANOPARTICLES; SYNTHESIS; QUATERNARY AMMONIUM COMPOUNDS; SILVER NITRATES; BOROHYDRIDES; SODIUM COMPOUNDS; TEMPERATURE RANGE 0273-0400 K; OPTIMIZATION

Optimization of the synthesis conditions, the crystal structure of perovskite-like compounds $YBaCo_{2-y}Zr_zFe_yCu_zO_{6-\delta}$

S37

Bryuzgina, A.V. (FGAOU VPO Ural'skij Federal'nyj Univ. im. Pervogo Prezidenta Rossii B.N. El'tsina, Ekaterinburg (RU)), e-mail: anna.bryuzgina@mail.ru

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Optimizatsiya uslovij sinteza, kristallicheskaya struktura perovskitopodobnykh soedinenij sostava $YBaCo_{2-y}Zr_zFe_yCu_zO_{6-\delta}$

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(RU)

4 figs.

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YTTRIUM OXIDES; BARIUM OXIDES; IRON OXIDES; COBALT OXIDES; COPPER OXIDES; CERAMICS; CHEMICAL COMPOSITION; TEMPERATURE RANGE 1000-4000 K; X-RAY DIFFRACTION; SOLID SOLUTIONS; LATTICE PARAMETERS; PHASE DIAGRAMS

Influence of copper (II) sulfide synthesis conditions on the sorption properties of produced particles

S37

Bulgakova, A.V. (GNU NTK Inst. Monokristallov NAN Ukrainy, Khar'kov (UA)), e-mail: bulgakova.alena.vladimirovna@gmail.com

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COPPER SULFIDES; SORPTIVE PROPERTIES; CHEMICAL PREPARATION; PH VALUE; TEMPERATURE RANGE 0273-0400 K; CADMIUM NITRATES; CHEMICAL COMPOSITION

The sequence of basic and impurity phases formation during the solid phase synthesis of bismuth ferrite

S37

Verbenko, I.A. (NII Fiziki Yuzhnogo Federal'nogo Univ., Rostov-na-Donu (RU)), e-mail: ilich001@yandex.ru

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(RU)

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FERRITES; BISMUTH COMPOUNDS; FERROELECTRIC MATERIALS; CHEMICAL PREPARATION; TEMPERATURE RANGE 0400-1000 K; TEMPERATURE RANGE 1000-4000 K; IRON OXIDES; BISMUTH OXIDES; CHEMICAL COMPOSITION

Chemical decorating the surface of ZnO nanoparticles as a way of engineering the band gap

S77

Vokhmintsev, K.V. (IMET RAN, Moscow (RU)), e-mail: VoKirill@gmail.com

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Khimicheskoe dekorirovanie poverkhnosti nanochastits ZnO kak sposob inzhenerii shiriny zapreshchennoj zony

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ZIRCONIUM OXIDES; INTERFACES; SURFACES; INFRARED SPECTRA; COMPOSITE
MATERIALS

Sorbent based on modified hydroxyapatite for water treatment

S37

Vu Tkhi Lien (Belgorodskij Gosudarstvennyj Natsional'nyj Issledovatel'skij Univ. NIU BelGU, Belgorod (RU)), e-mail: bimatcuahanhphuc.2012@yandex.ru

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Sorbent na osnove modifitsirovannykh gidroksiapatitov dlya vodoochistki

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(RU)

3 refs.

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ADSORBENTS; SORPTIVE PROPERTIES; WATER TREATMENT; APATITES;
HYDROXIDES; MODIFICATIONS; CARBONATES; SILICON COMPOUNDS; CHEMICAL
COMPOSITION; IRON IONS

Study of phase equilibria in the system Sm-Ca-Fe-Co-O

S37

Galajda, A.P. (Ural'skij Federal'nyj Univ., Ekaterinburg (RU)), e-mail: anastasia.galayda@yandex.ru

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(RU)

1 fig., 2 tabs.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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CHEMICAL COMPOSITION; TEMPERATURE RANGE 1000-4000 K; ANNEALING;
PHASE STUDIES; QUANTITATIVE CHEMICAL ANALYSIS; PHASE DIAGRAMS;
CRYSTAL STRUCTURE

The continuous microreactor synthesis of silver nanoclusters

S77

Gerasina, Yu.S. (Donskoj Gosudarstvennyj Tekhnicheskij Univ., Rostov-na-Donu (RU)), e-mail: ygerasina@gmail.com

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Nepreryvnyj mikroreaktornyj sintez nanoklasterov serebra

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SILVER; HYDRODYNAMICS; FLUID FLOW; NANOSTRUCTURES; SYNTHESIS;
NANOPARTICLES; CITRATES; BORON HYDRIDES; APPROPRIATE TECHNOLOGY

Preparation and physicochemical properties of compounds in the system $\text{La}_{2-x}\text{Sr}_x\text{Ni}_1-y\text{Fe}_y\text{O}_{4-\delta}$

S37

Gilev, A.R. (FGAOU VPO Ural'skij Federal'nyj Univ. im. Pervogo Prezidenta Rossii B.N. El'tsina, Ekaterinburg (RU)), e-mail: dip-90@mail.ru

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2 figs.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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CHEMICAL COMPOSITION; SOLID SOLUTIONS; ANNEALING; TEMPERATURE
RANGE 1000-4000 K; TEMPERATURE DEPENDENCE; ELECTROMOTIVE FORCE;
ELECTRIC CONDUCTIVITY; PRESSURE DEPENDENCE

Growing nanoscale semiconductor heterostructures with sharp p-n-junctions by MOCVD

S77

Gorskij, A.A. (Moskovskij Gosudarstvennyj Univ. Tonkikh Khimicheskikh Tekhnologij imeni M.V. Lomonosova - MITKhT im. M.V. Lomonosova, Moscow (RU)), e-mail: andrey.gorskiy@list.ru

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INDIUM ARSENIDES; GALLIUM ARSENIDES; HYDROGEN

A method for producing developed surface of copper icosahedral particles

S77

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POROUS MATERIALS

Optimization of methods for the preparation, structure and properties of oxides in the systems Sm-Ba-Cu-Fe-O and Sm-Sr-Cu-O

S37

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Immobilization of gold nanoparticles by hydrated zirconia and its surface activity

S37

Do Tkhyui Maj (Rossijskij Univ. Druzhy Narodov, Moscow (RU))

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Synthesis of luminescent soluble bivalent europium complex $\text{EuCl}_2 \cdot 2(\text{THF})_2$ in THF

S37

Eliseeva, S.M. (Federal'noe Gosudarstvennoe Byudzhethoe Uchrezhdenie Nauki Inst. Neftekhimii i Kataliza RAN, Ufa (RU)), e-mail: eliseevasm@yandex.ru

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$\text{EuCl}_2 \cdot 2(\text{THF})_2$ v TGF

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LUMINESCENCE

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Physicochemical bases of chlorosilanes synthesis by the reaction of catalytic

disproportionation of trichlorosilane

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Ivanov, A.S. (FGBOU VPO Nizhegorodskij Gosudarstvennyj Tekhnicheskij Univ. imeni R.E. Alekseeva, Nizhnij Novgorod (RU)), e-mail: sasha_nevsky@mail.ru

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The mechanism of catastrophic oxidation of copper

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Klimashin, A.A. (IMET RAN, Moscow (RU)), e-mail: klim@imet.ac.ru

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Titanium(IV) oxohydroxo phosphates - sorbents for reliable immobilization of metal cations

S37

Kornejko, R.I. (Federal'noe Gosudarstvennoe Byudzhetnoe Uchrezhdenie Nauki Inst. Khimii i Tekhnologii Redkikh Ehlementov i Mineral'nogo Syr'ya im. I.V. Tananaeva Kol'skogo Nauchnogo Tsentra RAN (IKhTREhMS KNTs RAN), Apatity (RU)), e-mail: korneikov@chemy.kolasc.net.ru

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Character of the conductivity and the nature of the charge carriers in the MeMoO₄ {Me=Ca, Sr}

S37

Koteneva, E.A. (Ural'skij Federal'nyj Univ. im. Pervogo Prezidenta Rossii El'tsina B.N., Ekaterinburg (RU)), e-mail: kotenyoaelena@yandex.ru

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Solid solutions in the system Sm-Sr-Co-O: synthesis, crystal structure and oxygen nonstoichiometry

S37

Maklakova, A.V. (Ural'skij Federal'nyj Univ. im. Pervogo Prezidenta Rossii El'tsina B.N., Ekaterinburg (RU)), e-mail: anastasia_maklakova@mail.ru

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Tverdye rastvory v sisteme Sm-Sr-Co-O: sintez, kristallicheskaya struktura i kislorodnaya nestekhiometriya

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Development of polishing etching agents based on bromine separate H₂O₂-HBr solutions for removing thin layers from the surface of lead telluride monocrystals

S37

Malanich, G.P. (Inst. Fiziki Poluprovodnikov im. V.E. Lashkareva NAN Ukrainy, Kiev (UA)), e-mail: galya_malanich@mail.ru

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Razrabotka poliruyushchikh travitelej na osnove bromvydelyayushchikh rastvorov H₂O₂ - HBr dlya udaleniya tonkikh sloev s poverkhnosti monokristallov tellurida svintsya

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CHEMICAL POLISHING; CONCENTRATION RATIO

Growing crystals β -Ga₂O₃ from intrinsic melt in a sapphire crucible **S36**

*Maslov, V.N. (OOO Sovershennye Kristally, Sankt-Peterburg (RU)), e-mail:
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NITRIDES; MOLYBDENUM; ANNEALING; TEMPERATURE RANGE 0065-0273 K

Tetrahedral structure and thermodynamic properties of the phases in the **Ag₂Se-CdSe-SnSe-Se system**

S37

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619 p.

SILVER SELENIDES; CADMIUM SELENIDES; TIN SELENIDES; BINARY ALLOY SYSTEMS; ELECTROMOTIVE FORCE; SOLID SOLUTIONS; FREE ENTHALPY; ENTROPY; CHEMICAL COMPOSITION

Perovskite-like oxides $\text{SmBaCo}_{2-x}\text{Fe}_x\text{O}_{6-\delta}$ ($x=0; 0.6$): defect structure and electrotransport properties

S37

Mychinko, M.Yu. (Ural'skij Federal'nyj Univ. im. Pervogo Prezidenta Rossii El'tsina B.N., Ekaterinburg (RU)), e-mail: bloodbear1402@bk.ru

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1 fig.

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Exciton states in semiconductor quasi-zero-dimensional nanosystems

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Naumenko, O.V. (NPU im. M.P. Dragomanova, Evpatoriya (RU); Inst. Metallofiziki im. G.V. Kurdyumova NAN Ukrainy, Kiev (UA)), e-mail: oksanochka97@gmail.com, pokutnyi_sergey@inbox.ru

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SEMICONDUCTOR MATERIALS; NANOSTRUCTURES; EXCITONS; EFFECTIVE MASS; QUASI PARTICLES; HOLES; PERMEABILITY

Electric arc synthesis of higher fullerenes

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Nekrasov, V.M. (Inst. Problem Khimicheskoy Fiziki RAN, Chernogolovka (RU)), e-mail: Inekrasov@inbox.ru

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Research of dynamics of carbon nanofibers treatment in the nitric acid solutions S37

Nemzorova, M.A. (Novosibirskij Gosudarstvennyj Tekhnicheskij Univ., Novosibirsk (RU)), e-mail: nemzorova.m@mail.ru

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NITRIC ACID; CARBON FIBERS; CARBON NANOTUBES; CLEANING; MIXTURES;
CONCENTRATION RATIO; TIME DEPENDENCE; IGNITION

**New approaches to the synthesis of derivatives of the cluster anion
[B₁2H₁2]²⁻- with exopolyhedral bonds B-OH**

S37

Ogarkov, A.I. (IMET RAN, Moscow (RU)), e-mail: ogarkov_al@rambler.ru

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Physicochemical properties of perovskite-like compounds BaCo_{1-x-y}Me_xO_{3-d}

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Rudik, V.V. (Ural'skij Federal'nyj Univ. im. Pervogo Prezidenta Rossii El'tsina B.N., Ekaterinburg (RU)), e-mail: vika_19_93@mail.ru

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The growth rate of carbon nanostructures under catalytic pyrolysis of alcohol S77

*Rybin, V.V. (FGOU VPO Ul'yanovskij Gosudarstvennyj Univ., Ul'yanovsk (RU)), e-mail:
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CHEMICAL REACTION KINETICS

Ecologically pure piezoceramic materials of new generation

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Sadykov, Kh.A. (NII Fiziki Yuzhnogo Federal'nogo Univ., Rostov-na-Donu (RU)), e-mail: hizir-2010@mail.ru

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Sipyagina, N.A. (Inst. Fiziologicheskii Aktivnykh Veshchestv RAN, Chernogolovka (RU)), e-mail: dolmatin_89@mail.ru

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Structural and chemical characteristics of decationized clinoptilolite natural forms

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Spiridonov, A.M. (Severo-Vostochnyj Federal'nyj Univ. im. M.K. Ammosova, Yakutsk (RU)), e-mail: spalmik@mail.ru

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3 refs.

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COMPOSITION; CHEMICAL STATE; SORPTIVE PROPERTIES

The structural characteristics and physicochemical properties of oxyfluoride Ba₃In₂O₅F₂

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*Tarasova, N.A. (Ural'skij Federal'nyj Univ. im. Pervogo Prezidenta Rossii El'tsina B.N.,
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COMPOSITION; CUBIC LATTICES; THERMAL GRAVIMETRIC ANALYSIS;
TEMPERATURE DEPENDENCE; HUMIDITY

Structure and electrical properties of monocrystal EuBaCo_{1-x}Y_xO_{5-y}F_y

S37

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COMPOUNDS; CHEMICAL COMPOSITION; TEMPERATURE RANGE 0013-0065 K;

TEMPERATURE RANGE 0065-0273 K; TEMPERATURE RANGE 0273-0400 K;

TEMPERATURE RANGE 0400-1000 K; TEMPERATURE RANGE 1000-4000 K; ELECTRIC

CONDUCTIVITY; TEMPERATURE DEPENDENCE

Effect of ultrasonic treatment on the formation of adsorption layers of silver nanoparticles on the surface of silicon

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Tepanov, A.A. (MGU imeni M.V. Lomonosova, Khimicheskij Fakul'tet, Moscow (RU)), e-mail: atepanov@inbox.ru

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Determination of the conditions of chemical deposition from the In₂S₃ and In₂Se₃ thin films solutions and their preparation

S37

Tulenin, S.S. (Ural'skij Federal'nyj Univ. im. Pervogo Prezidenta Rossii El'tsina B.N., Ekaterinburg (RU)), e-mail: stast1989@mail.ru

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p. 419-421

(RU)

2 refs., 2 figs.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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THIN FILMS; PRECIPITATION; INDIUM SULFIDES; INDIUM SELENIDES; CHEMICAL COMPOSITION; PH VALUE; ABUNDANCE; CHEMICAL REACTIONS; POTENTIOMETRY; NANOSTRUCTURES

The preparation of CdS-PbS solid solutions using ion-exchange synthesis technology

S37

Forostyanaya, N.A. (Ural'skij Federal'nyj Univ. im. Pervogo Prezidenta Rossii El'tsina B.N., Ekaterinburg (RU)), e-mail: natal-ku8@yandex.ru

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(RU)

2 refs., 1 fig.

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CADMIUM SULFIDES; LEAD SULFIDES; SOLID SOLUTIONS; CHEMICAL PREPARATION; ION EXCHANGE; THIN FILMS; AQUEOUS SOLUTIONS; TEMPERATURE RANGE 0273-0400 K; PH VALUE; MICROSTRUCTURE; CHEMICAL COMPOSITION

Investigation into the catalytic activity of the composite material based on oxide compounds of transition metal

S36

Khramenkova, A.V. (FGBOU VPO YuRGPU (NPI) imeni M.I. Platova, Novocheerkassk (RU)), e-mail: anna.vl7@yandex.ru

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(RU)

3 refs., 1 fig.

Molybdenum, cobalt, nickel, iron oxide compounds

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COMPOSITE MATERIALS; MOLYBDENUM OXIDES; ELECTROPLATING; CATALYTIC EFFECTS; REDUCTION; AQUEOUS SOLUTIONS; ELECTROLYTES; NANOSTRUCTURES

Preparation of nanosized system SiO₂-superplasticizer Sika[®] in the presence of acetic acid
S77

Chebakina, V.K. (Voronezhskij Gosudarstvennyj Arkhitekturno-Stroitel'nyj Univ., Voronezh (RU)), e-mail: chebakina.victoria@yandex.ru

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Poluchenie nanorazmernoj sistemy SiO₂ - superplastifikator Sika[®] v prisutstvii uksusnoj kisloty

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(RU)

3 refs., 2 figs., 1 tab.

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The effects of the cluster disorder in the processes of heat treatment of copper

nanoparticles

S77

Chepkasov, I.V. (Khakasskij Gosudarstvennyj Univ. im. N.F. Katanova, Abakan (RU)), e-mail: ilya_chepkasov@mail.ru

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(RU)

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COPPER; NANOPARTICLES; NANOSTRUCTURES; ATOMIC CLUSTERS; HEATING; SYNTHESIS; TEMPERATURE RANGE 0065-0273 K; TEMPERATURE RANGE 0273-0400 K; TEMPERATURE RANGE 0400-1000 K; TEMPERATURE RANGE 1000-4000 K; SINTERING

Anionic alloying of antimony sulfoiodide in the sonochemical synthesis

S36

Chirkova, D.V. (Yuzhnyj Federal'nyj Univ., Rostov-na-Donu (RU)), e-mail: sdanamail@list.ru

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Preparation of nanosized system SiO₂-superplasticizer Sika[®] ViscoCrete[®] 20HE in the presence of hydrochloric acid
S36

Shvedova, M.A. (Voronezhskij Gosudarstvennyj Arkhitekturno-Stroitel'nyj Univ., Voronezh (RU)), e-mail: marishwedowa@mail.ru

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CHEMICAL REACTION KINETICS; HYDRATION; TENSILE PROPERTIES;
HYDROCHLORIC ACID

Obtaining of titanium alloys in argon-arc furnace

S36

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(RU)

3 figs.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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Study of the beryllium distribution on products of melting of copper-beryllium ligature wastes

S36

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Issledovanie raspredeleniya berilliya po produktam plavki otkhodov medno-berillievoy ligatury

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(RU)

1 fig.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya

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COPPER ALLOYS; BERYLLIUM ALLOYS; MELTING; INDUSTRIAL WASTES;
TEMPERATURE RANGE 1000-4000 K; CHEMICAL COMPOSITION; ABUNDANCE;
FURNACES

Dissolution of oxygen into molten Fe-Ni system containing titanium

S36

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Rastvorimost' kisloroda v rasplavakh sistemy Fe-Ni, sodержashchikh titan

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(RU)

5 refs., 2 figs., 1 tab.

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IRON BASE ALLOYS; NICKEL ALLOYS; TITANIUM ADDITIONS; OXYGEN;
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ABUNDANCE

The research of oxygen activity in the metal during the oxidation refining of stainless steel

S36

Alpatov, A.V. (IMET RAN, Moscow (RU)), e-mail: Alpat72@mail.ru

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(RU)

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Behavior of iron when magnetized roasting of leucoxene concentrate

S36

Anisonyan, K.G. (IMET RAN, Moscow (RU)), e-mail: grikar84@mail.ru

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(RU)

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MAGNETIC PROPERTIES

Heterophase interaction of exogenous refractory nanophases with surfactant in the melt based on iron

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Anuchkin, S.N. (IMET RAN, Moscow (RU)), e-mail: AHC2@yandex.ru

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(RU)

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NANOSTRUCTURES; REFINING; SURFACTANTS; IRON BASE ALLOYS; TIN ALLOYS;
ALUMINIUM OXIDES; MAGNESIUM OXIDES; PARTIAL PRESSURE; FREE ENTHALPY

Physicochemical verification for developing a combined flotation-hydrometallurgical technology of disseminated ore deposits enrichment on the Norilsk industrial region with sulfuric acid leaching of copper-nickel intermediate product of flotation

S37;S36

Bol'shikh, A.O. (IMET RAN, Moscow (RU)), e-mail: bolshih@hotmail.com

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2 refs., 1 tab.

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XI Rossijskaya ezhegodnaya konferentsiya molodykh nauchnykh sotrudnikov i aspirantov Fiziko-khimiya i tekhnologiya neorganicheskikh materialov (s mezhdunarodnym uchastiem).

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FLOTATION; ORE CONCENTRATES; ORE ENRICHMENT; LEACHING; HYDROMETALLURGY; HYDROLYSIS; CHEMICAL COMPOSITION; DEPOSITS; COPPER ALLOYS; NICKEL ALLOYS

On the issue of the industrial implementation of a new process of titanomagnetite concentrates processing using direct iron production methods

S36

Goncharov, K.V. (IMET RAN, Moscow (RU)), e-mail: goncharov-imet@mail.ru

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2 refs.

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ORE CONCENTRATES; MAGNETITE; TITANIUM OXIDES; VANADIUM OXIDES; IRON OXIDES; ORE PROCESSING; HYDROMETALLURGY; LEACHING; ROASTING;

TEMPERATURE RANGE 1000-4000 K

Study of interaction of nanoparticles Al₂O₃ and MgAl₂O₄ with SAA-sulfur in the model melt Fe-S

S36;S77

Grenev, N.Yu. (IMET RAN, Moscow (RU)), e-mail: ngrenev@gmail.com

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6 refs., 1 tab.

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NANOPARTICLES; ALUMINIUM OXIDES; ALUMINATES; MAGNESIUM

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Grekhov, A.P. (Rossijskij Khimiko-Tekhnologicheskij Univ. imeni D.I. Mendeleeva, Moscow (RU)), e-mail: alexeygrekhov@mail.ru

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(RU)

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S77

Guseva, E.S. (Ehngel'sskij Tekhnologicheskij Inst. (Filial) Saratovskogo Gosudarstvennogo Tekhnicheskogo Univ., Ehngel's (RU)), e-mail: deinblick@yandex.ru, tep@techn.sstu.ru

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MANGANESE OXIDES; SODIUM COMPOUNDS; MOLYBDATES; LITHIUM TUNGSTATES; NANOSTRUCTURES; NANOTECHNOLOGY; ELECTRODES; CARBON; MODIFICATIONS; ELECTRIC POTENTIAL; POLARIZATION; ADSORPTION; COMPLEXES

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S37

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Using plasma heating for recovering zinc and lead from steel-making dust
S36

Zhuravleva, O.E. (IMET RAN, Moscow (RU)), e-mail: zhuravleva.lesya@yandex.ru

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(RU)

1 tab.

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Chemical desiliconization of leucoxene concentrate by lime milk in an autoclave
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Zablotskaya, Yu.V. (IMET RAN, Moscow (RU)), e-mail: Nboxclear@gmail.com

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(RU)

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Zagvozdin, V.P. (IMET RAN, Moscow (RU)), e-mail: zagvozdin.v@gmail.com

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(RU)

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NICKEL ORES; REDUCTION; COKE; BITUMINOUS COAL; ANNEALING; TEMPERATURE RANGE 0400-1000 K; TEMPERATURE RANGE 1000-4000 K; IRON OXIDES; MAGNESIUM OXIDES; FERROMAGNETIC MATERIALS

The ion exchange method for the separation and concentration of non-ferrous metals from sulfuric acid leaching solutions

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Ikanina, E.V. (UrFU imeni Pervogo Prezidenta Rossii B.N. El'tsina, Ekaterinburg (RU)), e-mail: ikael@yandex.ru

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Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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Kirpichev, D.E. (IMET RAN, Moscow (RU)), e-mail: dym2004@bk.ru

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6 refs., 2 figs.

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Modeling the interaction between the slag and metal phases in the steel industry, the development of software for the description of technological processes **S36**

Komolova, O.A. (IMET RAN, Moscow (RU)), e-mail: o.a.komolova@gmail.com

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Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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Synthesis of wollastonite in the processing of silica-titanium raw materials **S36**

Kop'ev, D.Yu. (IMET RAN, Moscow (RU)), e-mail: dimas1803@mail.ru

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RUTILE; TITANIUM OXIDES; SILICON OXIDES; ORE ENRICHMENT;

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Lapidus, A.O. (OAO VNIKhT, Moscow (RU)), e-mail: raremetals@mail.ru

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6 refs., 2 figs.

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OPTIMIZATION; IMPURITIES

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Levchuk, O.M. (IMET RAN, Moscow (RU)), e-mail: grachi.2005@gmail.com

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(RU)

5 refs., 2 figs.

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MAGNETIC FIELDS; AMMONIUM HYDROXIDES; TUNGSTEN COMPOUNDS;
RHENIUM COMPOUNDS; ELECTRIC CONDUCTIVITY; TIME DEPENDENCE;
AQUEOUS SOLUTIONS

Sorption of cerium(III) by amino-sulfo-carboxyl cation exchanger from sulfate-chloride solutions

S37

Moisenko, V.A. (Rossijskij Khimiko-Tekhnologicheskij Univ. imeni D.I. Mendeleeva, Moscow (RU)), e-mail: vasya0o@mail.ru

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(RU)

2 refs.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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Preparation of titanium alloys based on titanium iodide

S36

Radushinskij, S.M. (OAO VNIKhT, Moscow (RU)), e-mail: radushinskiy@yandex.ru

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1 fig.

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619 p.

NICKEL ALLOYS; TITANIUM IODIDES; CHEMICAL COMPOSITION; HARDNESS;
TITANIUM ALLOYS; VACUUM FURNACES; IMPURITIES

Electrochemical behavior of thallium in sulfate electrolytes

S37

Usipbekova, E.Zh. (KazNU imeni al'-Farabi, Almaty (KZ)), e-mail: enlik-86taraz@mail.ru

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(RU)

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Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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619 p.

THALLIUM SULFIDES; ELECTROLYTES; ELECTROPLATING; GLASS; CARBON;
ELECTRODES; TEMPERATURE DEPENDENCE; ABUNDANCE

Improving the resistance to erosion of high temperature coatings in super- and hypersonic flows of oxygen plasma

S36

Astapov, A.N. (Moskovskij Aviatsionnyj Inst., Moscow (RU)), e-mail: Lexxa1985@inbox.ru

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(RU)

2 refs.

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619 p.

SURFACE COATING; COMPOSITE MATERIALS; CHEMICAL COMPOSITION; EROSION; WEAR; WEAR RESISTANCE; TEMPERATURE RANGE 1000-4000 K; PRESSURE RANGE; MOLYBDENUM SILICIDES; BORON ALLOYS

Diffraction studies of nanostructured films PbSe doped with iodine

S36

Bakanov, V.M. (FGAO VPO UrFU imeni Pervogo Prezidenta Rossii B.N. El'tsina, Ekaterinburg (RU)), e-mail: Bakanov-EVO@mail.ru

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(RU)

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619 p.

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Investigation of phase formation in the layered systems based on beryllium and copper

S37

Bekbolatova, R.A. (Aktyubinskij Regional'nyj Gosudarstvennyj Univ. im. K. Zhubanova, Aktyubinsk (RU)), e-mail: rai-b@mail.ru

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(RU)

2 refs., 3 figs.

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SOLID SOLUTIONS; INTERMETALLIC COMPOUNDS; BERYLLIUM; COPPER; BERYLLIUM ALLOYS; COPPER ALLOYS; PHASE DIAGRAMS; TEMPERATURE RANGE 0400-1000 K; CUBIC LATTICES

Technology for creating a composite material #Left Double Quotation Mark#nitinol-surface layer of tantalum#Right Double Quotation Mark#

S36

Bikbova, G.N. (IMET RAN, Moscow (RU)), e-mail: bikbova_92@mail.ru

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poverkhnostnyj sloj iz tantala#Right Double Quotation Mark#

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619 p.

NICKEL ALLOYS; TITANIUM ALLOYS; COMPOSITE MATERIALS; TANTALUM ALLOYS; PLASMA ARC SPRAYING; NANOMATERIALS

Study of nanosized WO₃-Pb systems before and after heat treatment

S77

Bin, S.V. (FGBOU VPO Kemerovskij Gosudarstvennyj Univ., Kemerovo (RU)), e-mail: epsur@kemsu.ru

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Issledovanie nanorazmernykh sistem WO₃-Pb do i posle termoobrabotki

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619 p.

NANOSTRUCTURES; TUNGSTEN OXIDES; LEAD; TEMPERATURE RANGE 0400-1000 K; TEMPERATURE RANGE 0273-0400 K; HEAT TREATMENTS; DENSITY; THIN FILMS; ABSORPTION SPECTRA

Structural and phase transformations in layered intermetallic coatings of the Al-Ni system in the high temperature heat treatment

S36

Bogdanov, A.I. (Volgogradskij Gosudarstvennyj Tekhnicheskij Univ., Volgograd (RU)), e-mail: mv@vstu.ru

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(RU)

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619 p.

INTERMETALLIC COMPOUNDS; COMPOSITE MATERIALS; NICKEL ALLOYS; ALUMINIUM ALLOYS; TEMPERATURE RANGE 1000-4000 K; ANNEALING; SURFACE COATING; LAYERS

Corrosion-electrochemical behavior of the system #Left-Pointing Double Angle Quotation Mark#steel St.3 - plasma sprayed coating (FBKh 6-2; Nanosteel)#Right-Pointing Double Angle Quotation Mark#

S36

Gladkikh, N.A. (Natsional'nyj Issledovatel'skij Tekhnologicheskij Univ. MISiS (NITU MISiS), Moscow (RU)), e-mail: fuchsia32@bk.ru

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Korrozionno-ehelektrokhimicheskoe povedenie sistem #Left-Pointing Double Angle Quotation
Mark#stal' St.3 - plazmennoe napylennoe pokrytie (FBKh 6-2; Nanosteel)#Right-Pointing
Double Angle Quotation Mark#

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1 ref.

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CORROSION PROTECTION; PLASMA ARC SPRAYING; CARBON STEELS; POWDERS;
NANOSTRUCTURES; CHROMIUM; MOLYBDENUM; ABUNDANCE; PROTECTIVE
COATINGS

The electrical properties of thin-film ferromagnetic-ferroelectric nanocomposites S77

Gorshkov, A.G. (Voronezhskij Inst. GPS MChS Rossii, Voronezh (RU)), e-mail: gorasic@mail.ru

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Ehlektricheskie svoystva tonkoplnochnykh nanokompozitov ferromagnetik-segnetoehlektrik

p. 502-503

(RU)

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk,
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NANOCOMPOSITES; THIN FILMS; PZT; PERMITTIVITY; FERROELECTRIC MATERIALS; ELECTRIC CONDUCTIVITY; CHEMICAL COMPOSITION

The use of carbon nanomaterials to increase the heat transfer coefficient of the modified electrochemical aluminum oxide coatings

S77

Gravin, A.A. (FGBOU VPO Tambovskij Gosudarstvennyj Tekhnicheskij Univ., Tambov (RU)), e-mail: nagval_89@mail.ru

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p. 503-504

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619 p.

NANOMATERIALS; CARBON NANOTUBES; ALUMINIUM ALLOYS; HEAT TRANSFER; ANODIZATION; ELECTROLYTES

Study patterns of high-temperature deformation and fracture of the surface layer of aluminized steels

S36

Gul'tsev, E.O. (Volgogradskij Gosudarstvennyj Tekhnicheskij Univ., Volgograd (RU)), e-mail: omd@vstu.ru

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Fiziko-khimiya i tekhnologiya neorganicheskikh materialov (s mezhdunarodnym uchastiem)
Issledovanie zakonornosti vysokotemperaturnoj deformatsii i razrusheniya poverkhnostnogo
sloya alitirovannykh stalej

p. 505-506

(RU)

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SATURATION; STEEL-CR18NI10TI; ALUMINIUM OXIDES; ALUMINIUM; DIFFUSION
COATING; DIFFUSION COATINGS; CREEP; DEFORMATION; TEMPERATURE RANGE
0400-1000 K

Precipitation of the nanofilm in the apparatus Plasma focus using cathode evaporation of the target material

S77

Demin, A.S. (IMET RAN, Moscow (RU)), e-mail: casha@bk.ru

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Osazhdenie nanoplenki v ustanovke plazmennyy fokus s ispol'zovaniem katodnogo ispareniya
materiala misheni

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(RU)

3 refs., 4 figs.

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Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst.
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NANOSTRUCTURES; THIN FILMS; PLASMA FOCUS; CATHODES; EVAPORATION;
SPRAY COATING; SPRAYED COATINGS; MICROSTRUCTURE

Tribological properties of the Ti-Cu intermetallic coating

S36

Evstropov, D.A. (Volgogradskij Gosudarstvennyj Tekhnicheskij Univ., Volgograd (RU)), e-mail: mv@vstu.ru

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Magneto receptive effect in a multilayer structure

{{(Co₄1Fe₃9B₂0)₃3.9(SiO₂)₆6.1}/[Te₃Bi₂]}₁0₁

S36

Zhilova, O.V. (Voronezhskij Gosudarstvennyj Tekhnicheskij Univ., Voronezh (RU)), e-mail: zhilova105@mail.ru

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Magnitorezistivnyj ehffekt v mnogoslojnoj strukture

{[(Co₄1Fe₃9B₂0)₃3₉(SiO₂)₆6₁]/[Te₃Bi₂]}₁0₁

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(RU)

2 figs.

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NANOSTRUCTURES; LAYERS; INTERMETALLIC COMPOUNDS; COBALT ALLOYS;
IRON ALLOYS; BORON ALLOYS; SILICON OXIDES; TELLURIUM ALLOYS; BISMUTH
ALLOYS; CHEMICAL COMPOSITION; MAGNETIC PROPERTIES

The change in the optical properties of nanosized cobalt films before and after the thermal action at T=473 K

S77

Zhuravleva, S.V. (FGBOU VPO Kemerovskij Gosudarstvennyj Univ., Kemerovo (RU)), e-mail: epsur@kemsu.ru

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vozdejstviya pri T=473 K

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2 refs., 3 figs.

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The influence of electromechanical treatment on the structure and microhardness of a heat-treated in vacuum plasma coating FBKh6-2

S36

Ivannikov, A.Yu. (IMET RAN, Moscow (RU)), e-mail: ivannikov-a@mail.ru

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(RU)

2 refs., 1 fig.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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PLASMA ARC SPRAYING; MICROSTRUCTURE; NANOSTRUCTURES;
MICROHARDNESS; HEAT TREATMENTS; TEMPERATURE RANGE 1000-4000 K;
PLASTICITY

Electrochemical deposition of nanoparticles YSZ on the surface of dense cathodes LSM obtained by pyrolysis and solid-phase synthesis

S36

Kalinina, E.G. (IEhF UrO RAN, Ekaterinburg (RU)), e-mail: linina@iep.uran.ru

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Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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Phase composition of the surface of wear-resistant surfacing after electron beam treatment

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Kapralov, E.V. (Sibirskij Gosudarstvennyj Industrial'nyj Univ., Novokuznetsk (RU)), e-mail: gromov@physics.sibsiu.ru

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619 p.

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CARBIDES; CHROMIUM CARBIDES; IRON CARBIDES; WELDING; NANOPARTICLES

Production of a metal oxide nanostructured nonmetallic inorganic coating on zirconium alloys by the method of microplasma oxidation

S77

Konstantinova, T.A. (Natsional'nyj Issledovatel'skij Tomskij Gosudarstvennyj Univ., Tomsk (RU)), e-mail: konstantinova.ta9@gmail.com

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(RU)

2 refs., 1 fig.

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ZIRCONIUM ALLOYS; CERMETS; ELECTRODEPOSITED COATINGS;
NANOSTRUCTURES; CHEMICAL COMPOSITION; LAYERS; OXIDATION

Wear-resistant nickel-boron (Ni-B) coating

S36

Leshchev, K.A. (FGUP VIAM, Moscow (RU)), e-mail: kirleshev@gmail.com

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WEAR RESISTANCE; MICROHARDNESS; NICKEL BASE ALLOYS; BORON ALLOYS; CHROMIUM STEELS; POROSITY; MICROSTRUCTURE

Formation of thin films ZnO:Ag by ion-beam methods and investigation of their structural and optical properties

S36

Lyadov, N.M. (Kazanskij Fiziko-Tekhnicheskij Inst. im. E.K. Zavojskogo KazNTs RAN, Kazan' (RU)), e-mail: nik061287@mail.ru

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619 p.

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Microscopy of porous silicon formed by the implantation of metal ions

S77

Lyadov, N.M. (Kazanskij Fiziko-Tekhnicheskij Inst. im. E.K. Zavojskogo KazNTs RAN, Kazan' (RU)), e-mail: nik061287@mail.ru

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(RU)

1 ref., 2 figs.

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619 p.

SILICON; POROSITY; ION IMPLANTATION; NANOPARTICLES; SILVER IONS; COPPER IONS; COBALT IONS; MONOCRYSTALS; SYNTHESIS

Controlling the growth rate of amorphous silicon films during magnetron sputtering

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Mitin, D.M. (Saratovskij Gosudarstvennyj Univ. imeni N.G. Chernyshevskogo, Saratov (RU)), e-mail: mitindm@mail.ru

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Upravlenie skorost'yu rosta plenok amorfnogo kremniya pri sinteze metodom magnetronnogo raspyleniya

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2 refs., 1 fig.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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619 p.

SILICON; AMORPHOUS STATE; THIN FILMS; SYNTHESIS; REACTION KINETICS; SPUTTERING; PRESSURE RANGE MILLI PA; PRESSURE DEPENDENCE

Study of abrasive wear resistance of nanostructured coatings based on multicomponent oxides

S77

Morozov, A.V. (Inst. Problem Mekhaniki im. A.Yu. Ishlinskogo RAN, Moscow (RU)), e-mail: morozovalexei@mail.ru

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Issledovanie abrazivnoj iznosostojkosti nanostrukturirovannykh pokrytij na osnove mnogokomponentnykh oksidov

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2 refs., 1 fig.

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ALUMINIUM OXIDES; ZIRCONIUM OXIDES; YTTRIUM OXIDES; WEAR RESISTANCE; NANOSTRUCTURES; THIN FILMS; STEEL-CR18NI10TI; SURFACE COATING

Damage to tungsten by deuterium ion and deuterium plasma pulsed fluxes during irradiation in plants Plasma focus

S36

Morozov, E.V. (IMET RAN, Moscow (RU)), e-mail: lieutenant@list.ru

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Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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TUNGSTEN; DEUTERIUM IONS; PHYSICAL RADIATION EFFECTS; SURFACE PROPERTIES; PULSE TECHNIQUES; X-RAY SPECTRA; CHROMIUM-NICKEL STEELS; CRACKS

Protective properties of coatings obtained by plasma electrolytic oxidation followed by polymer application

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The effect of annealing conditions on the properties of Cu-Zn-Sn-Se thin films, obtained by the single-stage electrodeposition method

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Features of the formation of the weld metal structure when nanopowders are introduced into the weld zone

S36

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Smirnova, M.A. (FGBUN Inst. Obshchej i Neorganicheskoy Khimii im. N.S. Kurnakova RAN, Moscow (RU)), e-mail: smirnova_macha1989@mail.ru

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Study of the thermostimulated oxidation mechanism of nanosized bismuth films

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Surovaya, V.Eh. (Kuzbasskij Gosudarstvennyj Tekhnicheskij Univ. im. T.F. Gorbacheva, Kemerovo (RU)), e-mail: sur.vik@mail.ru

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An investigation of the phase-structural state of FeZrN and FeTiB films obtained by magnetron sputtering with target heating

S37

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Tregubov, I.M. (Voronezhskij Gosudarstvennyj Tekhnicheskij Univ., Voronezh (RU)), e-mail: ilia.tregubov@mail.ru

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Mikrotverdost' nanostrukturnykh pokrytij $Co_x(Al_2O_3)_{1-x}$, $Co_x(SiO_2)_{1-x}$ i $Co_x(CaF_2)_{1-x}$

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1 fig.

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Features of the effect of nanodiamonds on the characteristics of composite electroplating coatings based on copper

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Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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The magnetic structure of nanocrystalline and magnetically soft films

Fe₇Zr₇N₁ with induced magnetic anisotropy

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Shapiev, I.M. (FGBOU VPO Dagestanskij Gosudarstvennyj Univ., Makhachkala (RU)), e-mail: s-deybuk@mail.ru

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Shafeev, M.R. (Tol'yattinskij Gosudarstvennyj Univ., Tol'yatti (RU)), e-mail: shelf-tlt@yandex.ru

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New composite electrochemical coatings based on zinc

S36

Shevchenko, T.Yu. (Ehngel'sskij Tekhnologicheskij Inst. (Filial) FGBOU VPO Saratovskij

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Effect of hot isostatic pressing on the structure and properties of high-strength cast aluminum alloys of a new generation - nikalines ATs6N0,5Zh and ATs6N4

S36

Akopyan, T.K. (IMET RAN, Moscow (RU)), e-mail: aktorgom@gmail.com

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S36

Ataev, Sh.O. (IMET RAN, Moscow (RU)), e-mail: shuxrat-muz@mail.ru

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STEEL-CR18NI10TI; THERMAL FATIGUE; MICROSTRUCTURE; DEFORMATION;
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Structure and properties of low-temperature thermoelectrical materials based on bismuth and antimony chalcogenides obtained by the method of equal-channel angular pressing

S36

*Bogomolov, D.I. (Natsional'nyj Issledovatel'skij Tekhnologicheskij Univ. MISiS, Moscow (RU)),
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Creation of a technology for the production of bimetallic materials for electrical contacts by the method of explosive action

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Krasnoperova, Yu.G. (Inst. Fiziki Metallov UrO RAN, Ekaterinburg (RU)), e-mail: highpress@imp.uran.ru

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Development of plasticity of a powder metal alloy of the iron-tungsten-molybdenum-chromium-vanadium system under thermomechanical stretching
S36

Kutepov, S.N. (Tul'skij Gosudarstvennyj Pedagogicheskij Univ. im. L.N. Tolstogo, Tula (RU)), e-mail: kutepov.sergei@mail.ru

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Diffusion of an isotope Ni63 in an intensively plastically deformed alloy Cu-Cr-Hf

S36

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COPPER BASE ALLOYS; CHROMIUM ALLOYS; HAFNIUM ALLOYS; NICKEL 63; DIFFUSION; DEFORMATION; PHYSICAL RADIATION EFFECTS; DISLOCATIONS; ANNEALING

The effect of equal-channel angular pressing and annealing regimes on the microstructure, texture, and anisotropy of the strength properties of a magnesium alloy MA2-pi

S36

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Metal heterophase system Fe-6W-5Mo-4Cr-2V of traditional clinker limit and the resource of its deformation capacity under external thermal and mechanical influences

S36

Tsurin, Yu.S. (Tul'skij Gosudarstvennyj Pedagogicheskij Univ. im. L.N. Tolstogo, Tula (RU)), e-mail: tsuker@yandex.ru

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XI Rossijskaya ezhegodnaya konferentsiya molodykh nauchnykh sotrudnikov i aspirantov Fiziko-khimiya i tekhnologiya neorganicheskikh materialov (s mezhdunarodnym uchastiem) Metallicheskaya geterofaznaya sistema Fe-6W-5Mo-4Cr-2V traditsionnogo slitkovogo predela i resurs ee deformatsionnoj sposobnosti pri vneshnikh teplovykh i mekhanicheskikh vozdeystviyakh

p. 583-584

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IRON BASE ALLOYS; TUNGSTEN ALLOYS; MOLYBDENUM ALLOYS; CHROMIUM ALLOYS; VANADIUM ALLOYS; CHEMICAL COMPOSITION; DEFORMATION; PLASTICITY

Influence of chromium content on the structure and properties of ultrafine-grained Cu-Cr alloys

S36

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COPPER BASE ALLOYS; CHROMIUM ALLOYS; CHEMICAL COMPOSITION; TORSION; DEFORMATION; MICROHARDNESS; ELECTRIC CONDUCTIVITY; TEMPERING; TWINNING; DISLOCATIONS

Influence of microplastic deformation on the intensification of the processes of chemical-thermal treatment

S36

Shtolik, T.N. (Ob'edinennyj Inst. Mashinostroeniya NAN Belarusi, Minsk (BY)), e-mail: tatianasholik@gmail.com

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619 p.

CHROMIUM STEELS; TEMPERING; DEFORMATION; DIFFUSION; CARBON;
NITROGEN; TEMPERATURE RANGE 0400-1000 K; MICROSTRUCTURE

Manganese-containing nanosized particles stabilized by high-pressure polyethylene for modifying polypropylene fibers

S77

Biryukova, M.I. (IMET RAN, Moscow (RU)), e-mail: marino4cka.b@yandex.ru

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1 tab.

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619 p.

POLYPROPYLENE; NANOPARTICLES; MODIFICATIONS; MANGANESE; COMPOSITE MATERIALS; FIBERS; POLYETHYLENES; CRACKS; HARDNESS; WEAR RESISTANCE

New polyurethane paint composition with protective properties

S36

Galkina, N.V. (Kazanskij Natsional'nyj Issledovatel'skij Tekhnologicheskij Univ., Kazan' (RU)), e-mail: galkinatasha@yandex.ru

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(RU)

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619 p.

POLYURETHANES; PAINTS; NANOSTRUCTURES; PROTECTIVE COATINGS; PIPELINES; SYNTHESIS

Investigation of the structure and properties of 3D-reinforced carbon fiber composites of the grade of the face #Left-Pointing Double Angle Quotation Mark#Grani#Right-Pointing Double Angle Quotation Mark#

S36

Gareev, A.R. (OAO Nauchno-Issledovatel'skij Inst. Konstruktsionnykh Materialov na Osnove Grafita NIIgrafit, Moscow (RU)), e-mail: gareyev@niiigrafit.org

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Issledovanie struktury i svojstv trekhmerno-armirovannykh ugleplastikov marki #Left-Pointing Double Angle Quotation Mark#Grani#Right-Pointing Double Angle Quotation Mark#

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619 p.

COMPOSITE MATERIALS; CARBON FIBERS; REINFORCED MATERIALS;
OPTIMIZATION; MICROSTRUCTURE; TENSILE PROPERTIES

**Synthesis of hybrid nanocomposites of poly (p-phenylenevinylene) - copper:
study of structure and optical properties**

S77

Gor'kov, K.V. (Inst. Neftekhimicheskogo Sinteza im. A.V. Topchieva RAN, Moscow (RU)), e-mail: gorkovk@yandex.ru

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NANOCOMPOSITES; SYNTHESIS; ALKYLATED AROMATICS; COPRECIPITATION;
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Application of secondary polymers for the modification of cement binders

S36

Grushevskaya, E.N. (Grodzenskij Gosudarstvennyj Univ. imeni Yanki Kupaly, Grodno (BY)), e-mail: grushevskay_en@grsu.by

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4 figs.

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PORTLAND CEMENT; POLYAMIDES; PVC; DENSITY; MODIFICATIONS; OPTIMIZATION; VISCOSITY; BONDING

Elastic fluorescent nanocomposites based on polymethylmethacrylate

S77

Dolzhenkov, V.S. (Moskovskij Gosudarstvennyj Univ. imeni M.V. Lomonosova, Khimicheskij Fakul'tet, Moscow (RU)), e-mail: vdolzhenkov@gmail.com

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SYNTHESIS

Effect of polyalkylene oxides on the properties of polymer matrices based on polymethylmethacrylate

S36

Zhirnov, A.E. (Moskovskij Gosudarstvennyj Univ. imeni M.V. Lomonosova, Khimicheskij Fakul'tet, Moscow (RU)), e-mail: jy@vms.chem.msu.ru

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PMMA; EPOXIDES; CHEMICAL COMPOSITION; NANOSTRUCTURES; VISCOSITY;
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SULFIDES

Synthesis and studies of polyurethanes exhibiting antistatic properties

S37

Zaripov, I.I. (FGBOU VPO Kazanskij Natsional'nyj Issledovatel'skij Tekhnologicheskij Univ., Kazan' (RU)), e-mail: zaripovilnaz@gmail.com

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1 fig.

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POLYURETHANES; COPPER COMPLEXES; COBALT COMPLEXES; CHEMICAL COMPOSITION; CHEMICAL PREPARATION; ELECTRIC CONDUCTIVITY; TENSILE PROPERTIES

Effect of the size of quantum dots on effects when they are included in an elastic matrix based on polymethylmethacrylate

S77

Il'yasov, L.O. (Moskovskij Gosudarstvennyj Univ. imeni M.V. Lomonosova, Khimicheskij Fakul'tet, Moscow (RU)), e-mail: illeo98@mail.ru

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Vliyanie razmera kvantovykh toчек na ehffekty pri ikh vklyuchenii v ehlastichnuyu matritsu na osnove polimetilmetakrilata

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Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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**Structural studies of polymer-silicate composites based on
polytetrafluoroethylene**

S36

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POLYTETRAFLUOROETHYLENE; SPINELS; SERPENTINE; FILLERS; WEAR RESISTANCE; X-RAY DIFFRACTION; SPECTRA; COMPOSITE MATERIALS; FRICTION

Polymer composite cathode for chemical current sources

S37

Kovyneva, N.N. (Saratovskij Gosudarstvennyj Tekhnicheskij Univ. imeni Gagarina Yu.A., Saratov (RU)), e-mail: k.natasha_86@bk.ru

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CATHODES; NITRILES; SOL-GEL PROCESS; ANTIMONY SULFIDES; SOLID ELECTROLYTES; LITHIUM; ANODES; NANOCOMPOSITES

Regulation of the functional properties of polymer composites by introducing a chemically modified filler

S77

Kochetkova, A.S. (Sankt-Peterburgskij Gosudarstvennyj Tekhnologicheskij Inst. (Tekhnicheskij Univ.), Sankt-Peterburg (RU)), e-mail: annywka_08@list.ru

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(RU)

7 refs., 1 fig.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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PVC; NANOCOMPOSITES; NANOPARTICLES; ALUMINIUM OXIDES; ATOMIC FORCE MICROSCOPY; FILLERS; PERMITTIVITY; OPTIMIZATION

Water composites dispersion of polyaniline and polyacrylates

S36

Kochkina, N.V. (Yaroslavskij Gosudarstvennyj Tekhnicheskij Univ., Yaroslavl' (RU)), e-mail: knadi@list.ru

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3 refs., 2 figs.

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619 p.

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The use of pernigraniline form of polyaniline for the modification of epoxy polymeric materials

S36

Kurbatov, V.G. (Yaroslavskij Gosudarstvennyj Tekhnicheskij Univ., Yaroslavl' (RU)), e-mail: kurbatovvg@list.ru

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XI Rossijskaya ezhegodnaya konferentsiya molodykh nauchnykh sotrudnikov i aspirantov Fiziko-khimiya i tekhnologiya neorganicheskikh materialov (s mezhdunarodnym uchastiem) Ispol'zovanie pernigranilinovoj formy polianilina dlya modifikatsii ehпокsidnykh polimernykh materialov

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(RU)

6 refs., 1 fig.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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2014

619 p.

AMINES; EPOXIDES; POLYMERS; CORROSION PROTECTION; COMPOSITE MATERIALS; PROTECTIVE COATINGS

Investigation of the electrophysical properties of epoxy composites based on thermally expanded graphite

S36

Tikhonina, K.V. (Novosibirskij Gosudarstvennyj Tekhnicheskij Univ., Novosibirsk (RU)), e-mail: ktobik@yandex.ru

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Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst.

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New radiation-induced bactericidal film nanomaterials based on carboxymethylchitin and silver nanoparticles

S77

Shirokova, L.N. (INKhS RAN, Moscow (RU)), e-mail: shirokova@ips.ac.ru

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(RU)

1 ref.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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619 p.

NANOPARTICLES; MUCOPOLYSACCHARIDES; NANOCOMPOSITES; SILVER IONS; CHEMICAL RADIATION EFFECTS; SYNTHESIS; FRACTALS

Investigation of possible causes of technological plasticity failures of nickel alloys

S36

Arsenkin, A.M. (IMET RAN, Moscow (RU)), e-mail: alex_arsenkin@yahoo.com

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(RU)

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya

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2014

619 p.

NICKEL BASE ALLOYS; HEAT RESISTING ALLOYS; TEMPERATURE RANGE 1000-4000 K; PLASTICITY; IMPACT TESTS; IMPACT STRENGTH; MICROSTRUCTURE; DENDRITES

Effect of equal-channel angular pressing on the mechanical and functional properties of hot rolled semi-finished products for a long-length product of alloys Ni-Ti

S36

Prosvirnin, V.V. (IMET RAN, Moscow (RU)), e-mail: ProsvirninV@meritogroup.ru

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(RU)

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619 p.

NICKEL ALLOYS; TITANIUM ALLOYS; HOT WORKING; ROLLING; SHAPE MEMORY EFFECT; DEFORMATION; VICKERS HARDNESS

Changes in the structure of the properties of fluorohydroxyapatite (FHA) ceramics under the influence of gamma radiation

S36

Antonova, O.S. (FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkoba Rossijskoj Akademii Nauk, Moscow (RU)), e-mail: osantonova@yandex.ru

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6 refs.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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Application of the EBSD method to study the processes of crack formation in weld joint

S36

Sud'in, V.V. (FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova Rossijskoj Akademii Nauk, Moscow (RU)), e-mail: sudin.vlad@gmail.com

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Primenenie metoda EBSD dlya izucheniya protsessov obrazovaniya treshchin v svarnykh shvakh

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LOW ALLOY STEELS; WELDED JOINTS; CRACK PROPAGATION; ELECTRON DIFFRACTION; METALLOGRAPHY; MICROSTRUCTURE; ELECTRON MICROSCOPY; CRYSTALLIZATION

Mechanical characteristics of steel EK181

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Prosvirnin, D.V. (FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova Rossijskoj Akademii Nauk, Moscow (RU))

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Mekhanicheskie kharakteristiki stali tipa EhK181

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(RU)

1 fig., 1 tab.

Federal'noe Agentstvo Nauchnykh Organizatsij, Moscow (RU); Rossijskaya Akademiya Nauk, Moscow (RU); Otdelenie Khimii i Nauk o Materialakh RAN, Moscow (RU); FGBUN Inst. Metallurgii i Materialovedeniya im. A.A. Bajkova RAN, Moscow (RU); FNM MGU im. M.V. Lomonosova, Moscow (RU); Sovet Molodykh Uchenykh RAN, Moscow (RU); Korporatsiya INSTRON - OOO Novatest, Moscow (RU)

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FERRITIC STEELS; MARTENSITIC STEELS; TENSILE PROPERTIES; YIELD STRENGTH; TEMPERATURE RANGE 0273-0400 K; TEMPERATURE RANGE 0400-1000 K; FATIGUE

Innovative designs and technologies of nuclear power. IV International scientific and technical conference. Book of abstracts

S11;S21

Aksionernoe Obshchestvo Ordena Lenina Nauchno-Issledovatel'skij i Konstruktorskij Inst. Ehnergotekhniki imeni N.A. Dollezhalya, Moscow (RU)

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2016

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(RU, EN)

LEADING ABSTRACT; MEETINGS; NUCLEAR POWER; NUCLEAR POWER PLANTS; SPENT FUELS; WASTES; WASTE MANAGEMENT

IV International scientific and technical conference #Left Double Quotation Mark#Innovative designs and technologies of nuclear power#Right Double Quotation Mark# has been organized and is conducted by JSC NIKIET with support from Rosatom State Corporation, the International Atomic Energy Agency, the Russian Academy of Sciences and the Nuclear Society of Russia. The conference topics include: innovative designs of nuclear facilities for various applications, nuclear fuel and new materials, closed fuel cycle technologies, SNF and RW management, technological answers to nonproliferation problems, small power reactors (stationary, transportable, floatable, propulsion, space), integrated codes of a new generation for safety analysis of nuclear power plants and fuel cycles, controlled fusion

IV международная научно-техническая конференция #Left Double Quotation Mark#Иновационные проекты и технологии ядерной энергетики#Right Double Quotation Mark# организована и проводится АО НИКИЭТ при поддержке Госкорпорации Росатом, Международного агентства по атомной энергии, Российской академии наук и Ядерного общества России. Тематика конференции: инновационные проекты ядерных установок различного назначения; ядерное топливо и новые материалы; технологии замкнутого топливного цикла, обращение с ОЯТ и РАО, технологическое решение проблем нераспространения ядерных оружейных материалов; малая энергетика (стационарные, транспортные, плавучие, транспортные, космические установки); интегральные расчетные коды нового поколения для анализа безопасности ЯЭУ и топливного цикла; управляемый термоядерный синтез

Development of the #Left-Pointing Double Angle Quotation Mark#Generation IV#Right-Pointing Double Angle Quotation Mark# innovative reactor technologies in the framework of GIF

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Patarakin, O.O. (Goskorporatsiya Rosatom, Moscow (RU)); Rachkov, V.I.; Ashurko, Yu.M. (AO GNTs RF-FEhI, Obninsk (RU))

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Razvitie innovatsionnykh reaktornykh tekhnologij chetvertogo pokoleniya v ramkakh Mezhdunarodnogo foruma #Left-Pointing Double Angle Quotation Mark#Pokolenie-4#Right-Pointing Double Angle Quotation Mark#

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NUCLEAR POWER; NUCLEAR POWER PLANTS; REACTOR TECHNOLOGY; FAST REACTORS; GCFR REACTOR; MOLTEN SALT COOLED REACTORS; ROSATOM

Status of #Left-Pointing Double Angle Quotation Mark#Generation-IV#Right-Pointing Double Angle Quotation Mark# EURATOM LFR activities

S21

Alemberti, A. (Ansaldo Nucleare SpA, Genova (IT))

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(RU, EN)

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292 p.

NUCLEAR POWER; LFR REACTOR; MYRRHA FACILITY; AREVA NC; EURATOM;
FAST REACTORS; LIQUID METALS

Analysis of scientific justification for NP nuclear safety

S22

Gordon, B.G. (FBU NTTs YaRB, Moscow (RU))

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SAFETY; REACTOR SAFETY; RADIATION PROTECTION; REACTOR ACCIDENTS;
RADIATION ACCIDENTS; ACCIDENT MANAGEMENT

Detailed design of the BREST-OD-300 reactor facility: development stages and justification

S21

Dragunov, Yu.G.; Lemekhov, V.V.; Moiseev, A.V.; Smirnov, V.S.; Yarmolenko, O.A.; Vasyukhno, V.P.; Cherepnin, Yu.S. (AO NIKIEhT, Moscow (RU))

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(RU, EN)

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Sapozhnikov, A.I. (Rostekhnadzor, Moscow (RU)); Molchanova, G.A.; Paramonov, V.V.; Polyakov, D.N. (FBU NTTs YaRB, Moscow (RU))

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Kuznetsov, V.; Fesenko, G. (MAGATEh, Vena (XA)); Andrianov, A. (NIYaU MIFI, Moscow (RU))

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Chukhlov, A.G. (AO NIKIEhT, Moscow (RU)); Kolomiets, D.O.; Lashko, A.V.; Trubakov, Yu.P.

(AO GNTs RF - FEhI, Obninsk (RU))

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*Natsional'noe Agentstvo po Novym Tekhnologiyam, Ehnergetike i Ustoichivomu
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Tuchek, K.; Novotny, R.; Fyutterer, M.; Timke, T.; Nilsson, K-F.; Khener, P. (Evropejskaya Komissiya, Tsentr Ob''edinennykh Issledovaniy, Petten (XE))

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Innovatsionnye proekty i tekhnologii yadernoj ehnergetiki. IV Mezhdunarodnaya nauchno-tekhnicheskaya konferentsiya

Razrabotka eksperimental'noj ustanovki dlya donormativnykh materialovedcheskikh ispytaniy v tyazhelom zhidkom metalle

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S22;S11

Ministerstvo Obrazovaniya i Nauki Rossijskoj Federatsii, Moscow (RU); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU); Snezhinskiy Fiziko-Tekhnicheskij Inst. NIYaU MIFI, Snezhinsk (RU)

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Nauchnaya sessiya NIYaU MIFI-2016

Nauchnaya sessiya NIYaU MIFI-2016. Sbornik nauchnykh trudov. Zasedanie tematicheskikh seksij po napravleniyu Innovatsionnye yadernye tekhnologii. Tekhnologii zamykaniya yadernogo toplivnogo tsikla i vodorodnaya bezopasnost'

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43 p.

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LEADING ABSTRACT; MEETINGS; FUEL CYCLE; HYDROGEN; STEAM; GASES; MIXTURES; COMBUSTION; WWER TYPE REACTORS; FAST REACTORS; REACTOR ACCIDENT SIMULATION; COMPUTERIZED SIMULATION; COMPUTER CODES

The collection contains scientific papers presented on meeting of the sessions in the field of Innovative nuclear technologies of annual NRNU MEPhI Scientific session on the base of Snezhinsk Physics and Technology Institute of NRNU MEPhI (May, 5-6, 2016). In the papers the problems of nuclear fuel cycle closure technology and hydrogen safety are under consideration

Сборник содержит научные труды, представленные на заседаниях тематических секций по направлению Инновационные ядерные технологии ежегодной Научной сессии НИЯУ МИФИ на базе Снежинского физико-технического института НИЯУ МИФИ (5-6 мая 2016 г.) В работах рассмотрены вопросы технологии замыкания ядерного топливного цикла и водородной безопасности

Change of physico-mechanical properties and structure of austenitic corrosion resistant steels Cr18Ni10Ti and Cr16Ni15Mo3Nb samples at low-temperature irradiation

S36

Bakshutova, G.N.; Makarov, E.I.; Neustroev, V.S. (DITI NIYaU MIFI, AO Innovative nuclear technologies of annual NRNU MEPhI Scientific session on the base of Snezhinsk Physics and Technology Institute of NRNU MEPhI (May, 5-6, 2016), Dimitrovgrad (RU)), e-mail: galina.bakshutova.94@mail.ru

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Izmenenie fiziko-mekhanicheskikh svojstv i struktury obraztsov austenitnykh korrozionno-stojkikh stalej Kh18N10T i Kh16N15M3B pri nizkotemperaturnom obluchenii

p. 4-6

(RU)

3 refs., 3 figs., 2 tabs.

Ministerstvo Obrazovaniya i Nauki Rossijskoj Federatsii, Moscow (RU); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU); Snezhinskij Fiziko-Tekhnicheskij Inst. NIYaU MIFI, Snezhinsk (RU)

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43 p.

STEEL-CR16NI15MO3NB; STEEL-CR18NI10TI; PHYSICAL RADIATION EFFECTS; MICROSTRUCTURE; GRAIN SIZE; MICROHARDNESS; TEMPERATURE RANGE 0273-0400 K; RADIATION DOSES

The investigation results of steel Cr18Ni10Ti (1) and Cr16Ni15Mo3Nb (2) samples irradiated in SM-3 reactor are given. All samples tested show the structure which is specific to austenitic steels. Average grain diameter for steel 2 samples is equal to 9-11 μm and for steel 1 samples - 20-25 μm . It is pointed out that no visible structure changes is detected with the growth of damaging dose. Microhardness of steel 1 is within the range 142-181 kgf/mm^2 and of steel 2 - 162-220 kgf/mm^2 . There is weak dose dependence of microhardness values for both steels in the dose range 25-55 dpa. Investigation of temperature dependence of microhardness value distribution across Charpy sample shows weak temperature dependence of microhardness in the range of irradiation temperature (90-133 deg C)

Представлены результаты исследования облученных в реакторе СМ-3 образцов сталей X18H10T (1) и X16H15M3B (2). Во всех исследуемых образцах обнаружена структура, характерная для сталей аустенитного класса. Средний размер зерен для образцов стали 2 равняется 9-11 мкм, а для образцов стали 1 - 20-25 мкм. Отмечается, что не было обнаружено видимых изменений в структуре с ростом повреждающей дозы. Микротвердость для стали 1 находится в диапазоне 142-181 кгс/мм², а для стали 2 - 162-220 кгс/мм². Зависимости величин микротвердости от дозы для обеих сталей слабые в интервале доз 25-55 сна. При исследовании распределения величины микротвердости в зависимости от температуры по сечению образца Шарпи обнаружена слабая зависимость микротвердости от температуры в интервале температур облучения (90-133 град С)

Results of experimental investigations of stratification and combustion processes of HSGM in destructive prototype models

S22

Bezgodov, E.V.; Lavrenyuk, I.V.; Dorovskikh, I.A. (FGFUP #Left-Pointing Double Angle Quotation Mark#RFYaTs-VNIITF im. akadem. E.I. Zababakhina#Right-Pointing Double Angle Quotation Mark#, Snezhinsk (RU)), e-mail: e.v.bezgodov@vniitf.ru (and others)

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Rezultaty ehksperimental'nykh issledovaniy protsessov stratifikatsii i gorenija VPGS v razrushaemykh maketakh

p. 7-10

(RU)

4 figs.

Ministerstvo Obrazovaniya i Nauki Rossijskoj Federatsii, Moscow (RU); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU); Snezhinskij Fiziko-Tekhnicheskij Inst. NIYaU MIFI, Snezhinsk (RU)

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43 p.

HYDROGEN; STEAM; STEAM GENERATORS; REACTOR ACCIDENT SIMULATION; LEAKS; COMBUSTION; CONTAINMENT SHELLS; STRATIFICATION; GASES; VALIDATION; COMPUTER CODES; FLUID FLOW

The results of experiments aimed to investigation of the jet flows, ignition and combustion of hydrogen-containing steam gas mixtures (HSGM) in simulated containment in different scales from 1:10 to 1:200, are presented. The measurement procedures, experiment performance technologies and process historical data acquisition are worked out in the experiments. Leakage similar to the leakage of accident scenario #Left-Pointing Double Angle Quotation Mark#Small leak Dy 80 mm#Right-Pointing Double Angle Quotation Mark# is also simulated. Practically in all experiments HSGM stratification is observed. The results of experiments in light prototype models are supposed to use for validation of codes with lumped parameters, CFD-codes and codes with combustion models

Представлены результаты экспериментов, направленных на исследование струйных течений, воспламенения и режимов горения водородосодержащих парогазовых смесей (ВПГС) в макетах помещений защитной оболочки в разных масштабах от 1:10 до 1:200. В экспериментах отрабатывались методики измерения, технологии проведения экспериментов и получения данных о протекании процессов. Моделировалось также истечение, подобное истечению аварийного сценария #Left-Pointing Double Angle Quotation Mark#Малая течь Ду 80 мм#Right-Pointing Double Angle Quotation Mark#. Практически во всех экспериментах наблюдалась стратификация ВПГС. Результаты экспериментов в легких макетах предлагается использовать для валидации кодов с сосредоточенными параметрами, CFD-кодов и кодов с моделями горения

Calculational and theoretical justification of lumped molten salt reactor with sharing functions of power production and transfer

S21

Belonogov, M.N.; Simonenko, V.A.; Filonenko, E.M. (FGUP #Left-Pointing Double Angle Quotation Mark#RFYaTs-VNIITF im. akadem. E.I. Zababakhina#Right-Pointing Double Angle Quotation Mark#, Snezhinsk (RU)); Volkov, I.A. (FGUP #Left-Pointing Double Angle Quotation Mark#RFYaTs-VNIITF im. akadem. E.I. Zababakhina#Right-Pointing Double Angle Quotation Mark#, Snezhinsk (RU); FGAOU VPO #Left-Pointing Double Angle Quotation Mark#Ural'skij Federal'nyj Univ. imeni pervogo Prezidenta Rossii B.N. El'tsyna#Right-Pointing Double Angle Quotation Mark#, Ekaterinburg (RU)), e-mail: m.n.belonogov@vniitf.ru

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Raschetno-teoreticheskoe obosnovanie kontseptsii blochnogo rasplavnosolevogo reaktora s razdeleniem funktsij proizvodstva i peredachi ehnergii

p. 11-13

(RU)

9 refs., 2 figs.

Ministerstvo Obrazovaniya i Nauki Rossijskoj Federatsii, Moscow (RU); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU); Snezhinskij Fiziko-Tekhnicheskij Inst. NIYaU MIFI, Snezhinsk (RU)

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43 p.

MOLTEN SALT REACTORS; MOLTEN SALT FUELS; SODIUM FLUORIDES; POTASSIUM FLUORIDES; LITHIUM FLUORIDES; LITHIUM 7; URANIUM FLUORIDES; COMPUTER CODES; COMPUTERIZED SIMULATION

The concept on molten salt fixed-bed reactor is under consideration. Reactor is an assembly consisting from some blocks. Each block is a cylindrical vessel filled by fuel salt. In the vessel there are tubes through which salt-coolant is pumped over and in some tubes neutron flux control elements are mounted. Fuel salt is FLiNaK eutectics (11,5 NaF - 46,5 LiF - 42 KF, % mole) with UF₄ dissolved in it, enrichment on ⁷Li is up to 99,99 %. Salt-coolant is FLiNaK. The model of one block is constructed in the software pack ANSYS FLOTRAN, source file is prepared, it allows to start ANSYS in packet mode and to redesign task geometry and computational mesh using control parameters. The algorithm of control and protection system members movement is realized. Thermal and physical characteristics are conducting. Evaluation neutronic calculations of one block using software solution PRIZMA+RISK are carried out

Рассматривается концепция расплавносолевого реактора с неподвижным топливом. Реактор представляет собой установку, состоящую из нескольких блоков. Каждый блок - цилиндрический сосуд, заполненный топливной солью. Через него проходят трубки, по которым прокачивается соль-теплоноситель, а в некоторых из них установлены органы регулирования нейтронного потока. Топливная соль - эвтектика FLiNaK (11,5 NaF - 46,5 LiF - 42 KF, % моль) с растворенным в ней UF₄, обогащение по ⁷Li - до 99,99%. Соль-теплоноситель - FLiNaK. В программном пакете ANSYS FLOTRAN построена модель одного блока, подготовлен входной файл, позволяющий запускать ANSYS в пакетном режиме и с помощью управляющих параметров перестраивать геометрию задачи и расчетную сетку. Реализован алгоритм движения органов системы управления и защиты. Проводятся расчеты теплофизических характеристик. Проведены оценочные нейтронно-физические расчеты одного блока с помощью программного комплекса ПРИЗМА+РИСК

Advanced technologies and problems of thorium fuel cycle

S11

Bot'ko, E.N.; Kachan, S.M. (UVO #Left-Pointing Double Angle Quotation Mark#Belorusskij Natsional'nyj Tekhnicheskij Univ.#Right-Pointing Double Angle Quotation Mark#, Minsk (BY)), e-mail: tes_bntu@tut.by

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Perspektivnye tekhnologii i problemy torieвого топливного tsikla

p. 14-16

(RU)

4 refs., 1 fig.

Ministerstvo Obrazovaniya i Nauki Rossijskoj Federatsii, Moscow (RU); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU); Snezhinskij Fiziko-Tekhnicheskij Inst. NIYaU MIFI, Snezhinsk (RU)

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43 p.

THORIUM CYCLE; COMPARATIVE EVALUATIONS; WATER COOLED REACTORS; HEAVY WATER COOLED REACTORS; WATER MODERATED REACTORS; HEAVY WATER MODERATED REACTORS; MOLTEN SALT REACTORS; FAST REACTORS; ACCELERATOR-DRIVEN SUBCRITICAL SYSTEMS; SPENT FUELS; RADIOACTIVE WASTE MANAGEMENT; ISOTOPE RATIO

Various technologic solutions of thorium fuel cycle realization are analyzed: thorium use in light-water, heavy-water and fast reactors as well as in molten salt reactors and accelerator-driven subcritical reactor. Comparative analysis of safety and energy efficiency for uranium-plutonium and uranium-thorium fuel cycles is carried out. Composition and radiotoxicity dynamics of spent thorium fuel are under investigation. Specificity of spent fuel and thorium cycle high-level radioactive wastes management are under consideration

Анализируются различные технологические решения реализации ториевого топливного цикла: использование тория в легководных, тяжеловодных и быстрых реакторах, а также в реакторах на солевых расплавах и субкритическом реакторе с ускорительной системой. Проводится сравнительный анализ безопасности и энергоэффективности для уран-плутониевого и уран-ториевого топливных циклов. Исследуется состав и динамика радиотоксичности отработавшего ториевого топлива. Рассматривается специфика обращения отработавшим топливом и высокоактивными отходами ториевого топливного цикла

Specific character of radioactive wastes management during realization of closed U-Pu fuel cycle

S11;S12

Bot'ko, E.N. (UVO #Left-Pointing Double Angle Quotation Mark#Belorusskij Natsional'nyj Tekhnicheskij Univ.#Right-Pointing Double Angle Quotation Mark#, Minsk (BY)), e-mail: tes_bntu@tut.by

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Spetsifika obrashcheniya s radioaktivnymi otkhodami pri realizatsii zakrytogo U-Pu toplivnogo tsikla

p. 17-19

(RU)

10 refs., 1 fig.

Ministerstvo Obrazovaniya i Nauki Rossijskoj Federatsii, Moscow (RU); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU); Snezhinskij Fiziko-Tekhnicheskij Inst. NIYaU MIFI, Snezhinsk (RU)

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43 p.

NUCLEAR ENGINEERING; FAST REACTORS; FUEL CYCLE; NUCLEAR FUELS; SPENT FUELS; COMPARATIVE EVALUATIONS

Two scenarios of current nuclear power engineering development are under consideration: evolutionary (with the base on thermal reactors) and innovative (introduction of closed fuel cycle and fast reactors). Various variants of move towards closed fuel cycle as well as arising problems and prospects are under investigation. The analysis is given to the problems of radioactive waste handling in the case of closed U-Pu fuel cycle realization

Рассматриваются два сценарии развития современной ядерной энергетики: эволюционный (сохранение опоры на тепловые аппараты) и инновационный (внедрение замкнутого топливного цикла и быстрых реакторов). Исследуются различные варианты перехода к закрытому топливному циклу, а также возникающие при этом проблемы и перспективы. Анализируются вопросы обращения с радиоактивными отходами в условиях реализации закрытого U-Pu топливного цикла

Development and investigation of structure and properties of advanced neutron-shielding Al/B₄C-composites

S36

Gladkovskij, S.V.; Kuteneva, S.V.; Kamantsev, I.S. (Inst. Mashinovedeniya UrO RAN, Ekaterinburg (RU)); Kazeev, V.G.; Suvorov, E.A. (FGUP #Left-Pointing Double Angle Quotation Mark#RFYaTs-VNIITF im. akadem. E.I. Zababakhina#Right-Pointing Double Angle

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Razrabotka i issledovanie struktury i svoystv perspektivnykh nejtronozashchitnykh Al/B_4C-kompozitov

p. 20-21

(RU)

Ministerstvo Obrazovaniya i Nauki Rossijskoj Federatsii, Moscow (RU); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU); Snezhinskij Fiziko-Tekhnicheskij Inst. NIYaU MIFI, Snezhinsk (RU)

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43 p.

SPENT FUEL CASKS; ALUMINIUM; ALUMINIUM BASE ALLOYS; MAGNESIUM ALLOYS; BORON CARBIDES; SPECIFIC HEAT; THERMAL CONDUCTIVITY; TEMPERATURE RANGE 0400-1000 K; CRACK PROPAGATION; FRACTURE PROPERTIES; MICROSTRUCTURE

Using the hot rolling technique Al/B_4C-composite sheets intended to be the material of transport and storage cask jackets for transportation and storage of spent nuclear fuel are produced. Slaty structure of the composite is due to alternation of aluminium alloy AMg3 layers and layers of aluminium and boron carbide powder mixture. Structure, physico-mechanical and technological properties as well as specific characteristics of composite fracture microstructure are studied

Методом горячей прокатки получен листовой Al/B_4C-композит, предназначенный в качестве материала чехла транспортно-упаковочного контейнера для транспортирования и хранения отработавшего ядерного топлива. Слоистое строение композита обеспечивается за счет чередования слоев алюминиевого сплава AMg3 и прослоек из смеси порошков алюминия и карбида бора. Изучены структура, физико-механические и технологические свойства, а также особенности микростроения изломов композита

On the matter of choice of two-component system of nuclear power engineering S29

Dyrda, N.D.; Sheremet'eva, U.F.; Modestov, D.G.; Makeeva, I.R.; Simonenko, V.A. (FGUP #Left-Pointing Double Angle Quotation Mark#RFYaTs-VNIITF im. akadem. E.I. Zababakhina#Right-Pointing Double Angle Quotation Mark#, Snezhinsk (RU))

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K voprosu vybora dvukhkomponentnoj sistemy yadernoj ehnergetiki

p. 22-23

(RU)

2 refs.

Ministerstvo Obrazovaniya i Nauki Rossijskoj Federatsii, Moscow (RU); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU); Snezhinskij Fiziko-Tekhnicheskij Inst. NIYaU MIFI, Snezhinsk (RU)

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Nauchnaya sessiya NIYaU MIFI-2016. Sbornik nauchnykh trudov. Zasedanie tematicheskikh sekcij po napravleniyu Innovatsionnye yadernye tekhnologii. Tekhnologii zamykaniya yadernogo toplivnogo tsikla i vodorodnaya bezopasnost'

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Snezhinsk (RU)

SFTI NIYaU MIFI

2016

43 p.

NUCLEAR POWER; POWER GENERATION; FUEL CYCLE; THERMAL REACTORS; FAST REACTORS; SPENT FUELS; RADIOACTIVE WASTE PROCESSING; COMPUTER CODES

Different approaches to formation of nuclear power two-component system consisting from a set of thermal and fast reactors are under consideration. Such approach allows to use excess plutonium produced in fast reactors as a fuel for thermal reactors and to refuse from uranium enrichment technology, which is necessary for present-day thermal reactors fuel production thus additionally reduce the risk of proliferation

Рассматриваются различные подходы к формированию двухкомпонентной системы ядерной энергетики, состоящей из набора тепловых и быстрых реакторов. Такой подход позволяет эффективно использовать в качестве топлива тепловых реакторов избыточный плутоний, нарабатываемый в быстрых реакторах, и отказаться от технологии обогащения урана, необходимой для изготовления топлива современных тепловых реакторов, тем самым дополнительно снизив опасность распространения

Simulation of HSGM stratification process in experiments carried out by RFYaTs-VNIITF in hard large prototype models

S22;S08

Kupriyanets, T.A.; Lavrenyuk, I.V. (FGUP #Left-Pointing Double Angle Quotation Mark#RFYaTs-VNIITF im. akadem. E.I. Zababakhina#Right-Pointing Double Angle Quotation Mark#, Snezhinsk (RU)), e-mail: t.a.kupriyanets@vniitf.ru

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Nauchnaya sessiya NIYaU MIFI-2016

Modelirovanie protsessov stratifikatsii VPGS v ehksperimentakh, provodimyykh RFYaTs-VNIITF v prochnyykh bol'shikh maketakh

p. 24-25

(RU)

1 ref., 2 figs.

Ministerstvo Obrazovaniya i Nauki Rossijskoj Federatsii, Moscow (RU); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU); Snezhinskij Fiziko-Tekhnicheskij Inst. NIYaU MIFI, Snezhinsk (RU)

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43 p.

HYDROGEN; STEAM; GASES; MIXTURES; COMBUSTION; CONTAINMENT;
COMPUTERIZED SIMULATION; MOCKUP

Experimental investigations of HSGM stratification and combustion processes in conditions character to severe accidents

S21;S08

Lavrenyuk, I.V.; Kupriyanets, T.A.; Bezgodov, E.V.; Simonenko, V.A.; Ul'yanov, S.M.; Pasyukov, S.D.; Nikiforov, M.V.; Bakaev, V.P.; Fedyushkin, V.N.; Popov, I.A. (FGUP #Left-Pointing Double Angle Quotation Mark#RFYaTs-VNIITF im. akadem. E.I. Zababakhina#Right-Pointing Double Angle Quotation Mark#, Snezhinsk (RU)), e-mail: i.v.lavrenyuk@vniitf.ru

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Nauchnaya sessiya NIYaU MIFI-2016

Ehksperimental'nye issledovaniya protsessov stratifikatsii i gorenija VPGS v usloviyakh, kharakternykh dlya tyazhelykh avarij

p. 26-27

(RU)

2 figs.

Ministerstvo Obrazovaniya i Nauki Rossijskoj Federatsii, Moscow (RU); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU); Snezhinskij Fiziko-Tekhnicheskij Inst. NIYaU MIFI, Snezhinsk (RU)

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43 p.

HYDROGEN; GASES; STEAM; MIXTURES; COMBUSTION; DISTRIBUTION; FLAME PROPAGATION; REACTOR ACCIDENT SIMULATION

On the works on hydrogen safety of WWER NPP

S22;S08

Simonenko, V.A. (FGUP #Left-Pointing Double Angle Quotation Mark#RFYaTs-VNIITF im. akadem. E.I. Zababakhina#Right-Pointing Double Angle Quotation Mark#, Snezhinsk (RU)), e-mail: v.a.simonenko@vniitf.ru

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O rabotakh po vodorodnoj bezopasnosti AEhS s VVEhR

p. 28-30

(RU)

Ministerstvo Obrazovaniya i Nauki Rossijskoj Federatsii, Moscow (RU); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU); Snezhinskij Fiziko-Tekhnicheskij Inst. NIYaU MIFI, Snezhinsk (RU)

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43 p.

WATER COOLED REACTORS; REACTOR ACCIDENT SIMULATION; REACTOR SAFETY; COMPUTERIZED SIMULATION; FLAMMABILITY; MOCKUP; CONCENTRATION RATIO; FLAME PROPAGATION; TEMPERATURE RANGE 0273-0400 K; TEMPERATURE RANGE 0400-1000 K; PRESSURE RANGE KILO PA; COMPUTER CODES

Inflammability limits of hydrogen-containing steam air compositions (at hydrogen and water vapor concentration variation) are determined within the temperature range from 25 up to 200 deg C and pressure range from normal up to 0.5 MPa which are character to severe reactor accidents. Experimental data on combustion processes behaviour in rough conditions of connected rooms under NPP containment shell are obtained, the data can be used for current and advanced design codes validation. Design of WWER TOI is chosen as the base for refining model experiments set up

Определены значения концентрационных пределов воспламенения водородосодержащих паровоздушных составов (при изменении концентраций водорода и паров воды) в диапазоне температур от 25 до 200 град С и давлений от нормальных до 0,5 МПа, характерных для тяжелых аварий на АЭС с ВВЭР. Получены экспериментальные данные о протекании процессов горения в сложных условиях связанных помещений под

защитной оболочкой АЭС, которые можно использовать для валидации имеющихся и перспективных проектных кодов. Для конкретизации постановки модельных экспериментов в качестве базового выбран проект ВВЭР ТОИ

Organization of the #Left-Pointing Double Angle Quotation Mark#Arktika#Right-Pointing Double Angle Quotation Mark# code and possibilities of three-dimensional systems neutronic characteristics simulation S97

Trapeznikov, M.A.; Popov, I.S.; Sheremet'eva, U.F. (FGUP #Left-Pointing Double Angle Quotation Mark#RFYaTs-VNIITF im. akadem. E.I. Zababakhina#Right-Pointing Double Angle Quotation Mark#, Snezhinsk (RU)), e-mail: m.a.trapeznikov@vniitf.ru

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Organizatsiya koda #Left-Pointing Double Angle Quotation Mark#Arktika#Right-Pointing Double Angle Quotation Mark# i vozmozhnosti modelirovaniya nejtronno-fizicheskikh kharakteristik trekhmernykh sistem

p. 31

(RU)

Ministerstvo Obrazovaniya i Nauki Rossijskoj Federatsii, Moscow (RU); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU); Snezhinskij Fiziko-Tekhnicheskij Inst. NIYaU MIFI, Snezhinsk (RU)

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43 p.

COMPUTER CODES; NEUTRONS; ISOTOPES; RESONANCE; FUEL ELEMENTS; FAST REACTORS; HOMOGENIZATION METHODS

WWER-type reactor in uranium-plutonium fuel cycle

S21;S11

Sheremet'eva, U.F.; Dyrda, N.D.; Modestov, D.G.; Makeeva, I.R.; Simonenko, V.A. (FGUP #Left-Pointing Double Angle Quotation Mark#RFYaTs-VNIITF im. akadem. E.I. Zababakhina#Right-Pointing Double Angle Quotation Mark#, Snezhinsk (RU))

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Reaktor tipa VVEhR v uran-plutoniiovom toplivnom tsikle

p. 32-33

(RU)

3 refs.

Ministerstvo Obrazovaniya i Nauki Rossijskoj Federatsii, Moscow (RU); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU); Snezhinskij Fiziko-Tekhnicheskij Inst. NIYaU MIFI, Snezhinsk (RU)

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43 p.

WWER TYPE REACTORS; FUEL CYCLE; COMPUTERIZED SIMULATION; COMPUTER CODES; NEUTRONS; ISOTOPES; ISOTOPE RATIO; URANIUM; PLUTONIUM; FAST REACTORS

Calculational and experimental investigation of hydrogen-containing steam gas mixtures inflammation criteria

S42

Shul'ts, O.V.; Ushkov, A.V.; Fedyushkin, V.N.; Valeeva, O.V.; Davletchin, Yu.F.; Dorovskikh, I.A.; Lavrenyuk, I.V.; Nikul'shin, M.V. (FGUP #Left-Pointing Double Angle Quotation Mark#RFYaTs-VNIITF im. akadem. E.I. Zababakhina#Right-Pointing Double Angle Quotation Mark#, Snezhinsk (RU)), e-mail: o.v.shults@vniitf.ru

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Raschetno-ehksperimental'noe issledovanie kriteriev vosplamneniya vodorodosoderzhashchikh paro-gazovykh smesej

p. 37-38

(RU)

Ministerstvo Obrazovaniya i Nauki Rossijskoj Federatsii, Moscow (RU); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU); Snezhinskij Fiziko-Tekhnicheskij Inst. NIYaU MIFI, Snezhinsk (RU)

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43 p.

HYDROGEN; STEAM; GASES; INFLAMMATION; MOCKUP; CONCENTRATION RATIO; TEMPERATURE RANGE 0400-1000 K; TEMPERATURE RANGE 0273-0400 K; PRESSURE RANGE KILO PA; COMBUSTION

Development of prototype model of scintillation detector with high time resolution on the base of silicon photodiode

S46

Yudov, A.A.; Besov, S.S.; Ivanov, A.V. (FGUP #Left-Pointing Double Angle Quotation Mark#RFYaTs-VNIITF im. akadem. E.I. Zababakhina#Right-Pointing Double Angle Quotation Mark#, Snezhinsk (RU))

Scientific session of NRNU MEPHI-2016

Snezhinsk (RU)

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Nauchnaya sessiya NIYaU MIFI-2016

Razrabotka maketa ststintillyatsionnogo detektora s vysokim vremennym razresheniem na osnove kremnievogo fotodioda

p. 39-41

(RU)

2 refs., 2 figs.

Ministerstvo Obrazovaniya i Nauki Rossijskoj Federatsii, Moscow (RU); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU); Snezhinskij Fiziko-Tekhnicheskij Inst. NIYaU MIFI, Snezhinsk (RU)

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43 p.

PHOTODIODES; SOLID SCINTILLATION DETECTORS; TIME-OF-FLIGHT METHOD; NEUTRON SOURCES; GAMMA SOURCES; CALIFORNIUM; EFFICIENCY

Semiconductor micropixel avalanche photodiodes (MAPD) is an alternative to photoelectric multipliers. MAPD is a matrix consisting from paralleled separate avalanche photodiodes (micropixels) combined into single group which form macroscopic detector with considerable sensitive area. For studying time characteristics of MAPD detectors simple set up of source-detector time-of-flight measurements is suggested. Californium source is used as neutron and gamma source. It is shown that MAPD detector has good time resolution, MAPD has high and stable gain. Miniature size of MAPD and its work from low-voltage power supply allow to construct quite compact and self-supporting detectors for nuclear physics investigations

Полупроводниковые микропиксельные лавинные фотодиоды (МЛФД) - альтернатива фотоэлектронным умножителям. МЛФД представляет собой матрицу, состоящую из соединенных параллельно отдельных лавинных фотодиодов (микропикселей),

объединенных в единый массив, которые образуют макроскопический детектор с большой чувствительной областью. Для изучения временных характеристик детектора с МЛФД была предложена простая постановка времяпролетных измерений источник-детектор. В качестве источника нейтронов и гамма-квантов использовался калифорниевый источник. Показано, что детектор с МЛФД имеет хорошее временное разрешение, МЛФД обладает высоким и стабильным коэффициентом усиления. Миниатюрные размеры МЛФД и работа от источника питания низкого напряжения позволяет создавать достаточно компактные и автономные детекторы для ядерно-физических исследований

Tenth Petryanov's and First Fuks's reading. Book of abstracts

S54;S42

Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Gosudarstvennyj Nauchnyj Tsentr Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova, Moscow (RU)

Tenth Petryanov's and First Fuks's reading

Moscow (RU)

21-23 Apr 2015

Desyatye Petryanovskie i Pervye Fuksovskie chteniya

Desyatye Petryanovskie i Pervye Fuksovskie chteniya. Tezisy dokladov

INIS-RU--601

Moscow (RU)

АО NIFKhI im. L.Ya. Karpova

2015

175 p.

(RU)

LEADING ABSTRACT; MEETINGS; RADIOACTIVE AEROSOLS; AIR FILTERS; VENTILATION SYSTEMS; NANOFIBERS

The Tenth Petryanov's and the First Fuks's reading took place in Moscow in April, 21-23, 2015. In the collection there are the abstracts of the reports deal with the investigations in the field of natural and anthropogenic, including radioactive, aerosols; current state and future of filtration processes; development and upgrading of electroforming technology of nano- and microfibrous materials; methods of aerosols investigation and analysis

Десятые Петряновские и Первые Фуксовские чтения проходили в Москве 21-23 апреля 2015 г. В сборнике представлены тезисы докладов, посвященные исследованиям в области естественных и антропогенных, в том числе радиоактивных, аэрозолей; современному состоянию и перспективам развития процессов фильтрации; разработке и модернизации технологии электроформования нано- и микроволоконистых материалов; методам исследования и анализа аэрозолей

The role of aerosol investigations for ecological, nuclear and radiation safety

S61

Budyka, A.K. (Goskorporatsiya Rosatom, Moscow (RU); NIFKhI im. L.Ya. Karpova, Moscow (RU); Moskovskij Inzhenerno-Fizicheskij Inst., Moscow (RU))

Tenth Petryanov's and First Fuks's reading

Moscow (RU)

21-23 Apr 2015

Desyatye Petryanovskie i Pervye Fuksovskie chteniya
Rol' aehrozol'nykh issledovanij v obespechenii ehkologicheskoy, yadernoj i radiatsionnoj
bezopasnosti
p. 28-29
(RU)

Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Gosudarstvennyj
Nauchnyj Tsentr Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova,
Moscow (RU)

Tenth Petryanov's and First Fuks's reading. Book of abstracts

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Moscow (RU)

AO NIFKhI im. L.Ya. Karpova

2015

175 p.

RADIATION PROTECTION; NUCLEAR FACILITIES; ROSATOM; RISK ASSESSMENT;
RADIOACTIVE AEROSOLS; RADIATION MONITORING; RADIATION ACCIDENTS

Composite analytical filtering materials

S54

Filatov, I.Yu.; Kapustin, I.A.; Filatov, Yu.N. (AO Nauchno-Issledovatel'skij Inst. im. L.Ya. Karpova, Moscow (RU))

Tenth Petryanov's and First Fuks's reading

Moscow (RU)

21-23 Apr 2015

Desyatye Petryanovskie i Pervye Fuksovskie chteniya

Kompozitsionnye analiticheskie filtruyushchie materialy

p. 38-39

(RU)

Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Gosudarstvennyj
Nauchnyj Tsentr Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova,
Moscow (RU)

Tenth Petryanov's and First Fuks's reading. Book of abstracts

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Moscow (RU)

AO NIFKhI im. L.Ya. Karpova

2015

175 p.

RADIOACTIVE AEROSOLS; AEROSOL MONITORING; FILTERS; PARTICLE SIZE;
NANOFIBERS; NUCLEAR POWER PLANTS; VENTILATION SYSTEMS

New approach to simulating the process of aerosol nanoparticles precipitation on fibrous filters

S77

Kirsh, V.A.; Shabatin, A.V. (IFKhEh RAN, Moscow (RU))

Tenth Petryanov's and First Fuks's reading

Moscow (RU)

21-23 Apr 2015

Desyatye Petryanovskie i Pervye Fuksovskie chteniya

Novyj podkhod k modelirovaniyu protsessa osazhdeniya aehrozol'nykh nanochastits v voloknistykh fil'trakh

p. 39-42

(RU)

4 figs.

Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Gosudarstvennyj Nauchnyj Tsentr Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova, Moscow (RU)

Tenth Petryanov's and First Fuks's reading. Book of abstracts

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Moscow (RU)

AO NIFKhI im. L.Ya. Karpova

2015

175 p.

FILTERS; FIBERS; AEROSOLS; NANOPARTICLES; PRECIPITATION; COMPUTERIZED SIMULATION; DIFFUSION

Measurement of efficiency of filter elements after their mounting in NPP

S42

Grishin, A.G.; Yagodkin, I.V.; Posazhennikov, A.M. (AO GNTs RF - FEhI, Obninsk (RU))

Tenth Petryanov's and First Fuks's reading

Moscow (RU)

21-23 Apr 2015

Desyatye Petryanovskie i Pervye Fuksovskie chteniya

Izmerenie ehffektivnosti raboty fil'troval'nykh ehlementov posle montazha ikh v AEhS

p. 42

(RU)

Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Gosudarstvennyj Nauchnyj Tsentr Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova, Moscow (RU)

Tenth Petryanov's and First Fuks's reading. Book of abstracts

Desyatye Petryanovskie i Pervye Fuksovskie chteniya. Tezisy dokladov

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Moscow (RU)

AO NIFKhI im. L.Ya. Karpova

2015

175 p.

RADIOACTIVE AEROSOLS; AIR FILTERS; NUCLEAR POWER PLANTS; PARTICLE SIZE; EFFICIENCY; VENTILATION SYSTEMS

High-temperature aerosol and aerosol-adsorbing filters for NPP ventilation systems

S42

Kornienko, V.N. (Filial AO Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova, Obninsk (RU))

Tenth Petryanov's and First Fuks's reading

Moscow (RU)

21-23 Apr 2015

Desyatye Petryanovskie i Pervye Fuksovskie chteniya

Vysokotemperaturnye aehrozol'nye i aehrozol'no-sorbiruyushchie fil'try dlya system ventilyatsii
AEhS

p. 43

(RU)

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Peculiarities of aerosol filters control during operation

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Kapustin, I.A.; Filatov, Yu.N.; Filatov, I.Yu. (AO Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova, Moscow (RU))

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1 fig.

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New air recovery agents for individual and collective protection on the base of nanocrystal potassium superoxide on porous fiber matrix

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Gladysheva, T.V.; Plotnikov, M.Yu.; Dvoret'skij, S.I.; Gladyshev, N.F. (NOTs FGBOU VPO TGTU - OAO Korporatsiya Roskhimzashchita, Tambov (RU))

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Novye sredstva regeneratsii vozdukhha dlya individual'noj i kollektivnoj zashchity na osnove
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Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Gosudarstvennyj
Nauchnyj Tsentr Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova,
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OXYGEN; EQUIPMENT

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Rizin, A.I.; Fertman, D.E.; Tsoj, N.V. (OAO Spetsializirovannyj NII Priborostroeniya, Moscow (RU)); Chebyshov, S.B. (GK Rosatom, Gruppya Kompanij Atomehnergomash, Moscow (RU))

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4 figs.

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S12

Sukhoruchkin, A.K.; Stepenov, B.S.; Fedoseenkov, A.N. (NITs Kurchatovskij Inst., Moscow (RU))

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Opyt primeneniya ustanovok detektirovaniya aehrozolej UDA-1AB v usloviyakh provedeniya operatsij s TRO v khranilishche OYaT

p. 80-82

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3 refs., 4 figs.

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S12

Kulemin, V.V.; Lavrikov, V.A.; Rumer, I.A.; Kulyukhin, S.A. (Federal'noe Gosudarstvennoe Byudzhethnoe Uchrezhdenie Nauki Inst. Fizicheskoy Khimii i Ehletrokhimii im. A.N. Frumkina Rossijskoj Akademii Nauk, Moscow (RU)); Melikhov, I.V. (Khimicheskij Fakul'tet Moskovskogo Gosudarstvennogo Univ. im. M.V. Lomonosova, Moscow (RU))

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Iz vlechenie letuchikh soedinenij ^{137}Cs i ^{131}I iz vozdušnogo potoka

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(RU)

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Carbon-containing sorbents for extraction of radioactive iodine from vapor-gas-air media

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Yagodkin, I.V.; Posazhennikov, A.M. (AO GNTs RF - FEhI, Obninsk (RU)); Kulyukhin, S.A. (FGBUN Inst. Fizicheskoy Khimii i Ehlektrokhimii im. A.N. Frumkina Rossijskoj Akademii Nauk, Moscow (RU))

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Uglerodsoderzhashchie sorbenty dlya izvlecheniya radioaktivnogo joda iz paro-gazo-vozdushnykh sred

p. 83-84

(RU)

4 refs.

Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Gosudarstvennyj Nauchnyj Tsentr Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova, Moscow (RU)

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Purification of cover gas from heavy liquid metal coolant aerosols

S42

Posazhennikov, A.M.; Yagodkin, I.V.; Papovyants, A.K.; Grishin, A.G.; Isaev, A.Yu. (AO GNTs RF - FEhI, Obninsk (RU))

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Investigation of filtering properties of two-layer compositions of filter materials during radioactive aerosols sampling

S42

Kalinovskij, A.K. (Inst. Problem Bezopasnosti AEhS NAN Ukrainy, Chernobyl' (UA));

Ogorodnikov, B.I. (GNTs RF Fiziko-Khimicheskij Inst. im. L.Ya. Karpova, Moscow (RU))

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p. 85-86

(RU)

1 ref.

Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Gosudarstvennyj Nauchnyj Tsentr Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova, Moscow (RU)

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Isolation of nuclear material particles from filtered samples separated on Petryanov filter using alpha autoradiography method

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Lomakin, M.V.; Arkhipov, S.Yu.; Stebel'kov, V.A. (Nekommercheskoe Partnerstvo Lab. Analiza Mikrochastits, Moscow (RU))

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Vydelenie chastits yadernykh materialov iz fil'trovnykh prob, otobrannykh na FP, s ispol'zovaniem metoda al'fa-avtoradiografii

p. 87-88

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4 refs.

Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Gosudarstvennyj Nauchnyj Tsentr Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova, Moscow (RU)

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Use of filter material FPP in the works on monitoring natural and artificial radionuclides content in atmosphere and radionuclide contamination of the territory

S54

Katkova, M.N.; Polyanskaya, O.N.; Epifanov, A.O. (Federal'noe Gosudarstvennoe Byudzhethnoe Uchrezhdenie NPO Tajfun, Obninsk (RU))

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Ispol'zovanie fil'tryushchego materiala FPP v rabotakh po monitoringu sodержaniya radionuklidov estestvennogo i iskusstvennogo proiskhozhdeniya v atmosfere i radioaktivnogo zagryazneniya mestnosti

p. 95-98

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4 refs., 2 figs.

Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); Gosudarstvennyj Nauchnyj Tsentr Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova, Moscow (RU)

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Characteristics of aerosol and dust particles collected by FPA-70-0,12 inside the compartment 012/7 of #Left-Pointing Double Angle Quotation Mark#Shelter#Right-Pointing Double Angle Quotation Mark# object

S54

Vlasova, I.Eh. (Khimicheskij Fakul'tet MGU im. M.V. Lomonosova, Moscow (RU)); Shiryaev, A.A. (IFKhEh RAN, Moscow (RU)); Khimicheskij Fakul'tet MGU im. M.V. Lomonosova, Moscow (RU)); Ogorodnikov, B.I. (AO Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova, Moscow (RU)); Pakhnevich, A.V. (PIN RAN, Moscow (RU))

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Kharakteristiki aehrozol'nykh i pylevykh chastits, sobrannykh s primeneniem FPA-70-0,12 v pomeshchenii 012/7 ob'ekta #Left-Pointing Double Angle Quotation Mark#Ukrytie#Right-Pointing Double Angle Quotation Mark#

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(RU)

1 fig.

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Coagulation of particles with radioactive impurities: simulation with account of experimental data

S54

Zagajnov, V.A.; Lushnikov, A.A.; Maksimenko, V.V.; Kalashniakov, N.P.; Klyachin, N.A.; Tulumdzhiyan, M.N.; Dobychin, S.O. (NIYaU MIFI, Moscow (RU)); Agranovskij, I.E. (AO Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova, Moscow (RU)), e-mail: vzagajnov@yandex.ru

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Koagulyatsiya chastits s radioaktivnymi primesyami: modelirovanie s uchetom ehksperimental'nykh dannykh

p. 102-103

(RU)

2 refs., 1 fig.

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RADIOACTIVE AEROSOLS; VAPOR CONDENSATION; MATHEMATICAL MODELS; RADIOACTIVITY

Peculiarities of radioactive aerosols dispersity in under-reactor compartments of Chernobylsk NPP #Left-Pointing Double Angle Quotation Mark#Shelter#Right-Pointing Double Angle Quotation Mark# object

S54

Budyka, A.K. (AO Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova, Moscow (RU); Moskovskij Inzhenerno-Fizicheskij Inst., Moscow (RU)); Ogorodnikov, B.I. (AO Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova, Moscow (RU); Inst. Problem Bezopasnosti AEhS NAN Ukrainy, Chernobyl' (UA)); Khan, V.E. (Inst. Problem Bezopasnosti AEhS NAN Ukrainy, Chernobyl' (UA))

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Osobennosti dispersnosti radioaktivnykh aehrozolej v podreaktornykh pomeshcheniyakh ob"ekta #Left-Pointing Double Angle Quotation Mark#Ukrytie#Right-Pointing Double Angle Quotation Mark# ChAEhS

p. 105-108

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4 refs., 4 figs.

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Radioactive aerosols emission in 2014 from Chernobylsk NPP #Left-Pointing Double Angle Quotation Mark#Shelter#Right-Pointing Double Angle Quotation Mark# object**S54**

Khan, V.E.; Kalinovskij, A.K.; Krasnov, V.A. (Inst. Problem Bezopasnosti AEhS NAN Ukrainy, Chernobyl' (UA)); Ogorodnikov, B.I. (Inst. Problem Bezopasnosti AEhS NAN Ukrainy, Chernobyl' (UA); AO Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova, Moscow (RU))

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Vybrosty radioaktivnykh aehrozolej v 2014 g. iz ob"ekta #Left-Pointing Double Angle Quotation Mark#Ukrytie#Right-Pointing Double Angle Quotation Mark# Chernobyl'skoj AEhS

p. 108-109

(RU)

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RADIOACTIVE AEROSOLS; CHERNOBYLSK-4 REACTOR; FILTERS; VENTILATION SYSTEMS; SHELTERS; PARTICLE SIZE; RADIOACTIVITY; INDOOR AIR CONTAMINATION

Radioactive aerosols near lava conglomeration in compartment 012/15 of Chernobylsk NPP #Left-Pointing Double Angle Quotation Mark#Shelter#Right-Pointing Double Angle Quotation Mark# object in 2014

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Ogorodnikov, B.I. (AO Nauchno-Issledovatel'skij Fiziko-Khimicheskij Inst. im. L.Ya. Karpova, Moscow (RU); Inst. Problem Bezopasnosti AEhS NAN Ukrainy, Chernobyl' (UA)); Khan, V.E.; Koval'chuk, V.P. (Inst. Problem Bezopasnosti AEhS NAN Ukrainy, Chernobyl' (UA))

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(RU)

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Space-time distribution of nanoparticles in atmosphere of Siberia

S54

Arshinov, M.Yu.; Belan, B.D. (Inst. Optiki Atmosfery im. V.E. Zueva SO RAN, Tomsk (RU))

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Prostranstvenno-vremennoe raspredelenie nanochastits v atmosfere Sibiri

p. 117-118

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8 refs.

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Investigation of aerosol nanoparticles in electret filters

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Ivanov, V.V.; Efimov, A.A.; Volkov, I.A. (Moskovskij Fiziko-Tekhnicheskij Inst., Dolgoprudnyj (RU))

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Issledovanie aehrozol'nykh nanochastits v ehlektretnykh fil'trakh

p. 119-122

(RU)

7 refs., 2 figs.

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Forming of nanoparticles in free axisymmetric turbulent jet

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Gil'fanov, A.K.; Zaripov, Sh.Kh. (Kazanskij (Privolzhsckij) Federal'nyj Univ., Kazan' (RU)); Kokh, V. (Inst. Toksikologii i Eksperimental'noj Meditsiny, Gannover (DE))

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Rogozina, M.A.; Ekidin, A.A.; Zhukovskij, M.V.; Vasyanovich, M.E. (Inst. Promyshlennoj Ehkologii UrO RAN, Ekaterinburg (RU))

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Shiryayev, A.A. (IFKhEh RAN, Moscow (RU)); Khimicheskij Fakul'tet MGU im. M.V.

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Pripachkin, D.A.; Morgunova, T.S. (Federal'noe Gosudarstvennoe Byudzhethoe Uchrezhdenie

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The scientific program of the conference covers of all current problems in nuclear physics and its applications. The recent results of experimental investigations of atomic nucleus properties and nuclear reaction mechanisms are presented. The theoretical aspects of atomic nucleus, fundamental interactions and nuclear reactions are discussed. The particular attention is given to

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Научная программа конференции охватывает практически все направления современной ядерной физики. Представлены последние результаты экспериментальных исследований свойств атомного ядра и механизмов ядерных реакций. Обсуждаются теоретические аспекты структуры атомных ядер, фундаментальных взаимодействий и ядерных реакций. Особое внимание уделяется применению теории систем нескольких частиц в ядерной и атомной физике. Рассмотрено использование ядерно-физических экспериментальных методов в астрофизике, медицине и других областях ядерной техники

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Bunakov, V.E.; Krasnov, L.V.; Vlasnikov, A.K. (Saint-Petersburg State Univ., Saint-Petersburg (RU)), E-mail: vlasnic@list.ru

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Bunakov, V.E. (Saint-Petersburg State Univ., Saint-Petersburg (RU); Petersburg Nuclear Physics Inst., National Research Center Kurchatov Inst., Gatchina (RU)), E-mail: bunakov@VB13190.spb.edu

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Brudanin, V.B.; Klimenko, A.A.; Rukhadze, N.I.; Shitov, Yu.A. (Joint Inst. for Nuclear Research, Dubna (XJ)); Rukhadze, E.; #Latin Capital Letter S With Caron#tekl, I. (Inst. of Experimental and Applied Physics, CTU in Prague, Prague (CZ)), E-mail: rukhadze@jinr.ru

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Galanina, L.I.; Konykhova, I.A.; Lebedev, V.M.; Orlova, N.V.; Spassky, A.V.; Zelenskaya, N.S. (Lomonosov Moscow State Univ., Skobeltsyn Inst. of Nuclear Physics, Moscow (RU)), E-mail: wg2@anna19.sinp.msu.ru

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Duisebayev, A.; Duisebayev, B.A.; Zholdybayev, T.K.; Sadykov, B.M. (Inst. of Nuclear Physics, Almaty (KZ)); Ismailov, K.M. (Nazarbayev Univ., Astana (KZ)), E-mail: zholdybayev@inp.kz

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Duisebayev, A.; Duisebayev, B.A.; Zholdybayev, T.K.; Sadykov, B.M. (Inst. of Nuclear Physics, Almaty (KZ)); Ismailov, K.M. (Nazarbayev Univ., Astana (KZ)), E-mail: zholdybayev@inp.kz

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Konobeevski, E.; Mordovskoy, M.; Zuyev, S. (Inst. for Nuclear Research, Russian Academy of Sciences, Moscow (RU)); Lebedev, V.; Spassky, A. (Skobel'tsyn Inst. of Nuclear Physics, Moscow State Univ., Moscow (RU)), E-mail: konobeev@inr.ru

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Konobeevski, E.S.; Mordovskoy, M.V.; Zuyev, S.V.; Zheleznykh, I.M.; Gassanov, A.G. (Inst. for Nuclear Research, Russian Academy of Sciences, Moscow (RU)); Sadygov, Z.Y. (Joint Inst. of Nuclear Research, Dubna (XJ)), E-mail: mvmordovsk@mail.ru

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Burtebayev, N.; Burtebayeva, J.T.; Duisebayev, A. (Inst. of Nuclear Physics, Almaty (KZ)), E-mail: sbsakuta@mail.ru (and others)

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Burtebayev, N.; Baktybayev, M.; Kerimkulov, Zh. (Inst. of Nuclear Physics, Almaty (KZ)), E-mail: y.mukhamejanov@gmail.com (and others)

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Burtebayev, N.; Kerimkulov, Zh. (Inst. of Nuclear Physics, Almaty (KZ)); Demyanova, A.S.; Sakuta, S.B. (Kurchatov Inst., Moscow (RU)); Morzabayev, A.K.; Janseitov, D.M. (Eurasian National Univ., Astana (KZ)); Nassurlla, M.; Alimov, D.K.; Mukhamejanov, E.S.; Shakirov, A.K. (Al-Farabi Kazakh National Univ., Almaty (KZ)), E-mail: nburtebayev@yandex.ru

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Duisebayev, A.; Duisebayev, B.A.; Zholdybayev, T.K.; Sadykov, B.M. (Inst. of Nuclear Physics, Almaty (KZ)), E-mail: sadykovbm@inp.kz

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Berdnikov, A.Ya.; Ivanishchev, D.A. (Peter the Great Saint-Petersburg Polytechnic Univ., Saint-Petersburg (RU)); Kotov, D.O.; Riabov, Yu.G. (Peter the Great Saint-Petersburg Polytechnic Univ., Saint-Petersburg (RU)); NRC Kurchatov Inst., Petersburg Nuclear Physics Inst., Gatchina (RU)); Riabov, V.G.; Samsonov, V.M. (Peter the Great Saint-Petersburg Polytechnic Univ., Saint-Petersburg (RU)); NRC Kurchatov Inst., Petersburg Nuclear Physics Inst., Gatchina (RU); National Research Nuclear Univ. MEPhI, Moscow (RU)), E-mail: riabovvg@gmail.com

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Berdnikov, A.Ya.; Ivanishchev, D.A.; Safonov, A.S. (Peter the Great Saint-Petersburg Polytechnic Univ., Saint-Petersburg (RU)); Kotov, D.O.; Riabov, Yu.G. (Peter the Great Saint-Petersburg Polytechnic Univ., Saint-Petersburg (RU)); NRC Kurchatov Inst., Petersburg Nuclear Physics Inst., Gatchina (RU)); Riabov, V.G.; Samsonov, V.M. (Peter the Great Saint-Petersburg Polytechnic Univ., Saint-Petersburg (RU)); NRC Kurchatov Inst., Petersburg Nuclear Physics

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Mamatkulov, K.Z. (Joint Inst. for Nuclear Research, Dubna (XJ); A. Kodirii Jizzakh State Pedagogical Inst., Jizzakh (UZ)); Artemenkov, D.A.; Zarubin, P.I.; Zaitsev, A.A. (Joint Inst. for Nuclear Research, Dubna (XJ)); Bekmirzaev, R.N. (A. Kodirii Jizzakh State Pedagogical Inst., Jizzakh (UZ)), E-mail: kahramon@lhe.jinr.ru

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Burtebayev, N.; Zazulin, D.M.; Kerimkulov, Zh.K.; Baktybayev, M. (Inst. of Nuclear Physics of the Republic Kazakhstan, Almaty (KZ)); Alimov, D.K.; Mukhamejanov, Y.S.; Sairanbayev, D. (Inst. of Nuclear Physics of the Republic Kazakhstan, Almaty (KZ)); Al-Farabi Kazakh National Univ., Almaty (KZ)); Janseitov, D.M.; Nassurlla, M. (Inst. of Nuclear Physics of the Republic Kazakhstan, Almaty (KZ)); L.N. Gumilyov Eurasian National Univ., Astana (KZ)); Bahtibaev, A.N. (A. Yassavi IKTU, Turkestan (KZ)), E-mail: y.mukhamejanov@gmail.com

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Gikal, K.B.; Kozulin, E.M.; Bogachev, A.A. (Flerov Lab. of Nuclear Reactions, JINR, Dubna (XJ)), E-mail: kgikal@mail.ru (and others)

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Plavko, A.V. (Saint-Petersburg State Polytechnic Univ., Saint-Petersburg (RU)); Onegin, M.S. (Petersburg Nuclear Physics Inst., Gatchina (RU)); Kudriashov, V.I. (Saint-Petersburg State Univ., Saint-Petersburg (RU)), E-mail: kudr@comita.spb.ru

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Dubovichenko, S.B.; Dzhazairov-Kakhramanov, A.V. (V.G. Fessenkov Astrophysical Inst. NCSRT NSA, Almaty (KZ); Inst. of Nuclear Physics CAE MINT RK, Almaty (KZ)); Burtebayev, N.; Alimov, D.K. (Inst. of Nuclear Physics CAE MINT RK, Almaty (KZ)); Tkachenko, A.S. (V.G. Fessenkov Astrophysical Inst. NCSRT NSA, Almaty (KZ)), E-mail: dubovichenko@mail.ru, nburtebayev@yandex.ru, albert-j@yandex.ru

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Filikhin, I.N.; Vlahovic, B. (Nort Carolina Central Univ., Durham, NC (US)), E-mail: ifilikhin@nccu.edu

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Daurenbekov, D.H.; Kuterbekov, M.K.; Kalydulov, D. (L.N. Gumilyov National Univ., Astana
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Marinova, A.P.; Marinov, G.M. (Joint Inst. for Nuclear Research, DLNP, Dubna (XJ); Univ. of Sofia, Faculty of Chemistry and Pharmacy, Sofia (BG)); Dadakhanov, J.A. (Joint Inst. for Nuclear Research, DLNP, Dubna (XJ); Inst. of Nuclear Physics AS RUz, Ulugbek, Tashkent (UZ)); Happel, S. (Triskem International, Rue Maryse Bastie, Campus de Ker Lann, Bruz (FR)); Radchenko, V.I. (Johannes-Gutenberg Univ. Mainz, Mainz (DE)); Filosofov, D.V. (Joint Inst. for Nuclear Research, DLNP, Dubna (XJ)), E-mail: atanaskapavlovamarinova@gmail.com

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Morozov, V.A.; Morozova, N.V. (Joint Inst. for Nuclear Research, Dubna (XJ)), E-mail: vmorozov@nusun.jinr.ru

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Pop, O.M.; Stets, M.V.; Maslyuk, V.T. (Inst. of Electron Physics, National Academy of Sciences of Ukraine, Uzghorod (UA)), E-mail: oksana_pop@i.ua

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Egorov, V.G.; Ponomarev, D.V.; Rozova, I.E. (Joint Inst. for Nuclear Research, Dubna (XJ)), E-mail: egorov@jinr.ru (and others)

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Kuzenov, V.V. (Bauman Moscow State Technical Univ., Moscow (RU); Inst. for Problems in Mechanics of RAS, Moscow (RU)); Ryzhkov, S.V.; Shumaev, V.V. (Bauman Moscow State Technical Univ., Moscow (RU)), E-mail: shumaev@student.bmstu.ru

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Kuryndin, A.V.; Stroganov, A.A.; Kirkin, A.M.; Vernik, Yu.V.; Lyashko, I.A. (FBU NTTs YaRB, Moscow (RU))

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Marshalkin, V.E.; Povyshev, V.M. (RFYaTs-VNIEhF, Sarov (RU))

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Kulikov, K.N.; Ivanov, A.B.; Zaochinskiy, S.P.; Bogdanov, G.A.; Pospelova, A.V.; Sosnina, Yu.N.

(AO NIPTB Onega, Severodvinsk (RU)); Krasnoshchekov, A.N. (Otdelenie Guba Andreeva SZTs SevRAO, Filial FGUP RosRAO, Zaozersk (RU))

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Ignatov, A.A. (NITs Kurchatovskij Inst., Moscow (RU))

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FUEL CYCLE; MONTE CARLO METHOD

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Sinyavskij, V.V. (PAO RKK Ehnergiya imeni S.P. Koroleva, Korolev (RU))

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Gordeev, Eh.G.; Kaminskij, A.S.; Konyukhov, G.V.; Kuznetsov, V.S.; Pavshuk, V.A.; Pal'vanov, V.Ch.; Turbina, T.A.; Khmylev, A.N. (NITs Kurchatovskij Inst., Moscow (RU))

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Andrianov, D.I.; Popov, A.V.; Terekhov, D.N.; Shtonda, S.Yu. (GNTs FGUP Tsentra Keldysha, Moscow (RU))

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Tszin', M.; Chzhang, G.; Chzhou, T.; Tszou, S.; Chzhau, Ch. (Gruppa po Funktsional'noj Proektnoj Spetsifikatsii, Vedushchaya Lab. Nejtronno-Fizicheskikh Svojstv i Radiatsionnoj Bezopasnosti, Inst. Tekhnologii Bezopasnosti Yadernoj Ehnergetiki, Kitajskaya Akademiya Nauk, Khehfehj (CN))

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Dragunov, Yu.G.; Leshukov, A.Yu.; Strebkov, Yu.S. (AO #Left-Pointing Double Angle Quotation Mark#NIKIET#Right-Pointing Double Angle Quotation Mark#, Moscow (RU)) (and others)

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NIIEFA experience in applying NDT technologies for manufacturing of ITER components

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Belyaev, A.S.; Gur'eva, T.M.; Lyanzberg, D.V.; Panteleev, M.A.; Sokolov, I.V. (AO NIIEhFA, Sankt-Peterburg (RU))

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High heat flux tests of plasma facing components of the thermonuclear reactor ITER

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Pryanikov, V.V.; Kuznetsov, V.E.; Komarov, A.O. (AO NIIeHFA, Sankt-Peterburg (RU)) (and others)

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Calculations of energy and angle distributions of nuclear fusion products using improved formulae for differential cross sections

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Lithium capillary-pore systems as plasma facing material for fusion reactors

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Lyublinskij, I.E.; Vertkov, A.V. (AO Krasnaya Zvezda, Moscow (RU))

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Hazardous modes of liquid metal/molten salt heat transfer in a tokamak fusion reactor

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Genin, L.G. (NIU MEhI, Moscow (RU)); Sviridov, V.G. (NIU MEhI, Ob''edinennyj Inst. Vysokikh Temperatur RAN, Moscow (RU))

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Two-dimensional integrated hohlraum radiation hydrodynamic simulation of #Left-Pointing Double Angle Quotation Mark#coast low-foot#Right-Pointing Double Angle Quotation Mark# national ignition facility implosion

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Volkova, K.A.; Gnutov, A.S.; Golishnikov, D.N.; Dontsov, S.A.; Ul'yanov, A.S. (FGUP RFYaTs-VNIEhF, Sarov (RU))

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Barinov, M.A.; Mal'kin, A.G.; Pasharina, O.Yu.; Ul'yanov, A.S. (FGUP RFYaTs-VNIEhF, Sarov (RU))

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Bestuzhev, K.O.; Manzuk, M.V.; Avanesov, S.D.; Frolkin, V.I. (AO NIIEhFA, Sankt-Peterburg (RU))

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Zaurbekova, Zh.A.; Kul'sartov, T.V.; Gordienko, Yu.N.; Ponkratov, Yu.V.; Tazhibaeva, I.L.; Skakov, M.K. (Filial IAEh RGP NYaTs RK, Kurchatov (KZ))

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(Kalkhehmskij Tsentr po Termoyadernoj Ehnergii, Kalkhehm (GB))

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A small nuclear power plant with the ATGOR gas cooled reactor facility

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Romadova, E.L.; Kudinov, V.V.; Sleptsov, L.A.; Kobzev, P.V.; Kim, D.D.; Shebanin, V.V. (AO NIKIEhT, Moscow (RU))

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Study of VVER steam generator model operation in condensing mode under various parameters of emergency processes

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Morozov, A.V.; Shlepkina, A.S.; Kalyakin, D.S.; Soshkina, A.S. (AO GNTs RF-FEhI, Obninsk (RU))

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Zatirakha, A.V.; Shchukina, O.I.; Uzhel', A.S.; Smolenkov, A.D.; Shpigun, O.A. (MGU imeni M.V. Lomonosova, Moscow (RU)); Tataurova, O.G. (OOO NTTs BiASep, Moscow (RU))

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Development and justification of the MBIR reactor steam generator water chemistry

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Yurmanov, V.A.; Yurmanov, E.V.; Nikel', O.A. (AO NIKIEhT, Moscow (RU))

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Morozov, A.V.; Sakhigareev, A.R. (AO GNTs RF-FEhI, Obninsk (RU))

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Pipchenko, G.R.; Polikarpova, A.M.; Kavun, O.Yu.; Lankin, M.Yu. (FBU NTTs YaRB, Moscow (RU))

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Kozlov, F.A.; Sorokin, A.P. (AO GNTs RF - Fiziko-Ehnergeticheskij Inst. imeni A.I. Lejpunskogo, Obninsk (RU)); Konovalov, M.A. (AO GNTs RF - Fiziko-Ehnergeticheskij Inst. imeni A.I. Lejpunskogo, Obninsk (RU)); NIYaU MIFI, Moscow (RU))

Sistemy ochistki natriya AEhS s reaktorami na bystrykh nejtronakh (retrospektivno-perspektivnyj vzglyad)

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NUCLEAR POWER; FAST REACTORS; COOLANT LOOPS; PURIFICATION; COLD TRAPS; IMPURITIES; FILTERS; CORROSION PRODUCTS; DIFFUSION

Authors analyze the sodium purification systems for NPP with fast reactors. Based on the results of this work, recommendations for the directions of work toward improvement of characteristics of sodium purification systems for perspective NPPs with sodium coolant are given in the paper

Авторы провели анализ системы очистки натрия АЭС с реакторами на быстрых нейтронах. На основании результатов исследований рекомендованы основные направления работы по улучшению характеристик систем очистки перспективных АЭС с натриевым теплоносителем

Investigation of ecological constraints influence on competitiveness of nuclear power plants

S29

Marchenko, O.V.; Solomin, S.V. (Inst. Sistem Ehnergetiki imeni L.A. Melent'eva (ISEhM) SO RAN, Irkutsk (RU))

Issledovanie vliyaniya ehkologicheskikh ogranichenij na konkurentosposobnost' atomnykh ehlektrostantsij

p. 20-30

(RU)

18 refs., 4 figs., 2 tabs.

The main objective of this study is the comparison of economic efficiency of nuclear and non-nuclear power plants in the short and long-term (up to 2050). Authors give the perspective and an updated forecast of the development of nuclear energy in Russia and in the world against the backdrop of global energy development in general

Цель работы - сравнение экономической эффективности ядерных и неядерных электростанций на краткосрочную и долгосрочную (до 2050 г.) перспективу и уточнение прогнозов развития ядерной энергетики России и мира на фоне развития мировой энергетики в целом

Fast neutron heterogeneous scintillation detector with high discrimination of gamma background

S46

Chernukhin, Yu.I. (Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU)); Yudov, A.A.; Strel'tsov, S.I. (FGUP RFYaTs-VNIITF imeni Akademika E.I. Zababakhina, Snezhinsk (RU))

Geterogennyj stsintillyatsionnyj detektor bystrykh nejtronov s vysokoj diskriminatsiej gamma-fona

p. 31-39

(RU)

11 refs., 4 figs., 3 tabs.

Izvestiya Vysshikh Uchebnykh Zavedenij. Yadernaya Ehnergetika

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MEASURING INSTRUMENTS; SCINTILLATION COUNTERS; SENSORS; MEASURING METHODS; RADIATION DETECTORS; NEUTRON DETECTORS; RADIATION MONITORING

Authors examine the potential improvement of characteristics of multi-layer neutron detector with fiber-optic sensors based on lithium-silicate glass by means of polyethylene layers' replacement with hydrogen-containing layers of scintillating plastic

В статье рассматривается возможность улучшения характеристик многослойного детектора нейтронов с оптоволоконными сенсорами на основе литий-силикатного стекла за счет замены в нем слоев из полиэтилена на водородосодержащие слои из сцинтиллирующей пластмассы

Photometry of ionizing radiations

S61

Khryachkov, V.A.; Zhuravlev, B.V.; Talalaev, V.A. (AO GNTs RF - Fiziko-Ehnergeticheskij Inst. imeni A.I. Lejpunskogo, Obninsk (RU))

Fotometriya ioniziruyushchikh izluchenij

p. 40-48

(RU)

6 refs., 4 figs., 1 tab.

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RADIATION PROTECTION; DOSIMETRY; X-RAY DETECTION; RADIATION DETECTORS; IONIZING RADIATIONS; SENSORS; REMOTE SENSING

The prospect of designing a system of remote monitoring of radiation environment in a given point has been investigated and realized. The systems based on an ability of different mediums to irradiate optical photons under influence of ionizing radiation

В работе исследована и реализована возможность создания системы дистанционного контроля радиационной обстановки в заданных точках пространства. Система основана на способности некоторых сред под воздействием ионизирующего излучения испускать фотоны видимого света

Inner fuel element corrosion of steels CHS68 and EK164 at fast power reactor on basis of uranium dioxide

S21;S36

Kinev, E.A.; Shikhalev, V.S.; Barybin, A.V. (AO IRM, Zarechnyj (RU))

Vnutritvehl'naya korroziya stalej EhK-164 i ChS-68 bystrogo ehnergeticheskogo reaktora na osnove dioksida urana

p. 49-55

(RU)

5 refs., 4 figs., 1 tab.

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NUCLEAR POWER; FAST REACTORS; BELOYARSK-3 REACTOR; FUEL ELEMENTS; CLADDING; CORROSION PROTECTION; CHROMIUM-NICKEL STEELS; AUSTENITIC STEELS

Inner fuel element corrosion of steels CHS68 and EK164 at fast power reactor on the basis of uranium dioxide has been investigated in the paper. The gamma-scanning, electrical-potential scanning and optical metallography methods have been applied during the experiments. The comparative analysis of the inner fuel rod corrosion for steel CHS68 and steel EK164 have been carry out along core region height

Авторами изучена внутритвэльная коррозия сталей ЭК-164 и ЧС-68 быстрого энергетического реактора на основе диоксида урана. Используются методы гамма-сканирования, электропотенциальной резистометрии, оптической металлографии. Проведен сравнительный анализ внутритвэльной коррозии оболочек из сталей ЭК-164 и ЧС-68 по высоте активной зоны

Nodal S_N-method for HEX-Z-geometry

S21;S97

Bereznev, V.P. (Inst. Problem Bezopasnogo Razvitiya Atomnoj Ehnergetiki RAN, Moscow (RU))

Nodal'nyj S_N-metod v HEX-Z-geometrii

p. 56-62

(RU)

4 refs., 3 figs., 2 tabs.

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NUCLEAR POWER; FAST REACTORS; BN-800 REACTOR; FUEL ASSEMBLIES;
MATHEMATICS; GEOMETRY; HEXAGONAL CONFIGURATION

The authors have developed and implemented a S_N-nodal method for neutron and physical calculations. The calculation of 2D- and 3D-models of KNK-II reactor demonstrates the increased accuracy order and accurately correct distribution of neutron field by using the nodal S_N-method

Авторами разработан и реализован нодальный S_N-метод для нейтронно-физических расчетов. Проведенный расчет 2D- и 3D-модели реактора KNK-II демонстрирует повышенный порядок точности и качественно правильное распределение поля нейтронов при использовании нодального S_N-метода

Experimental researches of local hydrodynamics of coolant in PWR FA-Kvadrat using different mixing grids

S21

Dmitriev, S.M.; Borodin, S.S.; Varentsov, A.V.; Legchanov, M.A.; Sorokin, V.D.; Khrobostov, A.E. (Nizhegorodskij Gosudarstvennyj Tekhnicheskij Univ. imeni R.E. Alekseeva, Nizhnij Novgorod (RU))

Ehksperimental'nye issledovaniya lokal'noj gidrodinamiki teplonositelya v TVS-Kvadrat reaktora PWR pri postanovke peremeshivayushchikh distantsioniruyushchikh reshetok s razlichnymi tipami deflektorov

p. 63-72

(RU)

8 refs., 7 figs.

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NUCLEAR POWER; PWR TYPE REACTORS; FUEL ASSEMBLIES; COOLANTS;
HYDRODYNAMICS; REACTOR CORES; GRIDS

The results of experimental researches of local hydrodynamic characteristics of the coolant flow in the fuel assemblies PWR reactors using different mixing grids are presented in the paper

В работе приведены результаты экспериментальных исследований локальных гидродинамических характеристик потока теплоносителя в тепловыделяющих сборках реакторов PWR при использовании различных перемешивающих дистанционирующих решеток

Experimental justification of choice operating channel imitator for hydrodynamic reactor model

S21

Avdeev, E.F.; Chusov, I.A. (Obninskij Inst. Atomnoj Ehnergetiki (IAEh NIYaU MIFI), Obninsk (RU)); Levchenko, Yu.D.; Yur'ev, Yu.S. (AO GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU))

Ehksperimental'noe obosnovanie vybora imitatora rabocheho kanala gidrodinamicheskoy modeli reaktora

p. 73-84

(RU)

8 refs., 6 figs., 3 tabs.

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NUCLEAR POWER; NUCLEAR FACILITIES; PRESSURE DROP; REACTOR CORES; FUEL ASSEMBLIES; SIMULATION; REYNOLDS NUMBER

The paper presents the methodology of experimental study for hydrodynamic characteristics of imitators channel model with two-loop reactor facility

В работе описана методика экспериментального обоснования гидродинамических характеристик имитаторов рабочих каналов модели реакторной установки с двухпетлевой компоновкой

Experimental investigation of sodium boiling heat exchange in fuel subassembly mockup for perspective fast reactor safety justification

S21

Khafizov, R.R.; Poplavskij, V.M.; Rachkov, V.I.; Sorokin, A.P.; Trufanov, A.A.; Ashurko, Yu.M.; Volkov, A.V.; Ivanov, E.F.; Privezentsev, V.V. (AO GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU))

Ehksperimental'nye issledovaniya teploobmena pri kipenii natriya v modeli TVS v obosnovanie bezopasnosti perspektivnogo reaktora na bystrykh nejtronakh

p. 85-96

(RU)

16 refs., 4 figs.

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REACTORS; FAST REACTORS; BN-350 REACTOR; FUEL ASSEMBLIES; TWO-PHASE FLOW; COOLANTS; LIQUID METALS; SODIUM

Results of experimental investigation of sodium boiling heat exchange in fuel subassembly mockup for perspective fast reactor safety justification are presented in the paper

В работе представлены результаты экспериментальных исследований теплообмена при кипении натрия в модели ТВС в обоснование безопасности перспективного реактора на быстрых нейтронах

Fluid flow and heat transfer in fuel rods assembly with modified spacer grids **S21**

Krapivtsev, V.G.; Markov, P.V.; Solonin, V.I. (Moskovskij Gosudarstvennyj Tekhnicheskij Univ. imeni N.Eh. Baumana, Moscow (RU))

Techenie i teploperenos v puchkakh sterzhnevnykh tvehlov vodookhlazhdaemykh reaktorov s modifitsirovannymi sotovymi distantsioniruyushchimi reshetkami

p. 97-105

(RU)

9 refs., 5 figs., 1 tab.

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NUCLEAR POWER; WWER TYPE REACTORS; REACTOR CORES; FUEL ASSEMBLIES;
HEAT TRANSFER; PRESSURE DROP; GRIDS; REYNOLDS NUMBER

The analysis of the intensification of coolant's mixing by modified spacer grids with different boundary conditions has been presented in the paper. The conclusion about the possibility of application of modified spacer grids to decrease temperature non-uniformity in coolant flow due to generation of secondary flow has also been done

В работе выполнен анализ интенсификации перемешивания теплоносителя за решетками методом теплового следа при различных граничных условиях. Сделан вывод о возможности и эффективности применения модифицированных сотовых дистанционирующих решеток для выравнивания температурных неоднородностей в потоке теплоносителя за счет создания направленного конвективного переноса

Researches of hydrodynamic characteristics of coolant in VVER-1000 reactor mixed core of atomic power station #Left-Pointing Double Angle Quotation Mark#Temelin#Right-Pointing Double Angle Quotation Mark# **S21**

Dmitriev, S.M.; Doronkov, D.V.; Polozkova, E.N.; Pronin, A.N.; Sorokin, V.D.; Khrobostov, A.E. (Nizhegorodskij Gosudarstvennyj Tekhnicheskij Univ. imeni R.E. Alekseeva, Nizhnij Novgorod (RU))

Ehksperimental'nye issledovaniya gidrodinamicheskikh kharakteristik potoka teplonosatelya v smeshannoj aktivnoj zone reaktora VVEhR-1000 AEhS #Left-Pointing Double Angle Quotation Mark#Temelin#Right-Pointing Double Angle Quotation Mark#

p. 106-114

(RU)

5 refs., 8 figs.

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WWER TYPE REACTORS; TEMELIN-1 REACTOR; REACTOR CORES; FUEL ASSEMBLIES; COOLANTS; LIQUID FLOW; HYDRODYNAMICS

Authors present the results of researches of interassembly interaction of coolant flow in VVER-1000 core of NPP #Left-Pointing Double Angle Quotation Mark#Temelin#Right-Pointing Double Angle Quotation Mark#. Study of coolant flow process in fuel element bundle FA has been carried out on an aerodynamic experimental rig

Представлены результаты исследований межкассетного взаимодействия теплоносителя в активной зоне реактора ВВЭР-1000 АЭС #Left-Pointing Double Angle Quotation Mark#Temelin#Right-Pointing Double Angle Quotation Mark#. Изучение процессов течения теплоносителя в твэльном пучке тепловыделяющей сборки альтернативной конструкции (ТВСА) проводилось на аэродинамическом экспериментальном стенде

Parametric analysis of space nuclear power plants in thermodynamic design variables

S21;S97

Yuferov, A.G.; Linnik, V.A.; Nikolaev, M.A. (AO GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU))

Parametricheskij analiz kosmicheskikh yadernykh ehnergeticheskikh ustanovok v termodinamicheskikh proektnykh peremennykh

p. 115-123

(RU)

9 refs.

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NUCLEAR FACILITIES; SPACE POWER REACTORS; HEAT EXCHANGERS; PARAMETRIC ANALYSIS; MATHEMATICAL MODELS; SENSITIVITY ANALYSIS

The purpose of the work is to analyze the influence of the efficiency and the temperatures on a specific mass of space power plant and a specific area of cooler-radiator with taking into account the heat losses and mass subsystems

Целью работы является анализ влияния КПД и среднеэнтропийных температур на удельную массу космической энергоустановки и удельную площадь холодильника-излучателя с учетом тепловых потерь и массовых коэффициентов подсистем

Gas-cooled thorium reactor with fuel block of the unified design

S21

Shamanin, I.V.; Bedenko, S.V.; Chertkov, Yu.B. (Natsional'nyj Issledovatel'skij Tomskij Politekhnikeskij Univ., Tomsk (RU)); Gubajdulin, I.M. (RFYaTs-VNII Ekhspperimental'noj

Fiziki, Sarov (RU)

Gazookhlazhdaemyj yadernyj reaktor s torievym toplivom na osnove toplivnogo bloka unifitsirovannoj konstruksii

p. 124-134

(RU)

17 refs., 5 figs., 3 tabs.

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NUCLEAR POWER; BN-800 REACTOR; GCFR TYPE REACTORS; NEUTRON FLUX; REACTIVITY; FUEL ASSEMBLIES

The aim of the work is to develop the concept of a low-power secure thorium nuclear power plants based on fuel block of unified design. The analysis of information about application of thorium as fuel in reactor systems and its perspective use in the future is presented in the work

Целью работы является разработка концепции ториевой маломощной безопасной ядерной энергетической установки на основе топливного блока унифицированной конструкции. Авторами проведен анализ информационных материалов об использовании тория как топливного элемента в реакторных установках нового поколения и его дальнейших перспективах

Small power lead fast reactor for purposes of education

S21

Samokhin, D.S.; Khorasanov, G.L.; Terekhova, A.M.; Kuz'michev, S.A. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)); Tormyshev, I.V.; Zemskov, E.A.; Gostev, A.L. (AO GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU))

Bystryj svintsovyj reaktor maloj moshchnosti dlya uchebnykh tselej

p. 135-143

(RU)

10 refs., 4 figs., 2 tabs.

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POWER REACTORS; FAST REACTORS; THERMAL REACTORS; COOLANTS; REACTIVITY; NEUTRON SPECTRA; MONTE CARLO METHOD

Authors present a lead fast neutron reactor for the purpose of carrying out researches, teaching students and postgraduate students to operate innovative lead fast reactors, and training specialists for nuclear scientific centers and nuclear power plants

Рассматривается возможность создания реактора на быстрых нейтронах в целях проведения исследований, обучения студентов и аспирантов навыкам обращения с инновационными быстрыми реакторами, подготовки специалистов для атомных научных центров и атомных электростанций

Degenerate optimization problems of economics and power engineering

S29

Klimenko, A.V. (OF Inst. Sistemno-Ehkonomicheskikh Issledovaniy imeni Ya.V. Sheveleva, Ehlektrosta' (RU); NIYaU MIFI, Moscow (RU))

Vyrozhdennye zadachi optimizatsii ehkonomiki i ehnergetiki

p. 144-154

(RU)

5 refs., 7 figs.

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ECONOMICS; ECONOMIC ANALYSIS; INTEREST RATE; NUCLEAR ENERGY;
PARAMETRIC ANALYSIS; LINEAR PROGRAMMING

Optimization of large economic and power engineering systems leads to degenerate solutions of high dimension. This is a very strong mathematical complication. However, the author emphasized that it allows to consider future development of the power industry based on simultaneous use of nuclear power plants (NPPs) together with coal- and gas power plants, or solely on NPPs

Автор представляет возможные пути оптимизации больших систем экономики и энергетики, приводящие к вырожденным решениям большой размерности, позволяющим рассматривать будущее развитие энергетики как на совместной работе ядерных энергетических установок (ЯЭУ), энергетических установок на угле, энергетических установок на газе, так и только на ЯЭУ

Basic strategies of public protection in a nuclear power plant beyond-design basis accident

S22

Kut'kov, V.A. (NITs Kurchatovskij Inst., Moscow (RU)); Tkachenko, V.V.; Saakyan, S.P. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU))

Osnovy strategii zashchity naseleniya v sluchae zaproektnoj avarii na atomnoj stantsii

p. 5-14

(RU)

18 refs., 2 tabs.

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(no.4)

NUCLEAR POWER PLANTS; REACTOR SAFETY; RADIATION ACCIDENTS;
RADIATION PROTECTION; RADIATION HAZARDS; ACCIDENT MANAGEMENT

Basic strategies of public protection in a nuclear power plant beyond-design basis accident have been presented and discussed by the authors

Авторами подробно рассмотрены и проанализированы основы стратегии защиты населения в случае запроектной аварии на атомной станции

Improving the energy efficiency of NPP

S29

Shcheklein, S.E.; Tashlykov, O.L.; Dubinin, A.M. (Ural'skij Federal'nyj Univ., Ekaterinburg (RU))

Povyshenie ehnergoehffektivnosti AEhS

p. 15-25

(RU)

8 refs., 7 figs., 2 tabs.

Izvestiya Vysshikh Uchebnykh Zavedenij. Yadernaya Ehnergetika

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(no.4)

ECONOMICS; ECONOMIC POLICY; ENERGY EFFICIENCY; COST; ENERGY ACCOUNTING; HEAT RATE

The aim of this work is to analyze and estimate potential ways to improve the energy efficiency of nuclear power. The article describes the currently used and advanced thermodynamic cycles of nuclear power plants. The ways to improve the working fluid parameters in the NPP steam turbine circuit are given

Целью работы является анализ и оценка потенциальных путей повышения энергоэффективности атомной энергетики. Рассмотрены используемые и перспективные термодинамические циклы атомных электростанций в энергетике. Приведены пути повышения параметров рабочего тела в паротурбинном контуре АЭС

The study of ultrasonic reflex-radar waveguide coolant level gage for a nuclear reactor

S22

Mel'nikov, V.I.; Teplyashin, I.A.; Ivanov, V.V. (Nizhegorodskij Gosudarstvennyj Tekhnicheskij Univ. imeni R.E. Alekseeva, Nizhnij Novgorod (RU))

Issledovanie ul'trazvukovogo refleks-radar'nogo volnovodnogo urovnemera teplonosatelya yadernoj ehnergeticheskoy ustanovki

p. 26-35

(RU)

5 refs., 8 figs.

Izvestiya Vysshikh Uchebnykh Zavedenij. Yadernaya Ehnergetika

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(no.4)

NUCLEAR POWER; NUCLEAR POWER PLANTS; COOLANTS; ULTRASONIC TESTING; WATER; ACOUSTIC MEASUREMENTS

The results of the experimental study of ultrasonic level gage on the basis of reflex-radar principles have been discussed in the paper. It has been shown that the device operated steadily at high temperatures and pressure up to 18 MPa in the water coolant

В работе рассмотрены результаты экспериментального исследования работы ультразвукового рефлекс-радарного волноводного уровнемера в водяном теплоносителе высоких параметров при давлении до 18 МПа и температуре до 350 град С

The optimization of radiation protection composition

S61

Tashlykov, O.L.; Shcheklein, S.E.; Luk'yanenko, V.Yu.; Mikhajlova, A.F. (Ural'skij Federal'nyj Univ., Ekaterinburg (RU)); Russkikh, I.M.; Seleznev, E.N.; Kozlov, A.V. (OAO Inst. Reaktornykh Materialov, Zarechnyj (RU))

Optimizatsiya sostava radiatsionnoj zashchity

p. 36-42

(RU)

6 refs., 3 figs., 1 tab.

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(no.4)

RADIATION PROTECTION; RADIATION MONITORING; RADIATION DOSES; ALARA; DOSE RATES; DOSIMETRY; GAMMA RADIATION

The aim of this work is to develop an algorithm of design of homogeneous composition of the radiation protective materials (RPM) for the optimization of radiation protection. The homogeneous RPM such as Abris have been used for the investigation, the production technology of which allows to obtain the desired concentration of fillers

Целью работы является разработка алгоритма проектирования состава гомогенных радиационно-защитных материалов (РЗМ) для оптимизации радиационной защиты. Для проведения исследований использовались гомогенные РЗМ типа Абрис, технология производства которых позволяет получать требуемые концентрации наполнителей

The temperature monitoring during the reactor core material irradiation by analyzing the structure of graphite-like boron nitride

S22

Isaev, E.I.; Stepanov, V.A. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)); Stepanov, P.A. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)); Obninskoe Nauchno-Proizvodstvennoe Predpriyatie Tekhnologiya imeni A.G. Romashina, Obninsk (RU))

Monitoring temperatury vnutrikanal'nogo reaktornogo oblucheniya materialov s pomoshch'yu analiza struktury grafitopodobnogo nitrída bora

p. 43-52

(RU)

10 refs., 5 figs.

Izvestiya Vysshikh Uchebnykh Zavedenij. Yadernaya Ehnergetika

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(2015)

(no.4)

NUCLEAR POWER; REACTORS; REACTOR CORES; IRRADIATION; FUEL ASSEMBLIES; BORON NITRIDES

Authors have shown that the reactor temperature determination is possible, using a single analysis of the structure of irradiated boron nitride ceramic without time-consuming stage of the post-radiation annealing research

В работе показана возможность определения температуры внутриканального облучения с помощью однократного анализа структуры облученной керамики из нитрида бора без трудоемкой стадии исследований, включающей пострадиационные отжиги

Radiation-induced nanostructuring of the amorphous alloy

S36

Antoshina, I.A.; Visheratin, R.K.; Stepanov, V.A. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU)); Elmanov, G.N. (Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU))

Radiatsionno-indutsirovannoe nanostrukturirovanie amorfnoego splava

p. 53-60

(RU)

7 refs., 4 figs.

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MATERIALS; ALLOYS; METALLIC GLASSES; NANOSTRUCTURES; AMORPHOUS STATE; IONIZATION; DEFORMATION; CORROSION RESISTANCE; SWELLING

Structural changes in metallic glasses $\text{CoFe}_{30}\text{Si}_{20}\text{Mn}_{30}\text{B}_{15}$, after irradiation by Ar^{+} ions with energy of 30 keV at 100-300 #Degree Celsius# and post-radiation annealing to a temperature 600 #Degree Celsius#, have been investigated by methods of the differential scanning calorimetry, atomic probe microscopy and electron microscopy

Методами дифференциальной сканирующей калориметрии, атомно-силовой и электронной микроскопии в работе исследованы изменения структуры в металлических стеклах $\text{CoFe}_{30}\text{Si}_{20}\text{Mn}_{30}\text{B}_{15}$ после облучения ионами Ar^{+} с энергией 30 кэВ при температурах 100-300 #Degree Celsius# и пострадиационных отжигов до температуры 600 #Degree Celsius#

Application of a neural network to predict the FAC rate of NPP equipment

S22

Vishnyakova, A.D.; Gulina, O.M.; Sal'nikov, N.L. (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU))

Vozmozhnost' primeneniya apparata nejronnykh setej dlya prognozirovaniya ehrozionno-korrozionnogo iznosa oborudovaniya AEhS

p. 61-71

(RU)

5 refs., 8 figs., 1 tab.

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(no.4)

NUCLEAR POWER PLANTS; REACTOR SAFETY; PIPELINES; CORROSION RESISTANCE; LEAKS; EROSION CONTROL; WEAR RESISTANCE; NEURAL NETWORKS

The paper deals with development and education of optimal neural network to predict the flow-accelerated corrosion rate at nuclear power plant piping

В работе предлагается подход к созданию и обучению оптимальной нейронной сети для задачи прогнозирования интенсивности эрозионно-коррозионного износа в трубопроводах атомных станций

The research of feasible temperature modes in the ampoule channel with natural circulation

S22

Osipova, T.A. (AO GNTs NIIAR, Dimitrovgrad (RU); Dimitrovgradskij Inzhenerno-Tekhnologicheskij Inst. NIYaU MIFI, Dimitrovgrad (RU)); Valishin, M.F.; Uzikov, V.A.; Palachev, P.S. (AO GNTs NIIAR, Dimitrovgrad (RU))

Issledovanie dostizhimykh temperaturnykh rezhimov v ampul'nom kanale s estestvennoj tsirkulyatsiej

p. 72-78

(RU)

8 refs., 3 figs., 1 tab.

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(no.4)

NUCLEAR POWER; PWR TYPE REACTORS; RESEARCH REACTORS; REACTOR CHANNELS; COOLANTS; NATURAL CONVECTION; IRRADIATION; HELIUM

The current investigation provides the results of the calculation analysis of the thin-walled samples cooling during reactor irradiation in a double-hulled ampoule channel with a natural convection supply of the heatsink

В работе представлены результаты расчетного анализа охлаждения тонкостенных образцов при реакторном облучении в двухкорпусном ампульном канале с организацией теплоотвода естественной конвекцией

Library of graphic symbols for power equipment in the scalable vector graphics format

S97

Yuferov, A.G (Obninskij Inst. Atomnoj Ehnergetiki NIYaU MIFI, Obninsk (RU))

Biblioteka usloynykh graficheskikh oboznachenij ehnergeticheskogo oborudovaniya v formate masshtabiruemoj vektornoj grafiki

p. 79-89

(RU)

16 refs., 8 figs.

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MATHEMATICS; MATHEMATICAL MODELS; COMPUTERIZED SIMULATION;
INFORMATION; NUCLEAR DATA COLLECTIONS; STANDARDS; ETDE

The paper describes the results of creation and application of library of graphic symbols of elements of power equipment under Russian standards GOST 21.403-80 and GOST 2.789-74

В работе описаны результаты создания и применения библиотеки условных графических обозначений элементов энергетического оборудования, предусмотренных российскими стандартами ГОСТ 21.403-80 и ГОСТ 2.789-74

Temperature fields and heat transfer in free-packed pin bundles cooled by heavy liquid-metal

S22

Zhukov, A.V.; Kuzina, Yu.A.; Sorokin, A.P.; Privezentsev, V.V. (AO GNTs RF-FEhI imeni A.I. Lejpunskogo, Obninsk (RU))

Temperaturnye polya i teplootdacha v razdvinyutykh reshetkakh tvehlov, okhlazhdaemykh tyazhelym zhidkometallicheskim teplonositelem

p. 90-100

(RU)

12 refs., 8 figs.

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REACTORS; COOLANTS; LIQUID METALS; FUEL ASSEMBLIES; HEAT TRANSFER;
GRIDS; CRYSTAL LATTICES; NUSSELT NUMBER

The heat transfer and temperature fields in free-packed pin bundles with heavy liquid-metal coolant have been considered at various spacing ways by the authors. The correlations recommended for calculations of Nusselt numbers and temperature non-uniformities over the perimeter of fuel pins are given for the above-mentioned ways of spacing of fuel pins

В работе рассматриваются теплоотдача и поля температур в раздвинутой решетке твэлов с тяжелым жидкометаллическим теплоносителем при различных способах дистанционирования. Приводятся соотношения, рекомендуемые для расчетов чисел Нуссельта и неравномерностей температуры по периметру твэлов для указанных способов дистанционирования

Determinating the stored Wigner energy accumulation rate in the graphite moderator

S22

Mochalov, A.M.; Najmushin, A.G.; Nesterov, V.N.; Pugachev, D.K. (Natsional'nyj Issledovatel'skij Tomskij Politekhnikeskij Univ., Tomsk (RU))

Opređenje skorosti nakopleniya zapasenoj ehnergii Vignera v grafitovom zamedlitele
p. 101-110
(RU)

7 refs., 5 figs., 1 tab.

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NUCLEAR POWER; LWGR TYPE REACTORS; NUCLEAR FUELS; STORED ENERGY;
WIGNER THEORY; MODERATORS; DEFORMATION; FRACTURES

The aim of this work is to develop a method for determining the rate of accumulation of the Wigner energy in the graphite moderator. Stored energy is proportional to the number of point defects in the crystal structure, so the decision comes down to the determination of this number. The article contains analysis of four models to form cascades of defects

Работа посвящена решению задачи по определению значения запасенной энергии Вигнера в облученном графите. Проведен анализ расчетных моделей образования каскадов дефектов. Определены значения каскадных функций для многогруппового расчета и зависимости скоростей накопления запасенной энергии от времени эксплуатации графита

On the use of solution to the adjoint inhomogeneous transport equation in determining parameters of multiplying media

S97

Grabeznoj, V.A.; Dulin, V.A.; Dulin, V.V. (AO GNTs RF - Fiziko-Ehnergeticheskij Inst. imeni A.I. Lejpunskogo, Obninsk (RU))

Ob ispol'zovanii resheniya sopryazhennogo neodnorodnogo uravneniya perenosa pri opredelenii parametrov razmnozhayushchikh sred

p. 111-122

(RU)

14 refs., 6 figs., 5 tabs.

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EQUATIONS; ROSSI ALPHA METHOD; MULTIPLICATION FACTORS; ADJOINT DIFFERENCE METHOD; BOLTZMANN EQUATION; NUCLEAR DATA COLLECTIONS

The measurements to determine the weight of plutonium and the part of ^{239}Pu contained therein have been done by the detector Active Well Coincidence Counter. Authors have shown the unsuitability of the homogeneous quasi-critical equations solutions for the analysis of very deep subcriticalities as solutions of the adjoint inhomogeneous equation not taking into account the neutron multiplication near criticality

Проведены измерения для определения масс блоков плутония и долей содержащегося в них ^{239}Pu детектором нейтронов - активным колодезным счетчиком совпадений АУСС (Active Well Coincidence Counter). Показана непригодность решений однородного условно-критического уравнения для анализа при очень глубоких подкритиках и сопряженного неоднородного уравнения, не учитывающего размножение нейтронов вблизи критики

Experimental research and testing of pump characteristics with nuclear reactor's lead coolant

S22

Beznosov, A.V.; L'vov, A.V.; Bokov, P.A.; Bokova, T.A.; Shikhov, D.V. (Nizhegorodskij Gosudarstvennyj Tekhnicheskij Univ. imeni R.E. Alekseeva, Nizhnij Novgorod (RU))

Ehksperimental'nye issledovaniya i otrabotka kharakteristik nasosov na svintsovom teplonositele p. 123-132

(RU)

8 refs., 6 figs.

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(no.4)

NUCLEAR POWER; FAST REACTORS; PRIMARY COOLANT CIRCUITS; COOLANTS; LIQUID METALS; PUMPS; FLUID FLOW

Experimental research and testing of pumps with lead coolant for fast reactors have been described in the paper

В работе описаны экспериментальные исследования и отработка характеристик насосов на свинцовом теплоносителе для реакторов на быстрых нейтронах

Degenerate optimization problems and optimality of NPPs

S29

Klimenko, A.V. (OF Inst. Sistemno-Ehkonomicheskikh Issledovanij imeni Ya.V. Sheveleva, Ehlektrostal' (RU); Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI, Moscow (RU))

Vyrozhdennye zadachi optimizatsii i optimal'nost' YaEhU

p. 133-143

(RU)

15 refs., 2 figs., 1 tab.

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ECONOMICS; ECONOMIC ANALYSIS; INTEREST RATE; NUCLEAR ENERGY; PARAMETRIC ANALYSIS; LINEAR PROGRAMMING

Optimization of large economic and power engineering systems leads to degenerate solutions of high dimension. This is a very strong mathematical complication. However, the author emphasized that it allows to consider future development of the power industry based on simultaneous use of nuclear power plants (NPPs) together with coal- and gas power plants, or solely on NPPs

Автор представляет возможные пути оптимизация больших систем экономики и энергетики, приводящие к вырожденным решениям большой размерности, позволяющим рассматривать будущее развитие энергетики как на совместной работе ядерных

энергетических установок (ЯЭУ), энергетических установок на угле, энергетических установок на газе, так и только на ЯЭУ

Transactions of the 9-th International scientific and technical conference Safety assurance of NPP with WWER. Scientific and technical electronic edition S21

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

9-th International scientific and technical conference Safety assurance of NPP with WWER
Podol'sk (RU)

19-22 May 2015

9-ya mezhdunarodnaya nauchno-tehnicheskaya konferentsiya Obespechenie bezopasnosti AEhS s VVEhR

Sbornik trudov 9-j mezhdunarodnoj nauchno-tehnicheskoy konferentsii Obespechenie bezopasnosti AEhS s VVEhR. Nauchno-tehnicheskoe ehlektronnoe izdanie

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(RU, EN)

LEADING ABSTRACT; MEETINGS; REACTOR SAFETY; WWER TYPE REACTORS; NUCLEAR POWER PLANTS; SAFETY CULTURE; REACTOR OPERATION; RELIABILITY; VERIFICATION; OPTIMIZATION; FEASIBILITY STUDIES

Submitted materials of the conference (May 9-22, 2015), conducted by OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark#. 9 International scientific-technical conference #Left Double Quotation Mark#Safety Assurance of NPP with WWER#Right Double Quotation Mark# was aimed at the exchange of information and discussion of topics related to the safe operation of nuclear power plants with WWER reactors. The issues of reliability and safety of NPP, the calculation and theoretical and experimental justification of the RU with WWER, verification of computer programs for safety reasons were discussed. The problems of upgrading the RU for improving security are considered. The rationale for the safety of extending the life of NPP and the innovative fourth-generation reactors cooled by supercritical water were discussed at the conference

Представлены материалы конференции (9-22 мая 2015 г.), проведенной ОКБ #Left Double Quotation Mark#ГИДРОПРЕСС#Right Double Quotation Mark#. 9-я международная научно-техническая конференция #Left Double Quotation Mark#Обеспечение безопасности АЭС с ВВЭР#Right Double Quotation Mark# имела своей целью обмен информацией и обсуждение тем, связанных с безопасной эксплуатацией АЭС с ВВЭР. Обсуждались вопросы надежности и безопасности АЭС, расчетно-теоретического и экспериментального обоснования РУ с ВВЭР, верификации компьютерных программ для обоснования безопасности. Рассмотрены проблемы усовершенствования РУ с целью повышения безопасности. Обоснование безопасности продления срока службы АЭС и инновационные реакторы четвертого поколения, охлаждаемые водой сверхкритического давления, явились предметом обсуждения на конференции

Development of technology and modernization of the WWER equipment at SKODA JS

S21

Zdebor, J. (AO ShKODA, Pl'zen' (CZ))

9-th International scientific and technical conference Safety assurance of NPP with WWER

Podol'sk (RU)

19-22 May 2015

9-ya mezhdunarodnaya nauchno-tehnicheskaya konferentsiya Obespechenie bezopasnosti

AEhS s VVEhR

Razvitie tekhnologii i modernizatsii oborudovaniya VVEhR v kompanii SKODA JS

vp.

(RU)

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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NUCLEAR POWER PLANTS; WWER TYPE REACTORS; COMMERCIALIZATION; SERVICE LIFE; REACTOR OPERATION; REACTOR SAFETY; RELIABILITY; REACTOR MAINTENANCE

The process equipment of the reactor plant of the majority of the units with WWER at NPPs of the countries of the Central Europe was manufactured, according to Russian design, basically, at SKODA. Mastering and development of manufacturing method and a long-term operational experience of NPP with WWER enables SKODA experts to carry out maintenance of these units. The operational experience and permanent demands for enhancement of failure-free operation is the basis of gradual modernization of the reactor plant equipment as well as of the equipment for its maintenance. The obtained results enables to offer new designs destined for application in constructing new NPPs with WWER technology, generation III+

Технологическое оборудование реакторной установки большинства блоков ВВЭР на АЭС стран Центральной Европы было изготовлено по российскому проекту, в основном на предприятии ШКОДА. Освоение и развитие технологии изготовления и многолетний опыт эксплуатации АЭС с ВВЭР позволяет специалистам ШКОДА выполнять на современном уровне сервисное обслуживание этих блоков. Опыт эксплуатации и постоянные требования к повышению безотказной эксплуатации являются основой постепенной модернизации оборудования реакторной установки, а также оборудования для ее обслуживания. Достигнутые результаты позволяют предлагать новые наработки, предназначенные для применения при сооружении новых АЭС с технологией ВВЭР поколения III+

Today's nuclear power plants - requirements and implementation options

S21

Kazarin, A.M.; Molchanov, A.V.; Ershov, G.A. (AO ATOMPROEKT, Sankt-Peterburg (RU))

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AEhS s VVEhR

Sovremennaya AEhS - trebovaniya i puti realizatsii

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14 figs., 6 tabs.

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vp.

**WWER TYPE REACTORS; REACTOR SAFETY; NUCLEAR POWER PLANTS;
COMPARATIVE EVALUATIONS; RADIATION PROTECTION; RECOMMENDATIONS;
REGULATIONS**

The main directions of increasing the competitiveness of nuclear power plants in foreign markets are listed. Outlined deterministic requirements of the IAEA to ensure no-risk, requirements EUR to basic and advanced design. Given the targets of radiation safety for NPP-2006 project in accordance with the requirements of the EUR and Russian normative documents, as well as targeted probabilistic safety criteria. Provides information about the hierarchy of normative documents of Finland in the field of NPP safety. Describes the main directions of modernization of NPP-91/99 (NPP-2006) in order to increase the safety level selection logic techniques to ensure defense-in-depth. Comparison of NPP with WWER on key parameters. It is shown that the new NPP projects comply with all current national and international safety requirements

Перечислены основные направления повышения конкурентоспособности АЭС на зарубежных рынках. Изложены требования МАГАТЭ к обеспечению безопасности, требования EUR к базовому и расширенному проектированию. Приведены целевые показатели радиационной безопасности для проекта АЭС-2006 в соответствии с требованиями EUR и российских нормативных документов, а также целевые вероятностные показатели безопасности. Приведены сведения об иерархии нормативных документов Финляндии в области безопасности АЭС. Описаны основные направления модернизации АЭС-91/99 (АЭС-2006) в целях повышения уровня безопасности, логика выбора технических средств для обеспечения глубокоэшелонированной защиты. Произведено сравнение проектов АЭС с ВВЭР по основным параметрам. Показано, что

новые проекты АЭС удовлетворяют всем современным отечественным и зарубежным требованиям по безопасности

Issues associated with post-Fukushima strengthening of international safety requirements for design of nuclear power plants

S22

Misak, J. (Inst. Yadernykh Issledovaniy, Rzhesh (CZ))

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AEhS s VVEhR

Voprosy, svyazannye s uzhestocheniem mezhdunarodnykh trebovanij po bezopasnosti dlya proektov AEhS posle avarii na AEhS Fukusima

vp.

(RU)

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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FUKUSHIMA DAIICHI NUCLEAR POWER STATION; REACTOR SAFETY; IAEA; SAFETY STANDARDS; RADIATION PROTECTION; REACTOR ACCIDENTS; PROGRAM MANAGEMENT

Lessons learned from the Fukushima-Daiichi nuclear power plant accident are discussed, which influenced the world community's views on ensuring an adequate level of nuclear safety. Areas that need to be strengthened in accordance with the new security requirements include the independence of defense-in-depth levels, the practical exclusion of large radioactivity yields, increased resistance to external influences, more stringent requirements for heavy accident mitigation projects, use of mobile power sources and coolant supply, increased strength reserves and exclusion of threshold effects. Questions related to the implementation of increased requirements are presented, and possible ways of resolving open issues are discussed

Обсуждаются уроки, извлеченные после аварии на АЭС Фукусима-Даичи, которые повлияли на взгляды мирового сообщества по обеспечению адекватного уровня ядерной безопасности. Области, которые необходимо усилить в соответствии с новыми требованиями по безопасности, включают независимость уровней глубокоэшелонированной защиты, практическое исключение больших выходов радиоактивности, повышенную устойчивость к внешним воздействиям, более строгие требования к проектам систем по смягчению тяжелых аварий, использование мобильных

источников питания и подачи теплоносителя, увеличенные запасы прочности и исключение пороговых эффектов. Представлены вопросы, связанные с реализацией повышенных требований, и обсуждаются возможные пути решения открытых вопросов

A #Left Single Quotation Mark#New#Right Single Quotation Mark# vision for nuclear reactor safety

S21

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WWER TYPE REACTORS; NUCLEAR POWER PLANTS; REACTOR SAFETY; SAFETY ANALYSIS; DETERMINISTIC ESTIMATION; PROBABILISTIC ESTIMATION; RISK ASSESSMENT

The nuclear reactor safety technology (NRST) is considered, involving nuclear fission reactor and water-water reactors. A two-level idea is discussed. On the one hand, an independent mandatory safety analysis is required, planned and implemented in accordance with the latest scientific and technological developments. On the other hand - feedback analysis with the project, construction and operation of the nuclear power plant based on the design of systems and management. For this purpose, the parameters are determined and continuous monitoring of the extended safety stocks is carried out. The role of the safety analysis as a key component of the nuclear reactor safety technology is considered first. The concept of continuously monitored extended safety stocks is described

Рассматривается технология безопасности ядерного реактора (NRST), включая ядерные реакторы деления и водо-водяные реакторы. Обсуждается двухуровневая идея. С одной стороны, необходим независимый обязательный анализ безопасности, планируемый и выполняемый в соответствии с новейшими научными и технологическими разработками. С другой стороны - обратная связь анализа с проектом, строительством и эксплуатацией атомной электростанции, исходя из проекта систем и управления. С этой целью

осуществляется определение параметров и постоянный мониторинг расширенных запасов безопасности. Роль анализа безопасности в качестве ключевой составляющей технологии безопасности ядерного реактора рассматривается в первую очередь. Описана концепция непрерывно контролируемых расширенных запасов безопасности

Probabilistic safety analysis second level of WWER-TOI

S21

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Veroyatnostnyj analiz bezopasnosti vtorogo urovnya VVEhR-TOI

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REACTOR SAFETY; SAFETY STANDARDS; WWER TYPE REACTORS;

PROBABILISTIC ESTIMATION; RISK ASSESSMENT; NUCLEAR POWER PLANTS;

PLANNING

Probabilistic safety assessment (PSA) of Level-1 and Level-2 gives a comprehensive qualitative and quantitative evaluation of the safety of the project. The operation of the unit at rated power is considered. As sources of radioactivity in the development of the second-level PSA, nuclear fuel in the core of the reactor is considered. As initiating events, internal initiating events (including de-energizing) are considered, which may arise due to failures of NPP systems, equipment or components, or due to erroneous actions of personnel. In general, an assessment of the level of project safety shows that the WWER-TOI project complies with the requirements of the TOR, as well as all the requirements of modern Russian and foreign regulatory documents in the field of security

Вероятностный анализ безопасности (ВАБ) первого и второго уровней дает комплексную качественную и количественную оценку безопасности проекта. Рассматривается работа блока на номинальной мощности. В качестве источников радиоактивности при разработке ВАБ второго уровня рассматривается ядерное топливо в активной зоне реактора. В качестве иницирующих событий приняты внутренние иницирующие события (включая

обесточивание), которые могут возникнуть вследствие отказов систем, оборудования или компонентов АЭС или вследствие ошибочных действий персонала. В целом, оценка уровня безопасности проекта показывает, что проект ВВЭР-ТОИ соответствует требованиям ТЗ, а также всем требованиям современных российских и зарубежных нормативных документов в области безопасности

Time is a safety factor

S22

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NUCLEAR POWER PLANTS; INTERNATIONAL COOPERATION; COORDINATED RESEARCH PROGRAMS; REACTOR ACCIDENTS; REACTOR SAFETY; QUALITY ASSURANCE; RELIABILITY; RUSSIAN ORGANIZATIONS

The work of the World Association of Operators (WANO) of nuclear power plants is presented for the current period. The issues of non-commercial partnership of nuclear energy organizations are discussed. There is an increase in the number of countries that are members of the WANO NPP Moscow Center, improving performance, reducing events and violations. Section #Left-Pointing Double Angle Quotation Mark#Time is a security factor#Right-Pointing Double Angle Quotation Mark# dedicated to the 65th anniversary of nuclear energy, the 35th anniversary of the creation of national operational institutes, the 25th anniversary of the creation of international security institutions, the 5th anniversary of post-Fukushima security events. The activities of the WANO NPP Moscow Center for security improvement are discussed

Представлена работа Всемирной ассоциации операторов АЭС (ВАО АЭС) на настоящий период. Обсуждаются вопросы некоммерческого партнерства организаций атомной энергетики. Отмечается рост числа стран, входящих в Московский центр ВАО АЭС, улучшение показателей работы, уменьшение событий и нарушений. Раздел #Left-Pointing

Double Angle Quotation Mark#Время - фактор безопасности#Right-Pointing Double Angle Quotation Mark# посвящен 65-летию атомной энергетики, 35-летию создания национальных институтов по эксплуатации, 25-летию создания международных институтов по безопасности, 5-летию постфукусимских мероприятий по безопасности. Обсуждается деятельность Московского центра ВАО АЭС по безопасности

Experience in safety review of design solutions of the state-of-the-art WWER-type NPPs

S21

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WWER TYPE REACTORS; RUSSIAN ORGANIZATIONS; NUCLEAR POWER PLANTS; REACTOR SAFETY; DECISION MAKING; PLANNING; QUALITY ASSURANCE; SAFETY CULTURE

The experience of the Federal Budget Institution of the Scientific and Technical Center for Nuclear and Radiation Safety in the field of expertise of the safety rationales for nuclear power plants with WWER-type reactors of new projects is disclosed. In determining the priority, in addition to the necessary time and financial resources, it also took into account the extent to which these activities significantly affect the completeness of the implementation of levels of defense in depth related to the management of beyond-design-basis accidents, including severe ones. And also, what impact does this or that measure have on reducing the likelihood of the onset of severe radiation effects. When examining the safety justification for new design solutions (including for nuclear power plants with a reactor type WWER), it is advisable to adhere to the following approach: during the examination it is necessary to study the experience in the country and the world related to the proposed new design solutions; It is preferable to take advantage of the differential approach to assessing various aspects related to nuclear and

radiation safety. The result of the examination of the justification for new design solutions may be recommendations on the development of existing regulatory documents or development of the Regulatory Authority

Раскрывается опыт Федерального бюджетного учреждения Научно-Технический центр по ядерной и радиационной безопасности в области экспертизы обоснований безопасности атомных станций с реакторами типа ВВЭР новых проектов. При определении приоритетности принималось во внимание, помимо необходимых затрат временных, финансовых ресурсов, также и то, насколько указанные мероприятия существенно влияют на полноту реализации уровней глубокоэшелонированной защиты, связанных с управлением запроектными авариями, в том числе тяжелыми. А также, какое влияние оказывает то или иное мероприятие на снижение вероятности наступления тяжелых радиационных последствий. При проведении экспертизы обоснований безопасности новых проектных решений (в том числе, по атомным станциям с реакторной установкой типа ВВЭР) целесообразно придерживаться следующего подхода: при экспертизе следует изучать имеющийся в стране и мире опыт, относящийся к предлагаемым новым проектным решениям; предпочтительно использовать преимущества дифференцированного подхода к оценке различных аспектов, связанных с обеспечением ядерной и радиационной безопасности. Результатом экспертизы обоснования новых проектных решений могут являться рекомендации о доработке действующих или разработке новых нормативных документов Регулирующего органа

Competitiveness of WWER technology

S21

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TECHNOLOGY ASSESSMENT; NUCLEAR POWER PLANTS; NUCLEAR ENERGY;
FEASIBILITY STUDIES; COMPETITION; COMMERCIALIZATION; INTERNATIONAL
COOPERATION

Successes of the world expansion of technology and WWER are discussed, features of the implementation of foreign projects, as well as directions of new developments - evolutionary and innovative. Among the advantages of WWER technology is the reliability and safety of NPPs with WWER, based on experience in construction and operation, and the competitiveness of WWER technology on the world market. The NPP project can be implemented in any countries of the world taking into account the peculiarities of the national regulatory framework. WWER technology has the potential for development both within the evolution of the generation III+ installations and through the creation of innovative installations

Обсуждаются успехи мировой экспансии технологии ВВЭР, особенности реализации зарубежных проектов, а также направления новых разработок - эволюционные и инновационные. Среди преимуществ технологии ВВЭР - надежность и безопасность АЭС с ВВЭР, обоснованные опытом сооружения и эксплуатации, а также конкурентоспособность технологии ВВЭР на мировом рынке. Проекты АЭС с ВВЭР могут быть реализованы в любых странах мира с учетом особенностей национальной нормативной базы. Технология ВВЭР имеет потенциал развития как в рамках эволюции установок поколения III+, так и по линии создания инновационных установок

New structural materials for next-generation WWER-type reactors

S21

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Novye konstruktsionnye materialy dlya perspektivnykh VVEhR

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CHROMIUM-NICKEL-MOLYBDENUM STEELS; BRITTLENESS; REACTOR CORES;
LIFETIME EXTENSION; BAFFLES

The task is to create a new high-radiation-resistant steel for the internal WWER devices to ensure an increase in the design lifetime of the reactor, discuss the elaboration and commercial exploitation of new welding materials that ensure the weld metal's equal strength of the weld to the main metal of the reactor vessel, the development of a new highly radiation-resistant steel for the core baffle providing its operation for more than 60 years. The materials for the WWER-TOI type reactor vessel are presented - highly-radiation-resistant advanced steel 15Cr2MoVN-A mod A, which provides the strength category SC-45 for shells with a thickness of 600 mm

Ставится задача создания новой высокорадиационнотойкой стали для внутрикорпусных устройств ВВЭР с целью обеспечения повышения проектного срока их эксплуатации, обсуждается разработка и промышленное освоение новых сварочных материалов, обеспечивающих равнопрочность металла сварочного шва основному металлу корпуса реактора, разработка новой высокорадиационнотойкой стали для выгородки активной зоны реактора, обеспечивающей возможность ее эксплуатации свыше 60 лет. Представлены материалы для корпуса реактора типа ВВЭР-ТОИ - высокорадиационнотойкая усовершенствованная сталь 15Х2МФА-А мод А, обеспечивающая категорию прочности КП-45 для обечаек с толщиной 600 мм

Nuclear fuel for Russian power reactors. Stages of the evolution, current status and prospective development directions

S21

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Yadernoe toplivo dlya rossijskikh ehnergeticheskikh reaktorov. Ehtapy ehvolyutsii, tekushchee sostoyanie i perspektivnye napravleniya razvitiya

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WWER TYPE REACTORS; NUCLEAR FUELS; FUEL ASSEMBLIES; REVIEWS; REACTOR SAFETY; LIFETIME EXTENSION; FUEL CYCLE; BURNUP

The main requirements for nuclear fuel today are ensuring security, reliability, economic efficiency and competitiveness. Achieving these goals requires the constant development of existing fuel assembly designs for various power reactors. Guaranteeing the geometric stability of the WWER-1000 fuel assemblies, increasing the fuel assembly's operational life, increasing the operational reliability of fuel assemblies, creating collapsible (repairable fuel assemblies), and implementing safe and cost-effective fuel cycles provides the solution of the main tasks. Much attention is paid to **TVEL** devotes new fuel compositions and works on improving zirconium materials

Основными требованиями к ядерному топливу сегодня является обеспечение безопасности, надежности, экономической эффективности и конкурентоспособности. Достижение указанных целей требует постоянного развития существующих конструкций тепловыделяющих сборок (ТВС) для различных энергетических реакторов. Гарантирование геометрической стабильности конструкций ТВС ВВЭР-1000, увеличение эксплуатационного ресурса ТВС, повышение эксплуатационной надежности ТВС, создание разборных (ремонтпригодных) ТВС, реализация безопасных и экономически эффективных топливных циклов обеспечивает решение основных задач. Большое внимание ОАО **ТВЭЛ** уделяет созданию новых топливных композиций и работам по совершенствованию циркониевых материалов

Assurance of WWER plants safe operation. Results of 2014 and tasks for 2015 **S21**

Shutikov, A.V. (OAO Kontsern Rosehnergoatom, Moscow (RU))

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AEhS s VVEhR

Obespechenie bezopasnoj ehkspluatatsii AEhS s VVEhR. Itogi 2014 goda i zadachi na 2015 god
vp.
(RU)

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po
Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO
Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO
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WWER TYPE REACTORS; REACTOR SAFETY; REVIEWS; LIFETIME EXTENSION; RECOMMENDATIONS; REACTOR ACCIDENTS

The conclusions are drawn on the results of operation of nuclear power plants in Russia in 2014, where it is shown that the existing Russian nuclear power plants comply with Russian requirements for ensuring the safety of nuclear power plants and international recommendations for ensuring nuclear and radiation safety, measures have been developed and implemented to increase the stability of existing nuclear power plants to extreme external influences, and the objects of the new NPPs being built correspond to post-Fukushima the safety requirements are increased, reliability and resistance to extreme external influences of natural and technogenic nature. Ensuring the safe, reliable and cost-effective operation of all operating power units of the NPP with WWER, increasing the capacity of pilot unit 4 of the Balakovo NPP, putting into operation of the 3rd unit of Rostov NPP, bringing physical and power start-up to the 6th unit of Novovoronezh NPP, Balakovo NPP are the main tasks of OJSC Concern Rosenergoatom

Сделаны выводы по результатам эксплуатации АЭС в России в 2014 году, где показано, что действующие российские АЭС соответствуют российским требованиям по обеспечению безопасности АЭС и международным рекомендациям по обеспечению ядерной и радиационной безопасности, разработаны и выполняются мероприятия по повышению устойчивости действующих АЭС к экстремальным внешним воздействиям, а проекты новых сооружаемых энергоблоков АЭС соответствуют постфукусимским требованиям по безопасности, обладают повышенной надежностью и устойчивостью к экстремальным внешним воздействиям природного и техногенного характера. Обеспечение безопасной, надежной и экономически эффективной эксплуатации всех действующих энергоблоков АЭС с ВВЭР, повышение мощности пилотного 4-го блока Балаковской АЭС, введение в промышленную эксплуатацию 3-го блока Ростовской АЭС, приведение физического и энергетического пуска на сооружаемом 6-ом блоке Нововоронежской АЭС, продление проектного срока службы 1-го блока Балаклавской АЭС являются основными задачами ОАО Концерн Росэнергоатом

New generation codes for NPP safety justification

S21

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AEhS s VVEhR

Kody novogo pokoleniya dlya obosnovaniya bezopasnosti AEhS

вр.

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International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; NUCLEAR POWER PLANTS; REACTOR CORES; REACTOR
SAFETY; REACTOR ACCIDENT SIMULATION; ACCURACY; PROGRAMMING

As an example of the successful development of a precision complex, the concept and some results of research version of #Left Double Quotation Mark#Virtual NPP with WWER#Right Double Quotation Mark# creation, allowing to conduct neutron-physical, thermomechanical, physicochemical, thermal-hydraulic or thermodynamic calculations in a completely related formulation are presented. Including: - 3D calculation of neutron-physical characteristics of the core in stationary and transient modes, including emergency modes; - modeling of the behavior of fuel and fission products, including their propagation through the systems of the primary circuit and under the protective shell, taking into account radioactive and thermochemical interconversions; - calculation of modes under normal operation and violations of normal operation, including design and beyond design basis accidents, including two-phase currents, according to the two-fluid model; - modeling of non-stationary processes in the lower part of the WWER-type reactor (including RPC, the space between the bottom of the shaft and the reactor vessel, and also the reactor vessel itself)

В качестве примера успешной разработки прецизионного комплекса представлена концепция и некоторые результаты создания исследовательской версии программно-технического комплекса #Left Double Quotation Mark#Виртуальная АЭС с ВВЭР#Right Double Quotation Mark#, позволяющей проводить в полностью связанной постановке нейтронно-физические, термомеханические, физико-химические, теплогидравлические или теплогидродинамические расчеты. В том числе: 3D расчет нейтронно-физических характеристик активной зоны в стационарных и переходных режимах, включая аварийные режимы; моделирование поведения топлива и продуктов деления, включая их распространение по системам первого контура и под защитной оболочкой с учетом радиоактивных и термохимических взаимопревращений; расчет режимов при нормальной эксплуатации и нарушениях нормальной эксплуатации, включая проектные и запроектные аварии, в том числе с двухфазными течениями, по двухжидкостной модели; моделирование нестационарных процессов в нижней части реактора типа ВВЭР (включая НКР, пространство между днищем шахты и корпусом реактора, а также сам корпус реактора)

Integration between probabilistic and deterministic safety analysis: application of dynamic event trees to probabilistic safety assessment (PSA) level 2 model S22

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AEhS s VVEhR

Sliyanie veroyatnostnogo analiza bezopasnosti (VAB) i raschetnykh kodov dlya obosnovaniya bezopasnosti: analiz modeli VAB vtorogo urovnya na osnove dinamicheskikh derev'ev sobytij vp.

(RU)

21 refs., 1 fig.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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REACTOR SAFETY; PROBABILISTIC ESTIMATION; FAULT TREE ANALYSIS; PLANNING; COMPUTERIZED SIMULATION; CONTAINMENT; CONTAINERS; RADIATION PROTECTION; REACTOR ACCIDENTS; SCRAM; OPTIMIZATION; DECISION TREE ANALYSIS

Experience is presented on the merging of the probabilistic safety analysis (PSA) model (which was created using the ADAPT code) and the integrated code for modeling beyond-design MAAP4 crashes. As an example, a group of initiating events of the PSA model 2, leading to complete loss of external and internal power supply sources of the station, was analyzed. Estimates of the probabilities of various loss scenes of the hermetically sealed container (containment) due to severe accidents of the pressure vessel body (induced vaporization of the steam generator tubes, combustion of hydrogen, detonation of hydrogen, loss of the containment after spraying the fuel upon rupture of the reactor vessel and others) based on 10,000 MAAP4 calculations. The obtained results allow to conclude that the analyzed model of the second-level PSA gives a fairly conservative estimate of the probability of loss of the containment due to the serious accident. At the same time, calculations made using dynamic event trees indicate higher probability of dependent failures of certain designs, systems and elements of the reactor installation. The results obtained make it possible to significantly reduce the uncertainty of the PSA model, which is very important for making optimal decisions

Представлен опыт по слиянию модели вероятностного анализа безопасности (ВАБ) (которая была создана с помощью кода ADAPT) и интегрального кода для моделирования запроектных аварий МААР4. В качестве примера проанализирована группа инициирующих событий модели ВАБ 2, ведущих к полной потере внешних и внутренних источников электроснабжения станции. Произведены оценки вероятностей различных сценариев потери гермооболочки (контейнента) вследствие тяжелой аварии корпусного реактора под давлением (индуцированный разрыв трубок парогенератора, горение водорода, детонация водорода, потеря контейнента после распыления топлива при разрыве корпуса реактора и других) на основе 10000 вычислений МААР4. Полученные результаты позволяют заключить, что проанализированная модель ВАБ второго уровня дает достаточно консервативную оценку вероятностей потери контейнента из-за тяжелоаварийных явлений. В то же время вычисления, сделанные с помощью динамических деревьев событий, указывают на более высокие вероятности зависимых

отказов определенных конструкций, систем и элементов реакторной установки. Полученные результаты позволяют значительно уменьшить неопределенность модели ВАБ, что очень важно для принятия оптимальных решений

Conservative initial event time appearance determination method of transient processes for WWER-1000 cyclic load curve operation by KORSAR/GP code S21

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Konservativnaya metodika opredeleniya vremeni vozniknoveniya iskhodnogo sobytiya pri manevrennom rezhime izmeneniya moshchnosti dlya raschetov avarijnykh protsessov na RU VVEhR po kodu KORSAR/GP

vp.

(RU)

4 refs., 11 figs., 2 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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COMPUTERIZED SIMULATION; REACTOR KINETICS; WWER TYPE REACTORS; COMPUTER CODES; REACTOR SAFETY; ALGORITHMS; REACTOR OPERATION; REACTOR CORES; SENSITIVITY; THERMAL HYDRAULICS

Modeling of maneuvering mode with a daily load capacity plot was carried out. In the course of the work, the consistency of the results obtained from the interconnected calculation code with the three-dimensional neutron kinetics KORSAR /GP and the diffusion neutron-physical code SAPHIR_95&RC at the given thermohydraulic parameters in the core was checked. As a result of the conducted studies it was also shown that the proposed method based on the search for canonical expansions of the criteria parameters is suitable for development and application in preliminary analyzes on the grounding for safety of RUs operating in the mode of daily maneuvering. Using the proposed approach, it is possible to quickly evaluate the change in acceptance criteria in an emergency process, depending on the initial state. The methodology has ample opportunities for expanding the scope of application and involving additional algorithms and can be used as an element of the sensitivity analysis for performing interconnected calculations

Проведено моделирование маневренного режима с суточным графиком нагрузки мощности. В ходе работы проверена согласованность результатов, полученных по взаимосвязанному расчетному коду с трехмерной нейтронной кинетикой КОРСАР/ГП и диффузионному нейтронно-физическому коду САПФИР_95&RC при заданных теплогидравлических параметрах в активной зоне. В результате проведенных исследований было также показано, что предлагаемая методика, основанная на поиске канонических разложений критериальных параметров, пригодна для развития и применения в предварительных анализах по обоснованию безопасности РУ, работающих в режиме суточного маневрирования. При помощи предлагаемого подхода возможно быстро оценить изменение приемочных критериев в аварийном процессе в зависимости от исходного состояния. Методика имеет широкие возможности для расширения области применения и вовлечения в себя дополнительных алгоритмов и может использоваться как элемент анализа чувствительности при проведении взаимосвязанных расчетов

Effect of passive safety systems on temperature state of fuel rod simulators

S21

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Vliyanie passivnykh sistem bezopasnosti na temperaturnoe sostoyanie imitatorov tvehl
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WWER TYPE REACTORS; REACTOR SAFETY; VERIFICATION; REACTOR ACCIDENT SIMULATION; T CODES; K CODES; S CODES; PROCESS DEVELOPMENT UNITS; MATHEMATICAL MODELS; FUEL RODS; TEMPERATURE CONTROL

On a large-scale integrated thermophysical stand of PSS-WWER, nine experiments were performed with simulating three types of failures in the case of passive safety systems and the imposition of total loss of alternating current sources. The experimental results confirmed the effectiveness of passive safety systems provided for new WWER projects in terms of performing

critical safety functions, namely, maintaining the fuel rod surface temperature at a safe level. Based on the experimental data obtained, verification of the Russian calculation codes TRAP-KS, KORSAR/GP and SOKRAT

На крупномасштабном интегральном теплофизическом стенде ПСБ-ВВЭР выполнено девять экспериментов с моделированием аварии трех разных типов при работе пассивных систем безопасности и наложении полной потери источников переменного тока. Экспериментальные результаты подтвердили эффективность пассивных систем безопасности, предусмотренных в новых проектах ВВЭР с точки зрения выполнения критических функций безопасности, а именно, поддержание температуры поверхности твэла на безопасном уровне. На основании полученных экспериментальных данных выполнена верификация российских расчетных кодов ТРАП-КС, КОРСАР/ГП и СОКРАТ

Application of artificial neural networks for scram reliability analyses

S22

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NEURAL NETWORKS; EXPERT SYSTEMS; REACTOR SAFETY; SCRAM; SYSTEMS ANALYSIS; REACTOR PROTECTION SYSTEMS; PROBABILISTIC ESTIMATION; RISK ASSESSMENT; ALGORITHMS; REACTOR CONTROL SYSTEMS; CONTROL ELEMENTS

The subject of reactor scram reliability in case of multiple control rods failures is under study. An emergency protection failure means a breach of any safety criterion in the event that the system needs to be triggered. Probability of this non-fulfillment is determined by probability of failure of various number of CPS CR, conditional probability of violation of safety acceptance criteria with such failures. A technique based on the use of artificial neural networks has been

developed, which makes it possible to estimate the probability of violation of the safety criteria for various combinations of failures. The structure of an artificial neural network is described, as well as the algorithm of its training. The possibility of a neural network to predict the result is shown

Исследуется вопрос надежности системы аварийной защиты реактора с учетом отказов различного количества ОР СУЗ. Под отказом аварийной защиты понимается нарушение какого-либо критерия безопасности в случае, когда необходимо срабатывание этой системы. Вероятность такого отказа определяется вероятностью отказа различного количества ОР СУЗ, условной вероятностью нарушения приемочных критериев безопасности при конкретном количестве отказов. Разработана методика, основанная на применении искусственных нейронных сетей, которая позволяет оценить вероятность нарушения критериев безопасности для различных комбинаций отказов. Описывается строение искусственной нейронной сети, а также алгоритм ее обучения. Показана возможность нейронной сети предсказывать результат

Calculational study of hydrodynamics and heat and mass transfer in the pressure chamber of WWER-600 reactor

S21

Volkov, V.Yu.; Golibrodo, L.A.; Krutikov, A.A.; Kudryavtsev, O.V.; Nadinskij, Yu.N.; Nechaev, A.T.; Nikolaeva, A.V.; Skibin, A.P. (OKB GIDROPRESS, Podol'sk (RU))

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International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; PRESSURE CONTROL; HYDRODYNAMICS; MASS TRANSFER; HEAT TRANSFER; REACTOR COOLING SYSTEMS; DISCHARGE CANALS; IN PILE LOOPS; S CODES; COMPUTERIZED SIMULATION

The present work is devoted to the calculational study of hydrodynamics and heat and mass transfer in RPC of WWER-600 reactor with operation of two and one loop, using CFD-code

STAR-CCM+. Within the frame of the performed study the calculational model was developed with the dimensional representation as 275 mln. of control volumes in the selected calculated region for different operation conditions of the plant. The analysis of the obtained calculated data was made as well as a comparison with the known experimental and calculational data.

Настоящая работа посвящена расчетному исследованию гидродинамики и теплообмена в НКР ВВЭР-600 при работе на двух и на одной петле с помощью CFD-кода STAR-CCM+. В рамках проведенного исследования была разработана расчетная модель размерностью 275 млн. контрольных объемов и получены распределения параметров теплоносителя в выбранной расчетной области для различных режимов работы установки. Выполнен анализ полученных расчетных данных, проведено сопоставление с известными экспериментальными и расчетными данными

Experience in carrying out calculations of severe beyond design basis accidents for RU WWER using code SOKRAT

S21

Lityshev, A.V.; Pantyushin, S.I.; Aulova, O.V.; Gasparov, D.L.; Bukin, N.V.; Bykov, M.A. (OKB GIDROPRESS, Podol'sk (RU))

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AEhS s VVEhR

Opyt vypolneniya analizov tyazhelykh zaproektnykh avarij dlya RU VVEhR s ispol'zovaniem RK SOKRAT

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The main results of the work carried out in 2012-2014 by the OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark# using code SOKRAT/V1, as well as the list of planned works for 2015-2016 are presented. Using the SOKRAT/V1 code today, the following tasks are being accomplished: Substantiation of the safety of the reactor installation

during severe beyond design basis accidents; Substantiation of emergency instructions for beyond-design and heavy accidents; Determination of the parameters of mass and energy of the melt, coolant and hydrogen coming from the reactor to the containment; Substantiation of efficiency and working capacity of the system of intracorporeal confinement of the melt; Implementation of R&D and other works aimed at supporting the design and operation of NPP with the WWER ; Participation in the development of #Left Double Quotation Mark#Virtual NPP#Right Double Quotation Mark# concepts, including work on the introduction of supercomputer technologies. Approaches to the development of computational models and the implementation of analyzes of severe accidents using code SOKRAT are considered. In OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark# this process is clearly structured and provides a noticeable reduction in the influence of the human factor on the results, which reduces the possibility of the appearance of erroneous results

Представлены основные результаты работ 2012-2014 годов, выполненных ОКБ #Left Double Quotation Mark#ГИДРОПРЕСС#Right Double Quotation Mark# с использованием РК СОКРАТ/В1, а также перечень планируемых работ на 2015-2016 годы. С использованием кода СОКРАТ/В1 сегодня решаются следующие задачи: обоснование безопасности реакторной установки в ходе тяжелых запроектных аварий; обоснование аварийных инструкций для запроектных и тяжелых аварий; определение параметров массы и энергии расплава, теплоносителя и водорода, поступающих из РУ в ЗО; обоснование эффективности и работоспособности системы внутрикорпусного удержания расплава; выполнение НИОКР и других работ, направленных на поддержку проектирования и эксплуатации АЭС с РУ ВВЭР; участие в разработке концепций #Left Double Quotation Mark#Виртуальная АЭС#Right Double Quotation Mark#, включая работы по внедрению суперкомпьютерных технологий. Рассмотрены подходы к разработке расчетных моделей и выполнению анализов тяжелых аварий с использованием РК СОКРАТ. В ОКБ #Left Double Quotation Mark#ГИДРОПРЕСС#Right Double Quotation Mark# данный процесс четко структурирован и обеспечивает заметное снижение влияния человеческого фактора на результаты, что сокращает возможность появления ошибочных результатов

TRAP-KS code three-dimensional neutron kinetics model verification by calculation results of tests with reactivity variations

S21

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Verifikatsiya trekhmernoj modeli nejtronnoj kinetiki koda TRAP-KS po rezul'tatam raschetov testovyx zadach s izmeneniem reaktivnosti

vp.

(RU)

9 refs., 19 figs., 1 tab.

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WWER TYPE REACTORS; T CODES; VERIFICATION; COMPARATIVE EVALUATIONS; REACTOR ACCIDENT SIMULATION; REACTOR ACCIDENTS; REACTOR SAFETY; PROGRAMMING; COMPUTER CALCULATIONS; REACTOR KINETICS

The analysis of some computational tasks from the report on verification of the TRAP-KS software complex, which is the result of the upgrade of the TRAP-97 complex, is given. The code is intended for carrying out related neutron-physical and thermal-hydraulic calculations of dynamic processes in reactor plants with WWER. Verification was carried out on the basis of comparing the calculated results with the results of experiments on the field units of the NPP and the results of calculations for alternative software. As a result of the research it was shown that the TRAP-KS code with good accuracy calculates the main parameters of the spatial variation of the energy release field in the reactor in conditions of reactive accidents, asymmetrical operation of circulating loops and non-stationary poisoning during long transient processes. Based on the results of verification, the scope of application of the software in TRAP-KS version is extended to the calculation of reactive accidents

Приведен анализ некоторых расчетных задач из отчета по верификации программного комплекса ТРАП-КС, который является результатом модернизации комплекса ТРАП-97. Код предназначен для проведения связанных нейтронно-физических и теплогидравлических расчетов динамических процессов на реакторных установках с ВВЭР. Верификация проводилась на основе сравнения расчетных результатов с результатами экспериментов на натуральных блоках АЭС и результатами расчетов по альтернативным ПС. В результате исследований показано, что код ТРАП-КС с хорошей точностью рассчитывает основные параметры пространственного изменения поля энерговыделения в реакторе в условиях реактивных аварий, несимметричной работы циркуляционных петель и нестационарного отравления при длительных переходных процессах. По результатам верификации, область применения программного комплекса в версии ТРАП-КС распространена на расчет реактивных аварий

Modeling thermohydrodynamic processes in the water-cooled reactor core **S21**

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Modelirovanie termogidrodinamicheskikh protsessov v aktivnoj zone vodookhlazhdaemogo reaktora

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WWER TYPE REACTORS; THERMAL HYDRAULICS; FLUID FLOW; MATHEMATICAL MODELS; FUEL ASSEMBLIES; COMPUTER CALCULATIONS; PHASE VELOCITY; CONVECTION; COMPUTER CODES; REACTOR CORES

Has been made the computer code of simulation of the processes of the hydrodynamics and heat transfer in the flow of coolant, which should act as a liaison between the system code and the subchannel code. This code is designed to calculate the parameters of the coolant, which are averaged over the cross section of the fuel assembly, so we'll call it the cassette thermohydraulic code (or just #En Dash# cassette code). For the numerical implementation of the proposed mathematical subchannel model used semi-implicit numerical scheme. All variables that are present in the source terms and in the terms describing of interconnect on the interphases surfaces and on the surfaces of solid walls are treated in an implicit interpretation: they are interpreted as the values in new point in time. The phase velocities included in the convective terms of the mass and the energy, and pressure, which is present in all members of differentiable equations are interpreted in the same way. In contrast, members of the convective flux of mass, momentum, energy evaluated at the old time step, that is, in explicit form. For discretization in space (axial) variable used #Right Double Quotation Mark#chess#Right Double Quotation Mark# grid. The results of computational experiments are presented

Разработана теплогидравлическая программа, которая должна выступать в качестве связующего звена между системным и субканальным кодом. Она предназначена для расчета усредненных по поперечному сечению ТВС параметров теплоносителя, поэтому ее называют кассетным теплогидравлическим кодом (или просто - кассетным кодом). Для численной реализации используется полуявная численная схема. Все переменные, которые присутствуют в источниковых членах, и переменные, описывающие взаимодействие на межфазных поверхностях и твердых стенках, трактуются в неявной формулировке: они берутся в новый момент времени. Аналогично интерпретируются фазовые скорости, входящие в конвективные члены массы и энергии, и давления. Напротив, члены конвективных потоков массы, импульса и энергии оцениваются в старый момент времени, то есть в явном виде. Для дискретизации по пространственной (аксиальной) переменной используется #Left Double Quotation Mark#шахматная#Right Double Quotation Mark# сетка. Представлены результаты вычислительных экспериментов

the #Left Double Quotation Mark#post-Fukushima#Right Double Quotation Mark# period

S21

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Perspektivy razvitiya modelirovaniya i analiza tyazhelykh avarij v #Left Double Quotation Mark#post-fukusimskij#Right Double Quotation Mark# period

vp.

(RU)

2 refs., 1 tab.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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The analysis of the international SARNET projects on prioritization of further researches of settlement and experimental modeling of severe accidents in pressurized water reactors taking into account lessons of the Fukushima accident is submitted. In addition to the SARNET projects need of additional researches of increase of efficiency of strategy of management of pressure and cooling of the damaged active zone of the reactor for prevention of destructive steam-gas explosions is proved (including for the Mixed-Oxide fuel. Justifications of increase of efficiency of strategy of management of severe accidents are defined by need of improvement of methodical ensuring definition of conditions (criteria) of emergence of a hydrogen detonation and steam explosions at significantly dynamic modes of development of processes of severe accidents both on inside - and at extra- vessel stages. These directions of development of modeling and the analysis of severe accidents have to be priority during the #Left Double Quotation Mark#post-Fukushima#Right Double Quotation Mark# period

Представлен анализ международных проектов SARNET по приоретизации дальнейших исследований расчетного и экспериментального моделирования тяжелых аварий в корпусных ядерных реакторах с учетом уроков Фукусимской аварии. В дополнение к проектам SARNET обоснована необходимость дополнительных исследований повышения эффективности стратегий управления давлением и охлаждения поврежденной активной

зоны реактора для предотвращения разрушительных парогазовых взрывов (в том числе для МОХ-топлива). Обоснования повышения эффективности стратегий управления тяжелыми авариями определяет необходимость совершенствования методического обеспечения определения условий (критериев) возникновения водородной детонации и паровых взрывов при существенно динамических режимах развития процессов тяжелых аварий как на внутри-, так и на внекорпусных стадиях. Эти направления развития моделирования и анализа тяжелых аварий должны быть приоритетными в #Left Double Quotation Mark#пост-фукусимский#Right Double Quotation Mark# период

SMART2013 specimen: structural response by non-linear dynamic analysis

S22

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BENCHMARKS; NONLINEAR PROBLEMS; REACTOR STABILITY; RESPONSE FUNCTIONS; MATHEMATICAL MODELS; PARAMETRIC ANALYSIS; SEISMIC EFFECTS; REINFORCED CONCRETE; REACTOR ACCIDENT SIMULATION; REACTOR SAFETY

The modelling approach, basic material properties, finite element types and used software and obtained, simulated structural response results for SMART2013 specimen are described. For two horizontal directions the acceleration amplification of the SMART2013 specimen model response was quite moderate that indicates that in these two directions the nonlinear and cracking behaviour of the model response is dominating. The circumstances are different in the vertical direction where acceleration amplification from the bottom of the model up to the response point (node 7563) is nine times indicating that in this direction the linear behaviour in the model response is dominating.

Описываются подходы к моделированию, свойства основных материалов, типы конечных элементов и используемые программные средства, а также полученные результаты по

смоделированной динамической характеристике конструкции образца SMART2013. Для двух горизонтальных направлений усиление ускорения частотной характеристики модели образца SMART2013 довольно умеренное, что говорит о доминировании в отклике модели нелинейной характеристики и растрескивания в этих двух направлениях. Обстоятельства отличаются в вертикальном направлении, где наблюдается девятикратное усиление ускорения с нижней части модели до точки отклика (ячейка 7563), что говорит о доминировании линейной характеристики отклика модели в этом направлении

System for NPP accidents identification for various stages of its progression on base of system best estimate codes and neural networks

S21

Chyong Van, K.N.; Vorob'ev, Yu.B. (NIU MEhI, Moscow (RU))

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Sistema raspoznavaniya avarij na AEhS dlya razlichnykh stadij ikh razvitiya s ispol'zovaniem integral'nykh kodov i nejronnykh setej

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An approach is presented for the use of modern information technologies in conjunction with methods for analyzing the dynamics of emergency processes on the basis of integrated codes for improved estimation, an uncertainty analysis system for constructing an accident identification system for nuclear power plants. The architecture of the system is described, the basic steps for its creation are demonstrated. Particular attention is paid to the issues of system tuning, the choice of the optimal structure of neural networks. Also, the question of using the system in the accident process is examined to identify possible ways of its development. To this end, a qualitative formalization of the problem has been determined and solutions have been worked out. Within the framework of this issue, a study was carried out of the recognition of the type of

accident for different time points, which showed a complex nonlinear dependence of the indicators of difficulty and quality of training. The practical possibility of creating a system for identifying the type of emergency for NPPs with WWER-1000 is shown on the basis of the described approach, which can be used as an operator support system or a crisis center

Представлен подход по использованию современных информационных технологий совместно с методами анализа динамики аварийных процессов на основе интегральных кодов улучшенной оценки, системы анализа неопределенностей для построения системы идентификации аварии на АЭС. Описана архитектура системы, продемонстрированы основные шаги по ее созданию. Особое внимание уделяется вопросам настройки системы, выбору оптимальной структуры нейронных сетей. Также рассматривается вопрос использования системы в процессе аварии для идентификации возможных путей ее развития. Для этого определена качественная формализация проблемы и выработаны пути решения. В рамках данного вопроса проводилось исследование распознавания типа аварии для различных временных точек, которое показало сложную нелинейную зависимость показателей трудности и качества обучения. Показана практическая возможность создания системы идентификации типа аварийной ситуации для АЭС с ВВЭР-1000 на основе описанного подхода, которая может использоваться как система поддержки оператора или кризисного центра

OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark# participation in OECD/NEA-PSI PANDA Benchmark

S22

Volkov, V.Yu.; Golibrodo, L.A.; Krutikov, A.A.; Kudryavtsev, O.V.; Nadinskij, Yu.N.; Nechaev, A.T.; Nikolaeva, A.V.; Skibin, A.P.; Shchukin, A.A. (OKB GIDROPRESS, Podol'sk (RU))

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Uchastie OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark# v benchmarke OECD/NEA-PSI PANDA Benchmark

vp.

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COMPUTERIZED SIMULATION; BENCH-SCALE EXPERIMENTS; RUSSIAN ORGANIZATIONS; HELIUM; REACTOR SAFETY; ABUNDANCE; TEMPERATURE MEASUREMENT; TURBULENCE; EQUATIONS OF STATE

The processes of propagation and erosion of the stratified helium layer used as a replacement for hydrogen in the closed volume of the experimental PANDA tank installed at the Paul Scherrer Institute (Switzerland) were simulated in the OECD/NEA-PSI Benchmark . The experiment measured the nonstationary concentration and temperature of helium at several points in the tank volume, as well as the profiles of the velocity and intensity of turbulence along several lines. To simulate helium propagation, an additional equation of concentration transfer was used, for the helium-air mixture, the equation of state of an ideal gas was used. For the calculation, a block-structured hexahedral grid model with a dimension of 1.2 million control volumes was constructed. The calculation was carried out in a non-stationary formulation, the time step was chosen adaptively by the convergence criterion for the basic equations up to $1e-5$ for three internal iterations at step. There is good agreement between the results of computational studies on experimental data

Проведено моделирование процессов распространения и эрозии стратифицированного слоя гелия, используемого в качестве замены водорода в замкнутом объеме бака экспериментальной установки PANDA, установленной в Институте Пауля Шеррера (Швейцария) в рамках бенчмарка OECD/NEA-PSI PANDA Benchmark. В эксперименте измерялись нестационарные концентрации и температура гелия в нескольких точках в объеме бака, а также профили скорости и интенсивности турбулентности вдоль нескольких линий. Для моделирования распространения гелия использовалось дополнительное уравнение переноса концентрации, для смеси гелия с воздухом применялось дополнительное уравнение состояния идеального газа. Для расчета была построена блочно-структурированная гексаэдрическая сеточная модель размерностью 1.2 млн. контрольных объемов. Расчет проводился в нестационарной постановке, шаг по времени выбирался адаптивно по критерию сходимости по основным уравнениям до $1e-5$ за три внутренних итерации на шаге. Наблюдается хорошее соответствие результатов расчетных исследований экспериментальным данным

Optimization of perforated distribution plate in steam generator PGV-1000MKO using CFD

S21

Volkov, V.Yu.; Golibrodo, L.A.; Krutikov, A.A.; Kudryavtsev, O.V.; Lakhov, D.A.; Nadinskij, Yu.N.; Nikolaeva, A.V.; Skibin, A.P.; Sotskov, V.V. (OKB GIDROPRESS, Podol'sk (RU))

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Optimizatsiya paropriemnogo dyrchatogo lista parogeneratora PGV-1000MKO s primeneniem CFD

vp.

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WWER TYPE REACTORS; STEAM GENERATORS; REACTOR COOLING SYSTEMS; STEAM GENERATION; NUCLEAR POWER PLANTS; REMOVAL; OPTIMIZATION; PERFORATION; COMPUTER CALCULATIONS; HYDRODYNAMICS; NOZZLES

Design of steam distribution perforated sheet SDPS for PGV-1000MKO was optimized using CFD methods, and guidelines for choosing the design of SDPS perforation were provided. Computational studies for optimized design of SDPS were performed for several characteristic variants of the mass water level in the steam generator and the distribution of the steam load. Recommendations for commissioning tests procedure for the SG were given on the basis of the research results. Processes in the PGV-1000MKO version with the collector equipped with ten steam nozzles were modeled. A comparison of technical parameters of the design with a single steam nozzle and with ten steam nozzles was performed

Проведена оптимизация конструкции пароприемного дырчатого листа ППДЛ ПГВ-1000МКО с применением CFD методов и даны рекомендации по степени перфорации и параметрами отверстий в листах ППДЛ. Выполнено моделирование процессов в оптимизированном ППДЛ для нескольких характерных вариантов положения массового уровня воды в парогенераторе и распределения паровой нагрузки. По результатам исследований даны рекомендации к проведению пусконаладочных испытаний ПГ. Проведено моделирование процессов в варианте конструкции ПГВ-1000МКО с десятью паровыми патрубками и сопоставление с аналогичными характеристиками для оптимизированного варианта конструкции ПГВ-1000МКО с одним паровым патрубком

Evaluation of uncertainties in code models using a method for solution of backward verification problem

S21

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AEhS s VVEhR

Primenenie metoda resheniya obratnoj zadachi verifikatsii dlya obosnovaniya neopredelennostej model'nykh parametrov raschetnykh kodov

vp.

(RU)

11 refs., 4 figs., 1 tab.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO

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WWER TYPE REACTORS; REACTOR SAFETY; VERIFICATION; K CODES;
MULTIVARIATE ANALYSIS; DATA COVARIANCES; THERMAL HYDRAULICS;
REACTOR COOLING SYSTEMS; REACTOR ACCIDENT SIMULATION

A method for estimating uncertainties in the model parameters of thermohydraulic codes is proposed, based on the solution of the inverse verification problem. The method is based on the application of the laws of mathematical statistics and uses the objective criteria of the experimental information involved. Practical implementation of the method is demonstrated on the problem of estimating the uncertainty of the parameter corresponding to the code KORSAR RK for calculating the critical mass flow steam content in round tubes. The use of the method makes it possible in a number of cases to reduce the conservatism of expert estimates of the uncertainties in the model parameters of the calculation codes

Предложен метод оценки неопределенностей модельных параметров теплогидравлических кодов, основанный на решении обратной задачи верификации. Метод основан на применении законов математической статистики и использует объективные критерии привлекаемой экспериментальной информации. Практическая реализация метода продемонстрирована на задаче оценки неопределенности параметра, отвечающего в расчетном коде КОРСАР за расчет критического массового расходного паросодержания в круглых трубах. Использование метода позволяет в ряде случаев снизить консерватизм экспертных оценок неопределенностей модельных параметров расчетных кодов

Using the SAPFIR_95&RC_WWER combined method for calculation of pin-wise power distribution in the WWER reactor during power manoeuvres S21

Artemov, V.G.; Ivanov, A.S.; Kuznetsov, A.N.; Shemaev, Yu.P. (FGUP NITI im. A.P. Aleksandrova, Sosnovyj Bor, Leningradskaya Obl. (RU))

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Raschet potvehl'nogo ehnergoydeleniya pri manevrakh moshchnosti v VVEhR s ispol'zovaniem kombinirovannogo metoda v komplekse programm SAPFIR_95&RC_VVEhR

vp.

(RU)

2 refs., 4 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; FUEL PINS; FUEL ASSEMBLIES; REACTOR SAFETY; ENERGY SPECTRA; XENON 135; IODINE 135; S CODES; COMPUTERIZED SIMULATION; REACTOR OPERATION

The method of calculating the pin-wise power distribution by a combined method has been tested using the example of calculating the maneuvering regime. The combined method makes it possible to clarify the behavior of the linear fuel element load in comparison with the results of the superposition method in fuel assemblies with the regulator of the control and protection system in the boundaries of the control groups. The use of prepared on the basis of the results of macro calculation of the pin-wise characteristics, including ^{135}Xe and ^{135}I concentrations, allows to perform an adequate fine-grid calculation of the pin-wise power distribution without expensive calculation of feedbacks on a fine grid, which limits the application of a direct method for solving practical problems

Методика расчета потвэльного энерговыделения комбинированным методом апробирована на примере расчета маневренного режима. Комбинированный метод позволяет уточнить поведение линейной нагрузки твэл в сравнении с результатами методом суперпозиции в ТВС с органом регулирования системы управления и защиты в области границ регулирующих групп. Использование подготовленных на основе результатов макрорасчета потвэльных характеристик, в том числе и концентраций ^{135}Xe и ^{135}I , позволяет выполнить адекватный мелкосеточный расчет потвэльного энерговыделения без дорогостоящего расчета обратных связей на мелкой сетке, что ограничивает применение прямого метода для решения практических задач

Methods of analysis of beyond design basis accidents for the Balakovo NPP **S21**

Morozov, V.B.; Barsukov, A.F.; Minibaev, R.F. (AO Atomehnergoproekt, Moscow (RU))

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Metodika analiza zaproektnykh avarij dlya Balakovskoj AEhS

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International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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The list of beyond design basis accidents (BDBA) is based on the probabilistic safety analysis (PSA) of the first and second levels for various categories of information systems (IS) and a complete set of operational states. The analysis of scenarios of accidents that do not lead to damage to the core, when operating at power, for nuclear reactor standby mode caused by external IS of natural and man-made nature caused by seismic interactions, in-site fires and floods during power operation, and for severe accidents is analyzed. Based on the analysis, a preliminary list of candidates of the BDBA was proposed. For full-scale PSA (PSA-1 and PS-2), taking into account all kinds of effects, the methodology allows selecting the most complete representative list of BDBA

Перечень запроектных аварий (ЗПА) составлен на основе вероятностного анализа безопасности (ВАБ) первого и второго уровней для различных категорий информационных систем (ИС) и полного набора эксплуатационных состояний. Выполнен анализ сценариев аварий, не приводящих к повреждению активной зоны, при работе на мощности, для стояночных режимов, вызванных внешними ИС природного и техногенного характера, вызванных сейсмическими взаимодействиями, внутриплощадочными пожарами и затоплениями при работе на мощности, и для тяжелых аварий. На основе выполненного анализа предложен предварительный перечень кандидатов ЗПА. Для полномасштабного ВАБ (ВАБ-1 и ВАБ-2) с учетом всех видов воздействий изложенная методика позволяет выбрать наиболее полный представительный перечень ЗПА

The integral non-stationary equations of neutron transport for calculations of nuclear reactors kinetics using Monte-Carlo method

S21

Davidenko, V.D.; Zinchenko, A.S. (NITs Kurchatovskij Inst., Moscow (RU)); Kharchenko, I.K. (OKB GIDROPRESS, Podol'sk (RU))

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Integral'nye nestatsionarnye uravneniya perenosa nejtronov dlya raschetov kinetiki yadernykh reaktorov metodom Monte-Karlo

vp.
(RU)

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WWER TYPE REACTORS; NEUTRON FLUX; DELAYED NEUTRONS; ADIABATIC APPROXIMATION; MONTE CARLO METHOD; K CODES; NEUTRON TRANSPORT THEORY; REACTOR KINETICS EQUATIONS; LIQUID METALS; REACTOR CORES

An integral non-stationary equation for the neutron flux with delayed neutrons and integral equations for the form function in the improved quasistatic, quasistatic and adiabatic approximations are given. Approaches to the solution of these equations by the Monte Carlo method are described. The equations are implemented in the programs of KIR and KIR-P. The first one allows solving the integral equation of neutron transport without any approximations, and the second one, using the above approximations. In the latter case, the parameters of point kinetics are also calculated by the Monte Carlo method, taking into account the value function of fission neutrons.. Currently, the program is being tested for calculations on multiprocessor computers of the distributed kinetics of fast reactors with a liquid-metal coolant. It is proposed to combine these programs with the computational codes for the thermohydraulic and thermomechanical processes occurring in the active zones of these reactors

Приводятся интегральное нестационарное уравнение для потока нейтронов с учетом запаздывающих нейтронов и интегральные уравнения для форм-функции в усовершенствованном квазистатическом, квазистатическом и адиабатическом приближениях. Описываются подходы к решению этих уравнений методом Монте-Карло. Уравнения реализованы в программах КИР и КИР-П. Первая позволяет решать интегральное уравнение переноса нейтронов без каких-либо приближений, а вторая - с использованием перечисленных выше приближений. В последнем случае параметры точечной кинетики рассчитываются тоже методом Монте-Карло с учетом функции ценности нейтронов деления. В настоящее время программы тестируются для расчетов на многопроцессорных компьютерах распределенной кинетики быстрых реакторов с жидкометаллическим теплоносителем. Предполагается объединить эти программы с вычислительными кодами теплогидравлических и термомеханических процессов, протекающих в активных зонах этих реакторов

Some characteristics of interfacial interaction in two-phase media

S21

Tokar', O.V. (AO ATOMPROEKT, Sankt-Peterburg (RU))

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Nekotorye kharakteristiki mezhfaznogo vzaimodejstviya v dvukhfaznykh sredakh

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WWER TYPE REACTORS; REACTOR SAFETY; TWO-PHASE FLOW; REYNOLDS NUMBER; NUSSELT NUMBER; BUBBLES; GAS FLOW; HYDRODYNAMICS; LIQUID FLOW

Using the example of the equation of the balance of forces acting on the interphase surface of a bubble emerging in a liquid, it is shown that three universal hydrodynamic criteria can be distinguished from a large number of known dimensionless similarity criteria (Bo is the Bond number, Ku_k is the Kutateladze number and ReN_k is the Reynolds number) for modeling phase-to-phase interfaces. The application of universal similarity criteria made it possible to develop a single dependence for the interfacial resistance in the bubble and slug regime of the flow of a two-phase flow. The formal boundary between the bubbling and the slug regime of the two-phase flow is also determined. Comparison of the obtained dependence with the experimental data on the velocity and the coefficient of resistance of popping single bubbles in a stationary liquid showed fairly good results

На примере уравнения баланса сил, действующих на межфазную поверхность всплывающего в жидкости пузырька, показано, что из большого числа известных безразмерных критериев подобия можно выделить три универсальных гидродинамических критерия (Bo - число Бонда, Ku_k - число Кутателадзе и ReN_k - собственное число Рейнольдса), достаточных для моделирования межфазного взаимодействия фаз. Применение универсальных критериев подобия позволило разработать единую зависимость для межфазного сопротивления в пузырьковом и снарядном режимах течения двухфазного потока. Также определена формальная граница между пузырьковым и снарядным режимами течения двухфазного потока. Сопоставление полученной зависимости с экспериментальными данными по скорости и коэффициенту сопротивления всплывающих одиночных пузырьков в неподвижной жидкости показало достаточно хорошие результаты

About development of software for calculating the values of thinning of equipment and pipelines nuclear power plants, are prone to erosion-corrosion wear

S22

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9-ya mezhdunarodnaya nauchno-tehnicheskaya konferentsiya Obespechenie bezopasnosti AEhS s VVEhR

O razrabotke programmnykh sredstv po raschetu znachenij utonenij stenok oborudovaniya i truboprovodov AEhS, podverzhennykh ehrozionno-korrozionnomu iznosu

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REACTOR COOLING SYSTEMS; FLUID FLOW; PROGRAMMING; COMPUTER CALCULATIONS; DATA ANALYSIS; COMPUTER CODES; CORROSION; EROSION; PIPELINES; NUCLEAR POWER PLANTS; WEAR

Examples of the use of software for the calculation of pipeline damage due to erosion-corrosive wear (ECW) at domestic and foreign NPPs are given. General characteristics of computer codes (CC) and software are presented, development of software and calculation codes for domestic NPPs is discussed. Factors determining the intensity of ECW are shown. The use of software for calculating the speed of ECW and thinning the walls of pipelines of foreign and domestic NPPs and the use of CC and software for NPPs are discussed. The development and certification of the software for the calculation of ECW should be based on guidance documents

Приведены примеры использования программных средств (ПС) для расчетов повреждений трубопроводов по причине эрозионно-коррозионного износа (ЭКИ) на отечественных и зарубежных АЭС. Представлена общая характеристика компьютерных кодов (КК) и ПС, обсуждается разработка ПС и расчетных кодов для отечественных АЭС. Показаны факторы, определяющие интенсивность ЭКИ. Обсуждается использование ПС для расчета скоростей ЭКИ и уточнения стенок трубопроводов зарубежных и отечественных АЭС и использование КК и ПС на АЭС. Разработка и аттестация ПС по расчету ЭКИ должна проводиться на основании руководящих документов

Adaptation the Abaqus thermomechanics code to simulate 3D multipellet steady and transient WWER fuel rod behavior

S21

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AEhS s VVEhR

Adaptatsiya komp'yuternoj tekhnologii Abaqus dlya 3D modelirovaniya povedeniya uchastka
konstruktsii tvehla VVEhR po kampanii

vp.

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**FUEL CANS; WWER TYPE REACTORS; REACTOR OPERATION; FUEL PELLETS;
COMPUTER CODES; CREEP; FUEL ELEMENTS; REACTOR CORES; COMPUTERIZED
SIMULATION; MATHEMATICAL MODELS**

The study of Abaqus technology capabilities for modeling the behavior of the WWER-1000 fuel element for the campaign, taking into account the following features: multi-contact thermomechanical interaction of fuel pellet and fuel can, accounting for creep and swelling of fuel, consideration of creep of the can, setting the mechanisms of thermophysical and mechanical behavior of the fuel - cladding gap. The code was tested on the following developed finite element models: 3D fuel element model with five fuel pellets, 3D fuel element model with one fuel pellet and cleavage in the gap, 3D model of the fuel rod section with one randomly fragmented tablet. The position of the WWER-1000 fuel rod section in the middle of the core and the loads and material properties corresponding to this location were considered. The principal possibility of using Abaqus technology for solving fuel design problems is shown

Проведено исследование возможностей технологии Abaqus для моделирования поведения твэла ВВЭР-1000 по кампании с учетом следующих особенностей: мультиконтактное термомеханическое взаимодействие таблеток топлива и оболочки, учет ползучести и набухания топлива, учет ползучести оболочки, задание механизмов теплофизического и механического поведения зазора топливо - оболочка. Тестирование кода проводилось на следующих разработанных конечно-элементных моделях: 3D модель участка твэла с

пятью таблетками топлива, 3D модель участка твэла с одной таблеткой топлива и сколом в зазоре, 3D модель участка твэла с одной фрагментированной произвольным образом таблеткой. Рассматривалось положение участка твэла ВВЭР-1000 в середине активной зоны и соответствующие этому местоположению нагрузки и свойства материалов. Показана принципиальная возможность использования технологии Abaqus для решения задач проектирования твэлов

Experiment BALI for #Left Double Quotation Mark#LOGOS#Right Double Quotation Mark# verification

S21

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Verifikatsiya PK #Left Double Quotation Mark#LOGOS#Right Double Quotation Mark# na ehksperimente BALI

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International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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RAYLEIGH NUMBER; CONVECTION; PROGRAMMING; TURBULENT FLOW; CORIUM; COMPUTERIZED SIMULATION; COMPUTER CALCULATIONS; L CODES; REACTOR SAFETY; WWER TYPE REACTORS

Modeling of heat exchange processes in the experimental setup using various turbulence models was carried out, advantages and disadvantages of each model were revealed, the direction of further development of the specified program code was determined. The following verification tasks have been chosen with known experimentally confirmed results: investigation of developed turbulent convection in the cube, modeling of the BALI experiment (simulates the oxide phase of the melt bath in the case of the adiabatic upper boundary and the corium of the homogeneous structure in the case of the cooled upper boundary). All calculations are carried out within the framework of the LOGOS program package. The LOGOS at a sufficiently high level reproduces the heat transfer in a heat-generating liquid under natural convection under conditions of high Rayleigh numbers $Ra_i \# \text{Swung Dash} \# 10^6$

Проведено моделирование теплообменных процессов в экспериментальной установке с использованием различных моделей турбулентности, выявлены преимущества и недостатки каждой модели, определено направление дальнейшего развития указанного программного кода. Выбраны следующие верификационные задачи с известными экспериментально подтвержденными результатами: исследование развитой турбулентной конвекции в кубе, моделирование эксперимента BALI (имитирует оксидную фазу ванны расплава в случае с адиабатической верхней границей и кориум гомогенной структуры в случае с охлаждаемой верхней границей). Все расчеты проводятся в рамках пакета программ ЛОГОС. ЛОГОС на достаточно высоком уровне воспроизводит теплообмен в тепловыделяющей жидкости при естественной конвекции в условиях высоких чисел Рэлея Ra_i .

Verification of the model for molten core-concrete interaction during ex-vessel stage of severe accident

S21

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Verifikatsiya modeli vzaimodejstviya teplovydelyayushchego rasplava s betonom na vnekorporusnoj stadii tyazhelej avarii

vp.

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BWR TYPE REACTORS; WWER TYPE REACTORS; CONCRETES; BASALT; SERPENTINE; REACTOR CORES; H CODES; COMPUTERIZED SIMULATION; REACTOR SAFETY; REACTOR ACCIDENTS

A number of one- and two-dimensional experiments with different types of concretes, in particular from the SURC, ACE, and CCI series have been simulated. In the SURC-4 (ISP-24) experiment, the interaction between steel melt and basalt concrete was investigated. In the experiments of ACE L2 and ACE L4, the interaction of partially oxidized melt in the core of the BWR reactor with a concrete base, in the first case completely consisting of silicate concrete, was investigated, in the second case, consisting of a layer of serpentinite and a layer of

construction concrete whose composition is close to silicate. In the experiments of the CCI series, the interaction of the core melt with concrete structures in a two-dimensional configuration under water flood conditions was investigated. Good agreement with the experimental data is shown. The HEFEST-EVA design code can be used to simulate the interaction of melt with concrete in the analysis of scenarios of severe accidents at NPPs with WWER

Проведено моделирование ряда одно- и двумерных экспериментов с разными типами бетонов, в частности, из серии SURC, ACE, CCI. В эксперименте SURC-4 (ISP-24) исследовалось взаимодействие между расплавом стали и базальтовым бетоном. В экспериментах ACE L2 и ACE L4 исследовалось взаимодействие частично окисленного расплава активной зоны реактора BWR с бетонным основанием, в первом случае полностью состоящим из силикатного бетона, во втором случае с состоящим из слоя серпентинового и слоя строительного бетона, состав которого близок к силикатному. В экспериментах серии CCI исследовалось взаимодействие расплава активной зоны с бетонными конструкциями в двумерной конфигурации в условиях залива водой. Показано хорошее согласие с экспериментальными данными. Расчетный код HEFEST-EVA может быть использован для моделирования взаимодействия расплава с бетоном при анализе сценариев тяжелых аварий на АЭС с ВВЭР

Verification of SERPENT for nuclear safety analysis of nuclear facilities

S21

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Verifikatsiya programmnoy sredstva SERPENT dlya otsenki yadernoy bezopasnosti ob"ektov ispol'zovaniya atomnoy ehnergii

vp.

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International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; LWGR TYPE REACTORS; VERIFICATION; DATA ANALYSIS; COMPUTERIZED SIMULATION; REACTOR SAFETY; MULTIPLICATION

FACTORS; FISSION PRODUCTS; COMPUTER CODES; PROGRAMMING; REACTOR KINETICS

The results of the work on the verification of the SERPENT software for the calculation of K_{eff} of various systems containing nuclear fissile materials are presented. Comparison of results of calculations K_{eff} with use of software means SERPENT with experimentally measured values or results of calculations on the generally recognized precision (reference) programs is made. Verification calculations of K_{eff} using SERPENT were carried out for various types of systems containing nuclear fissile materials, such as: uranium-water systems with nuclear fuel of WWER-type reactors, uranium-water-graphite and uranium-water systems with nuclear fuel of RBMK type reactors, and A number of systems containing solutions of uranium and plutonium. The results of the calculations show that the values of K_{eff} of various critical and subcritical systems obtained using SERPENT are in good agreement with experimentally measured values, as well as values obtained using such widely used software tools as MCNP, MCU and SCALE

Представлены результаты работы по верификации программного средства SERPENT для расчета $K_{эфф}$ различных систем, содержащих ядерные делящиеся материалы. Проведено сравнение результатов расчетов $K_{эфф}$ с использованием программного средства SERPENT с экспериментально измеренными значениями или результатами расчетов по общепризнанным (реперным) программам. Верификационные расчеты $K_{эфф}$ с использованием SERPENT проводились для различных видов систем, содержащих ядерные делящиеся материалы, таких как: уран-водные системы с ядерным топливом реакторов типа ВВЭР, уран-водо-графитовые и уран-водные системы с ядерным топливом реакторов типа РБМК, а также ряд систем, содержащих растворы урана и плутония. Результаты выполненных расчетов показывают, что полученные с использованием SERPENT значения $K_{эфф}$ различных критических и подкритических систем хорошо согласуются с экспериментально измеренными значениями, а также значениями, полученными с использованием таких широко распространенных программных средств, как MCNP, MCU и SCALE

Results of design and experimental studies of light gas mixing processes for NPP accident control tasks within ERCOSAM-SAMARA projects

S21

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REACTOR SAFETY; WWER TYPE REACTORS; REACTOR ACCIDENT SIMULATION;
REACTOR ACCIDENTS; K CODES; COMPUTERIZED SIMULATION; NUCLEAR
POWER PLANTS; HYDROGEN; MIXTURES; HEAT EXCHANGERS; CONTAINMENT
SHELLS

Calculation and experimental studies of the possibility of hydrogen stratification in the containment shell under the scenario reproducing the sequence of events in the course of the development of a severe accident with the release of hydrogen in a light water reactor and the influence on the distribution of light gas of the means for managing severe accidents have been carried out. The experimental investigations were carried out with observance of the given initial and boundary conditions, which were determined taking into account the scaling of the volume of the containment shell model to the volume of containment shell of the NPP. The analysis of the results of pre-test and post-test numerical modeling has shown that the KUPOL-MT code allows a preliminary qualitative assessment of thermal and hydraulic processes in the containment shell during the development of a severe accident with hydrogen emission taking into account the operation of the sprinkler, heat exchanger and recombiner

Проведены расчетно-экспериментальные исследования возможности стратификации водорода в защитной оболочке по сценарию, воспроизводящему последовательность событий при развитии тяжелой аварии с выбросом водорода на легководном реакторе и влияния на распределение легкого газа средств управления тяжелыми авариями. Экспериментальные исследования проводились с соблюдением заданных начальных и граничных условий, которые были определены с учетом масштабирования объема модели защитной оболочки к объему защитной оболочки станции. Проведенный анализ результатов претестового и посттестового численного моделирования показал, что код КУПОЛ-МТ позволяет провести предварительную качественную оценку теплогидравлических процессов в защитной оболочке при развитии тяжелой аварии с выбросом водорода с учетом работы спринклера, теплообменника и рекомбинатора

Verification of KORSAR/BR and RELAP/SCDAPSIM/MOD3.4 codes using experiments on reflood of the FA models

S21

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Verifikatsiya kodov KORSAR/BR i RELAP/SCDAPSIM/MOD3.4 na ehksperimentakh s povtornym zalivom modelej TVS

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WWER TYPE REACTORS; PWR TYPE REACTORS; REACTOR SAFETY; LOSS OF COOLANT; VERIFICATION; COMPUTER CODES; BENCH-SCALE EXPERIMENTS; DATA COVARIANCES

The verification of codes KORSAR/BR and RELAP/SCDAPSIM/MOD3.4 showed that both codes reliably describe the core reflooding in the core of PWR/WWER reactors in LOCA accidents. The estimation of the uncertainty in the calculation of KORSAR/BR and RELAP/SCDAPSIM MOD3.4 codes showed an acceptable #Left Double Quotation Mark#width#Right Double Quotation Mark# of the uncertainty range for calculating the maximum temperature of fuel rod simulators (PCT). The performed work allowed to expand the database for verification and improve the accuracy of the description of the KORSAR/BR and RELAP/SCDAPSIM/MOD3.4 codes of the core reflooding

Выполненная верификация кода КОРСАР/БР И RELAP/SCDAPSIM/MOD3.4 показала, что оба кода достоверно описывают стадию повторного залива в активной зоне реактора типа PWR/ВВЭР в авариях LOCA. Проведенная оценка неопределенности расчетов кодов КОРСАР/БР и RELAP/SCDAPSIM/MOD3.4 показала приемлемую #Left Double Quotation Mark#ширину#Right Double Quotation Mark# диапазона неопределенности расчета максимальной температуры оболочек имитаторов твэлов (PCT). Проведенная работа позволила расширить базу данных для верификации и повысить точность описания кодами КОРСАР/БР и RELAP/SCDAPSIM/MOD3.4 стадии повторного залива активной зоны

Software package TRAP-KS for the analysis of WWER RU thermal-physical parameters under accident conditions

S21

Zajtsev, S.I.; Belyaev, Yu.V.; Zakutaev, M.O.; Bykov, M.A. (OKB GIDROPRESS, Podol'sk (RU))

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Programmnyj kompleks TRAP-KS dlya analiza teplogidravlicheskih i nejtronno-fizicheskikh parametrov RU s VVEhR v avarijnykh rezhimakh

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1 fig.

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WWER TYPE REACTORS; REACTOR KINETICS; PROGRAMMING; COMPUTER CALCULATIONS; COMPUTERIZED SIMULATION; REACTOR SAFETY; ATWS; CONTAINMENT SHELLS; REACTOR CONTROL SYSTEMS

The TRAP-KS program complex is the result of the modernization of the TRAP-97 certified in 1999. During the modernization of the complex, the following tasks were solved: calculation of the neutron kinetics of the reactor in the three-dimensional spatial approximation, calculation of the loop-to-loop coolant mixing of the coolant in the reactor chambers, modeling of passive safety systems, taking into account the mutual influence of processes in the reactor and the containment shells. The scope of the TRAP-KS program complex has expanded with the calculation of reactivity accidents, including ATWS, mode analysis for asymmetrical operation of circulation circuit, and the safety implications of new WWER-SW projects with passive safety systems, taking into account the processes in the containment shells under conditions of total loss of alternating current sources

Программный комплекс ТРАП-КС является результатом модернизации аттестованного в 1999 г. комплекса ТРАП-97. При модернизации комплекса были решены следующие задачи: расчет нейтронной кинетики реактора в трехмерном пространственном приближении, расчет межпетлевого перемешивания теплоносителя в камерах реактора, моделирование пассивных систем безопасности, учет взаимного влияния процессов в РУ и ЗО. Область применения программного комплекса ТРАП-КС расширилась в части расчета реактивных аварий, включая АТВС, анализа режимов при несимметричной работе циркуляционных петель, обоснования безопасности новых проектов РУ с ВВЭР пассивными системами безопасности с учетом процессов в ЗО в условиях полной потери источников переменного тока

Methodical and practical aspects of Tko, Tndt and RTndt tests for materials of nuclear power plants (NPP) vessels

S21

Titova, T.I.; Shul'gan, N.A.; Ben'yaminova, Ya.Yu.; Orestov, A.M. (OKB GIDROPRESS, Podol'sk (RU))

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Metodicheskie i prakticheskie aspekty ispytaniy Tko, Tndt i RTndt primenitel'no k materialam dlya reaktorov AEhU

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REACTOR COMPONENTS; SHIELDING MATERIALS; MANGANESE STEELS; CHROMIUM-NICKEL-MOLYBDENUM STEELS; BRITTLENESS; FRACTURE PROPERTIES; STRESS INTENSITY FACTORS; STANDARDIZATION; QUALITY CONTROL; REACTOR SAFETY; WWER TYPE REACTORS

The brittle fracture resistance characteristics are determined for the large-sized shell of 15Kh2NMFA steel intended for the case of Russian nuclear reactors, and compared with similar characteristics for 20MnMoNi55 steel used by European and American manufacturers for pressure vessels of the same type. Substantial differences in the characteristics of these materials are established and explanatory explanations for these differences are presented. It is shown that the criteria for brittle strength used in Russian (Tko) and foreign standards (Tndt and RTndt) have fundamental differences, therefore, the formal application of one criterion in place of the other is incorrect and can lead to a distorted assessment of resistance to brittle fracture of reactor materials

Характеристики сопротивления хрупкому разрушению определены для крупногабаритной обечайки из стали 15Х2НМФА, предназначенной для корпусов реакторов АЭУ российского производства, и сопоставлены с аналогичными характеристиками для стали 20MnMoNi55, используемой европейскими и американскими производителями для сосудов давления такого же типа. Установлены существенные различия в исследуемых характеристиках этих материалов и представлены доказательные объяснения этим различиям. Показано, что критерии хрупкой прочности, используемые в российских (Тко) и зарубежных стандартах (Тndt и RTndt) имеют принципиальные отличия, поэтому формальное применение одного критерия вместо другого является некорректным и может привести к искаженной оценке сопротивления хрупкому разрушению реакторных материалов

Validation and improvement of the STEG code on the basis of the experimental data, obtained at PGV test facility. Calculation analysis of the tests at the PGV

test facility

S21

Melikhov, O.I.; Melikhov, V.I.; Nikonov, S.M.; Parfenov, Yu.V.; Emel'yanov, D.A.; Nerovnov, A.A. (AO EhNITs, Ehlektrogorsk (RU))

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Validatsiya i usovershenstvovanie koda STEG na osnove ehksperimental'nykh dannykh, poluchennykh na stende PGV. Raschetnyj analiz ehksperimentov na stende PGV

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9 refs., 9 figs., 4 tabs.

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WWER TYPE REACTORS; MATHEMATICAL MODELS; THERMAL HYDRAULICS; BENCH-SCALE EXPERIMENTS; REACTOR SAFETY; PERFORATION; STEAM GENERATORS; OPTIMIZATION

The main aim was to validate and improve 3D thermalhydraulic STEG code for simulation of the thermalhydraulic processes in the horizontal steam generator. Code validation was carried out on the basis of the analysis of the experiments devoted to the investigation of the equalization ability of the submerged perforated sheet (SPS) carried out at the PGV test facility. The mathematical model of the STEG code is based on the 3D nonstationary equations of the multiphase fluid dynamics. The two-velocity, two-temperature model is used to simulate the water-steam flow. It is shown that the transition from uniform perforation to non-uniform, on the one hand, provides a better integral equalization (the total flow from the hot half is approximately equal to the total flow from the cold half). At the same time, steam separation deteriorates due to high local values of the residual nonuniformity coefficient near the interface of plates with different degrees of perforation

Основная цель состояла в валидации и усовершенствовании трехмерного теплогидравлического кода STEG, предназначенного для моделирования теплогидравлических процессов в горизонтальном парогенераторе. Валидация кода проводилась на основе анализа экспериментов по выравнивающей способности ПДЛ, выполненных на стенде ПГВ. Математическая модель кода STEG базируется на трехмерных нестационарных уравнениях механики многофазных сред. Для описания течения пароводяной смеси используется двухскоростная, двухтемпературная модель. Показано, что переход с равномерной перфорации на неравномерную, с одной стороны,

обеспечивает лучшее интегральное выравнивание (суммарный расход с горячей половины примерно равен суммарному расходу с холодной половины). При этом ухудшается сепарация пара из-за высоких локальных значений коэффициента остаточной неравномерности вблизи границы смыкания пластин с разной степенью перфорации

Core behaviour of the Loviisa WWER-440 facing a heterogeneous boron dilution during shutdown state natural circulation

S21

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REACTOR CORES; REACTOR SAFETY; LOVIISA-1 REACTOR; BORON; DILUTION; SHUTDOWN; COMPUTER CALCULATIONS; NATURAL CONVECTION; FEEDBACK

the core behaviour is analysed as a pure water plug enters the core during an outage, when the reactor is shutdown, pressure is about 25 bars and the coolant is flowing on natural circulation. In the beginning of the transient, the pure water fills the whole downcomer and lower plenum of the pressure vessel (40.2 m³). The analysis were performed with the Apros 3D nodal core model, which contains an own flow channel and 10 axial nodes for each shrouded fuel assembly. Moreover, a wide-range cross section model was used and the cross sections were calculated with CASMO-4E. The results demonstrate that the WWER-440 reactor can tolerate even large pure water plugs without fuel damages on natural circulation due to the negative feedback effects of reactivity and the backflows during the transient. Although, intense boiling and heat transfer crises occur temporarily but this does not directly indicate cladding failures

Анализируется поведение активной зоны при поступлении в нее пробки из чистой воды во время стоянки остановленного реактора в условиях давления приблизительно 25 бар и естественной циркуляции. В начале переходного процесса весь опускной канал и нижняя камера смешения реактора заполняется чистой водой (40.2 м³). Анализ был выполнен на

расчетной модели активной зоны Apros 3D, которая содержит собственный канал потока и 10 расчетных элементов для каждой чехловой кассеты. Кроме того, была использована модель сечения в большом масштабе, и сечения рассчитывались в CASMO-4E. Результаты показывают, что реактор ВВЭР-440 может выдержать образование даже больших пробок с чистой водой без повреждения топлива в условиях естественной циркуляции из-за отрицательной обратной связи по реактивности и противотоков во время переходного процесса. Несмотря на кратковременное интенсивное кипение и кризисы теплообмена, это указывает непосредственно на отказы оболочек ТВЭЛОВ

The integral non-stationary equations of neutron transport for calculations of nuclear reactors kinetics using Monte-Carlo method

S21

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Integral'nye nestatsionarnye uravneniya perenosa nejtronov dlya raschetov kinetiki yadernykh reaktorov metodom Monte-Karlo

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WWER TYPE REACTORS; INTEGRAL EQUATIONS; MONTE CARLO METHOD;
NEUTRON FLUX; DELAYED NEUTRONS; FORM FACTORS; ADIABATIC
APPROXIMATION; K CODES; THERMAL HYDRAULICS; REACTOR SAFETY;
REACTOR CORES

On the basis of the obtained integral equations, some aspects of the calculation technique in various approximations of fast transient processes in nuclear reactors are considered. An integral nonstationary equation for the neutron flux with delayed neutrons and integral equations for the form function in the improved quasistatic, quasistatic and adiabatic approximations are given. Approaches to the solution of these equations by the Monte Carlo method are described. The equations are implemented in the programs of KIR and KIR-P. The first one allows solving the integral equation of neutron transport without any approximations, and the second one, using the above approximations. In the latter case, the parameters of point kinetics are also calculated by

the Monte Carlo method, taking into account the function of the neutron value of fission. It is supposed to combine these programs with the computational codes of thermal and hydraulic and thermomechanical calculations of WWER to simulate the dynamic processes taking place in the active zones of these reactors

На основе полученных интегральных уравнений рассмотрены некоторые аспекты методики расчета в различных приближениях быстро протекающих переходных процессов в ядерных реакторах. Приводятся интегральное нестационарное уравнение для потока нейтронов с учетом запаздывающих нейтронов и интегральные уравнения для форм-функции в усовершенствованном квазистатическом, квазистатическом и адиабатическом приближениях. Описываются подходы к решению этих уравнений методом Монте-Карло. Уравнения реализованы в программах КИР и КИР-П. Первая позволяет решать интегральные уравнения переноса нейтронов без каких-либо приближений, а вторая - с использованием перечисленных выше приближений. В последнем случае параметры точечной кинетики рассчитываются тоже методом Монте-Карло с учетом функции ценности нейтронов деления. Предполагается объединить эти программы с вычислительными кодами теплогидравлических и термомеханических расчетов ВВЭР с целью моделирования динамических процессов, протекающих в активных зонах этих реакторов

The implementation of sensitivity analysis for Hanhikivi NPP severe accident calculation

S21

Astaf'eva, V.O.; Bezlepkin, V.V.; Semashko, S.E.; Sidorov, V.G. (AO ATOMPROEKT, Moscow (RU))

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Vypolnenie analiza chuvstvitel'nosti raschetov tyazhelykh avarij dlya AEhS #Left Double Quotation Mark#Khanhikivi#Right Double Quotation Mark#

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NUCLEAR POWER PLANTS; REACTOR ACCIDENTS; SENSITIVITY ANALYSIS;
COMPUTER CALCULATIONS; WWER TYPE REACTORS; FINLAND; DETERMINISTIC
ESTIMATION; RISK ASSESSMENT

Outlines the methodological approaches that are used to perform the sensitivity analysis of the calculation of severe accidents (SA) in the Hanhikivi NPP project, and the preliminary results of such an analysis. Severe accident scenarios for the analysis and equipment in operation were selected taking into account the requirements of Fennovoima based on the configuration of LAES-2 NPP, which is a reference design for Hanhikivi NPP

Излагаются методологические подходы, которые используются для выполнения анализа чувствительности расчетов тяжелых аварий (ТА) в проекте АЭС #Left Double Quotation Mark#Ханхикиви#Right Double Quotation Mark#, и предварительные результаты выполнения такого анализа. Выбор сценариев ТА для анализа и состава работающего оборудования выполнен с учетом требований Fennovoima на основе конфигурации референтного к проекту #Left Double Quotation Mark#Ханхикиви#Right Double Quotation Mark#-1 проекта ЛАЭС-2

The TDMCC code verification

S21

Semenova, T.V.; Grebennikov, A.N.; Zhitnik, A.K.; Ognev, S.P.; Artem'eva, E.V.; Volkov, N.A.; Lomteva, K.S. (FGUP RFYaTs-VNIIEhF, Sarov (RU)); Antonov, S.N.; Tikhomirov, A.V. (OKB GIDROPRESS, Podol'sk (RU))

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Verifikatsiya programmy TDMCC

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WWER TYPE REACTORS; REACTOR SAFETY; VERIFICATION; MONTE CARLO
METHOD; COMPUTER CODES; RADIATION PROTECTION; NUCLEAR FUELS;
BENCHMARKS

TDMCC (Time Dependent Monte Carlo Code) has been developed at the Russian Federal Nuclear Center - VNIIEF. This is a multiple-purpose code for the simulation of the problems of criticality and nuclear safety of individual fuel assemblies and reactor cores, evaluation of radiation safety, simulation of radiation shielding, selection of parameters to obtain the preset value, as well as for the fuel burnup calculations and simulation of fuel cycles in reactors. A brief description of the capabilities of TDMCC, the calculation schemes and models used, the technology for conducting verification calculations and verification results of the program to substantiate nuclear safety during the storage and transportation of WWER fuel are given. A variety of heterogeneous and homogeneous configurations with uranium, plutonium, and MOX fuels with different combinations of heavy elements and their isotopes were considered. Totally, 950 experimental tests have been performed for the TDMCC verification

TDMCC (Time Dependent Monte Carlo Code) - программа, разработанная в Российском Федеральном Ядерном Центре - ВНИИЭФ. Это многоцелевая программа, с помощью которой можно решать задачи оценки критичности и ядерной безопасности отдельных сборок и активных зон реакторных установок, оценивать радиационную безопасность, рассчитывать защиту от излучений, подбирать параметры для получения заданного значения, а также проводить расчеты выгорания топлива и моделировать топливные циклы реакторных установок. Приводится краткое описание возможностей TDMCC, используемых расчетных схем и моделей, технологии проведения верификационных расчетов и результатов верификации программы для обоснования ядерной безопасности при хранении и транспортировке топлива ВВЭР. Рассматривались разнообразные гетерогенные и гомогенные конфигурации с урановым, плутониевым и MOX-топливом в различных сочетаниях тяжелых элементов и их изотопов. Всего для верификации программы TDMCC было использовано более 950 экспериментальных тестов

Procedure for implementing CFD module in the KORSAR computer code S21

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Tekhnologiya adaptatsii CFD modulya v sostave funktsional'nogo napolneniya RK KORSAR/GP

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WWER TYPE REACTORS; REACTOR SAFETY; COMPUTERIZED SIMULATION;
THERMAL HYDRAULICS; REACTOR CORES; REACTOR KINETICS; ALGORITHMS;
POISSON EQUATION; FLUID MECHANICS; ONE-DIMENSIONAL CALCULATIONS;
THREE-DIMENSIONAL CALCULATIONS

The Russian KORSAR/GP computer code was developed by a joint team from Alexandrov NITI and OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark# for WWER safety analysis. The KORSAR/GP code includes a 3D reactor kinetics module. The code capability is intended for simulation of thermal-hydraulic phenomena in circulation circuits of reactor systems and also for 3D modeling of coupled thermal-hydraulic (complete-core approximation) and neutronic processes in the reactor core. The algorithm is based on combining the matrices of the Poisson equation for the pressure on a new time layer of 1D and 3D modules. The discrete analogues of the Poisson equation in frontier cells are written from the condition of maintaining the mass balance at the interface. The combined pressure field is calculated by the multigrid method. Results of testing the software implementation of the algorithm

Российский компьютерный код КОРСАР разработан для обоснования безопасности реакторов типа ВВЭР совместно специалистами ФГУП НИТИ и ОКБ #Left Double Quotation Mark#ГИДРОПРЕСС#Right Double Quotation Mark#. Представлено описание алгоритма стыковки CFD модуля с одномерным модулем кода КОРСАР. Расчетный код КОРСАР/ГП содержит трехмерный модуль кинетики реакторов. Заложенные в коде возможности позволяют осуществлять моделирование теплогидравлических процессов в циркуляционных контурах реакторных установок, а также пространственное моделирование сопряженных теплогидравлических (в поканальном приближении) и нейтронно-физических процессов в активной зоне. Алгоритм базируется на объединении матриц уравнения Пуассона для давления на новом временном слое 1D и 3D модулей. Дискретные аналоги уравнения Пуассона в приграничных ячейках записываются из условия сохранения баланса массы на границе стыковки. Объединенное поле давления рассчитывается многосеточным методом. Приведены результаты тестирования программной реализации алгоритма

Strain measurement analyses of NPP equipment

S21

Khajretdinov, V.U.; Malyshev, R.Yu.; Badanova, M.V. (AO OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark#, Podol'sk (RU))

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AEhS s VVEhR

Tenzometricheskie issledovaniya oborudovaniya AEhS

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Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; STRAIN GAGES; COMPARATIVE EVALUATIONS; REACTOR START-UP; REACTOR OPERATION; REACTOR SAFETY; PRIMARY COOLANT CIRCUITS; STRAINS; TEMPERATURE CONTROL

The tools of thermal-mechanical (strain measurement) inspection consisting of startup and adjustment measurement system (SAMS) are intended for determination of temperature and stress-strain state of the primary equipment and also thermal movements of equipment components due to RP self-compensation. Comparative tests of various strain gauges (traditional wire strain gages NMT-450, IMASH / #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark#, semiconductor strain transducers of the TPF deformations of 21 Moscow firms #Left Double Quotation Mark#SENSOR#Right Double Quotation Mark#, as well as high-tech strain gauges of the LZN series of the American company Vishay Micro-Measurements) are carried out. A comparison is made between the metrological and technological characteristics and properties declared by the manufacturer. The choice of the optimal design of the measuring points for static and dynamic strain gauging in the structure of SAMS has been worked out

Средства термомеханического (тензометрического) контроля в составе систем пусканаладоочных измерений (СПНИ) предназначены для определения температурного и напряженно-деформированного состояния оборудования первого контура, а также тепловых перемещений элементов оборудования от самокомпенсации РУ. Проведены сравнительные испытания различных тензодатчиков (традиционные проволочные тензорезисторы НМТ-450, ИМАШ/#Left Double Quotation Mark#ГИДРОПРЕСС#Right Double Quotation Mark#, полупроводниковые тензопреобразователи деформаций ТПФ 21 московской фирмы #Left Double Quotation Mark#СЕНСОР#Right Double Quotation Mark#, а также высокотехнологичные тензометры серии LZN американской компании Vishay Micro-Measurements). Выполнено сопоставление заявленных производителем метрологических и технологических характеристик и свойств. Проработан выбор оптимального конструктивного исполнения измерительных точек статического и динамического тензометрирования в составе СПНИ

Results of design analyses of in-vessel corium confinement for NPP with WWER-600 RP using realistic approaches

S21

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Rezultaty raschetnykh analizov vnutrikorpusnogo uderzhaniya rasplava dlya AEhS s RU VVEhR-600 s ispol'zovaniem realisticheskikh podkhodov

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27 refs., 12 tabs., 2 tabs.

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WWER TYPE REACTORS; CONTAINMENT SYSTEMS; CORIUM; REACTOR ACCIDENTS; REACTOR SAFETY; REACTOR CORES; FLUID MECHANICS; S CODES

The #Left Double Quotation Mark#Corium confinement and reactor vessel cooling system#Right Double Quotation Mark# (SUROK) is used for implementation of the concept of corium confinement inside the reactor vessel in WWER-600 RP design. SUROK helps to control severe beyond design basis accidents by flooding the reactor cavity with water arranging its circulation around its vessel in order to ensure corium confinement inside the reactor vessel and to prevent its through damage. The results of the refined calculation analysis of the core melt retention process in the reactor vessel are presented using realistic approaches using the certified code SOKRAT/B1

Для реализации концепции удержания расплава внутри корпуса реактора в проекте РУ ВВЭР-600 применяется #Left Double Quotation Mark#Система удержания расплава и охлаждения корпуса реактора#Right Double Quotation Mark# (СУРОК). С использованием СУРОК реализуется управление тяжелой запроектной аварией посредством залива водой шахты реактора с организацией циркуляции вокруг его корпуса с целью обеспечения удержания расплава активной зоны внутри корпуса реактора и предотвращения его сквозного повреждения. Представлены результаты уточненного расчетного анализа процесса удержания расплава активной зоны в корпусе реактора с использованием реалистических подходов с применением аттестованного кода РК СОКРАТ/В1

Inspection features of thermomechanical loading of RP equipment at putting into operation of Unit 1 of #Left Double Quotation Mark#Kudankulam#Right Double Quotation Mark# NPP

S21

Khajretdinov, V.U.; Mamontov, S.V.; Tupikov, R.A. (AO OKB #Left Double Quotation

Mark#GIDROPRESS#Right Double Quotation Mark#, Podol'sk (RU))

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Osobennosti kontrolya termomekhanicheskoy nagruzhennosti oborudovaniya RU pri vvode v ehkspluatatsiyu ehnergobloka nom. 1 AEhS #Left Double Quotation Mark#Kudankulam#Right Double Quotation Mark#

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REACTOR OPERATION; REACTOR START-UP; REACTOR COOLING SYSTEMS; REACTOR CONTROL SYSTEMS; REACTOR SAFETY; KUDANKULAM-1 REACTOR

During putting into operation of unit 1 of #Left Double Quotation Mark#Kudankulam#Right Double Quotation Mark# the RP equipment was subject to inspection of temperature and stress-strain state both of traditional composition: main coolant pipeline (MCP), steam generators (SG), pressurizing system (PRZ), reactor upper unit (UU), and of the systems implemented for the first time: passive heat removal system (PHRS), quick born injection system (QBIS). The results of traditional thermomechanical inspection of startup and adjustment measurement system (SAMS), in general, confirmed the correspondence of actual operating conditions of the inspected equipment to the design requirements and acceptability of its thermomechanical loading. At the same time, a number of deviations from the design requirements and particularities were detected by the operating conditions of some equipment joints. Test results of PHRS with SAMS application confirmed accuracy of design decisions. As a result of the tests, measures were developed to increase the efficiency of the PHRS and optimize the operating conditions of the equipment in the modes of heating and cooling

При вводе в эксплуатацию энергоблока ном. 1 АЭС #Left Double Quotation Mark#Куданкулам#Right Double Quotation Mark# контролю температурного и напряженно-деформированного состояния подвергалось оборудование РУ как в традиционном составе: главный циркуляционный трубопровод (ГЦТ), парогенераторы (ПГ), система компенсации давления (КД), верхний блок (ВБ) реактора, так и впервые реализованных систем: система пассивного отвода тепла (СПОТ), система быстрого ввода бора (СБВБ). Результаты традиционного термомеханического контроля систем пуска-наладочных измерений (СПНИ), в основном, подтвердили соответствие фактических условий эксплуатации контролируемого оборудования требованиям проекта и приемлемость его термомеханической нагруженности. Одновременно зафиксирован ряд

отклонений от требований проекта и особенностей по условиям эксплуатации некоторых узлов оборудования. Результаты испытаний СПОТ с применением СПНИ подтвердили правильность проектно-конструкторских решений. По результатам испытаний были разработаны мероприятия по повышению эффективности СПОТ и оптимизации условий эксплуатации оборудования в режимах разогрева и расхолаживания

Special features of thermal and warm-worked behaviour of WWER vessel at severe accidents under conditions of formation of inversely-stratified melt pool S21

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Osobennosti teplovogo i termodeformatsionnogo povedeniya korpusa VVEhR pri tyazhelej avarii v usloviyakh formirovaniya inversno stratifitsirovannoj vannы rasplava

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WWER TYPE REACTORS; COMPUTERIZED SIMULATION; MELTDOWN; DEFORMATION; TEMPERATURE RANGE 0400-1000 K; FAILURES; REACTOR SAFETY

The results of numerical simulation of the thermal and thermal strength behavior of the design of the medium-power WWER reactor case in the conditions of severe accident (SA) are presented and discussed when a melt pool having an inverse stratified structure is formed in the lower part of the vessel. The thermal state of the melt pool is modeled considering natural convection in the melt for various values of residual heat and dimensions of the melt pool. As a result of computational calculations the value and character of heat load distribution was determined, which affect the reactor vessel wall from the melt side at various conditions of SA, and which were used in subsequent modeling of thermal and thermal-mechanical behavior of WWER vessel structure. The reactor vessel thermal state was modeled considering the effects of the vessel wall melting at various conditions of its cooling on the external surface. The vessel warm-worked state and estimation of its load-carrying ability was determined on the basis of computational modeling considering high-temperature creep of the vessel material and its

damage ability under deformation. The analysis is carried out and estimates of the influence of each of the considered parameters on the load-carrying capacity of the WWER case and the characteristic time of its survivability under various conditions of SA

Представлены и обсуждаются результаты численного моделирования теплового и термопрочностного поведения конструкции корпуса РУ ВВЭР средней мощности в условиях тяжелой аварии (ТА), когда в нижней части корпуса формируется бассейн расплава, имеющий инверсную стратифицированную структуру. Тепловое состояние бассейна расплава моделируется с учетом естественной конвекции в расплаве для различных величин остаточного энерговыделения и размеров ванны расплава. В результате выполнения численных расчетов определялись величина и характер распределения тепловой нагрузки, действующей на стенку корпуса реактора со стороны расплава при различных условиях ТА, и которые использовались в последующем при моделировании теплового и термомеханического поведения конструкции корпуса ВВЭР. Тепловое состояние корпуса реактора моделировалось с учетом эффектов плавления стенки корпуса при различных условиях ее охлаждения на внешней поверхности. Термодформационное состояние корпуса и оценка его несущей способности определялись на основе численного моделирования с учетом высокотемпературной ползучести материала корпуса и его повреждаемости в процессе деформирования. Проведен анализ и получены оценки влияния каждого из рассматриваемых параметров на несущую способность корпуса ВВЭР и характерное время его живучести при различных условиях протекания ТА

Vibrodynamical tests of RP equipment with application of imitation area of WWER-1000 reactor

S21

Khajretdinov, V.U.; Tarkhanov, V.V.; Rodionova, I.N. (AO OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark#, Podol'sk (RU))

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Vibrodinamicheskie ispytaniya oborudovaniya RU s primeneniem imitatsionnoj zony reaktora VVEhR-1000

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WWER TYPE REACTORS; REACTOR SAFETY; REACTOR OPERATION; REACTOR START-UP; REACTOR ACCIDENT SIMULATION; FUEL ASSEMBLIES; THERMAL HYDRAULICS; MECHANICAL VIBRATIONS

Performance of preoperational tests and measurements with application of imitation area of the reactor is a distinctive characteristic of putting into operation of NPP Units with WWER-1000/1200. The imitation area consists of 163 full-scale FA models, where fuel matrixes made of nuclear-fissionable material, are replaced by leaden simulators. Vibrodynamic tests involve inspection of hydrodynamic disturbances in the primary circuit (dynamic impact on the inspected elements), characteristics of vibration response of the main equipment stress-deformed state of bearing structure, and also parameters of moving and geometry of the inspected objects (boundary conditions at process simulation). Preoperational tests and measurements on the simulated area of WWER-1000/1200 are obligatory and performed at every unit of NPP of this type

Выполнение пусконаладочных испытаний и измерений с использованием имитационной зоны реактора является отличительной особенностью ввода в эксплуатацию энергоблоков АЭС с ВВЭР-1000/1200. Имитационная зона состоит из 163 полномасштабных макетов ТВС, в которых топливные матрицы из ядерноделящегося материала заменены свинцовыми имитаторами. Вибродинамические исследования включают контроль гидродинамических возмущений в первом контуре (динамические воздействия на контролируемые элементы), характеристики вибрационного отклика основного оборудования, напряженно-деформированное состояние несущих конструкций, а также параметры движения и геометрии объектов контроля (граничные условия при моделировании процессов). Предэксплуатационные испытания и измерения на имитационной зоне ВВЭР-1000/1200 являются обязательными и проводятся на каждом блоке АЭС данного типа

Vibration examination of CPS drives of the reactor plant WWER-1000

S21

Khajretdinov, V.U.; Vakhrushev, P.A.; Saratov, A.Yu. (AO OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark#, Podol'sk (RU))

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Vibroissledovanie privodov SUZ reaktornoj ustanovki VVEhR-1000

vp.

(RU)

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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bezopasnosti AEhS s VVEhR. Nauchno-tehnicheskoe ehlektronnoe izdanie
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2015

vp.

WWER TYPE REACTORS; REACTOR CONTROL SYSTEMS; COMPUTERIZED
SIMULATION; REACTOR OPERATION; REACTOR SAFETY; MECHANICAL
VIBRATIONS; BENCH-SCALE EXPERIMENTS; EXPERIMENT PLANNING

In order to provide vibration resistance of CPS drive housing, the bench modal analyses were performed and also in-service vibration-loading assessments were obtained with subsequent simulation and diagnosis of different vibration conditions. Mathematical simulation of CPS housing in ANSYS software made it possible to gain idea about its own forms and frequencies of the given design in a clear view, convenient for further processing. Comparison of the experimental results with the mathematical model confirms the effectiveness and representativeness of the applied method of full-scale vibro-impact studies. The method is compact and convenient to carry out measurements directly on site, which is confirmed by the results of tests at the 3rd power unit of Rostov NPP

В обеспечение анализа вибростойкости чехла привода СУЗ проведены стендовые модальные исследования, а также получены оценки эксплуатационной виброн нагруженности с последующим моделированием и диагностированием различных вибросостояний. Математическое моделирование чехла СУЗ в программной среде ANSYS позволило получить представление о собственных формах и частотах данной конструкции в наглядном, удобном для дальнейшей обработки виде. Сравнение экспериментальных результатов с математической моделью подтверждают эффективность и представительность применяемого метода натуральных виброударных исследований. Метод отличается компактностью и удобством проведения измерений непосредственно на месте эксплуатации конструкции, что подтверждено результатами испытаний на 3-м энергоблоке Ростовской АЭС

Systems of preoperational measurements of new design versions of NPP with WWER

S21

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Sistemy puskonaladochnykh izmerenij novykh proektnykh variantov YaEhU s VVEhR

vp.

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International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomenergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomenergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; NUCLEAR POWER PLANTS; REACTOR START-UP;
REACTOR OPERATION; REACTOR CONTROL SYSTEMS; DEFORMATION; STRAIN
GAGES; OPTIMIZATION; ALGORITHMS; ON-LINE CONTROL SYSTEMS

The features SAMS new design NPP embodiment WWER (#Left Double Quotation Mark#NPP-2006#Right Double Quotation Mark#, #Left Double Quotation Mark#WWER-TOI#Right Double Quotation Mark# et al.) are the availability of new, previously applied SAMS systems (control thermal mechanical and hydraulic SG characteristics), accurate test volume, composition and zone controlled equipment SAMS conventional systems, the application of new methods and means for measurement (semiconductor sensor of pressure pulsations and dynamic deformations autonomous measuring thermoelectric modules and strain measurement al.). Optimized to improve reliability layout of a distributed system of data collection, modernized SHC with updated software, including the use of on-line algorithms inverse heat conduction problems and thermoelasticity and dynamic analysis of time series as well as the combined use SAMS and MCDS for basic background information diagnostic monitoring equipment NPP provided during the work on the justification strength, durability and reliability of bearing structural elements in the main equipment at all stages of the life cycle

Особенностями СПНИ новых проектных вариантов ЯЭУ с ВВЭР (#Left Double Quotation Mark#АЭС-2006#Right Double Quotation Mark#, #Left Double Quotation Mark#ВВЭР-ТОИ#Right Double Quotation Mark# и др.) являются наличие новых, ранее не применявшихся систем СПНИ (контроль тепломеханических и гидравлических характеристик ПГ), уточнение объема испытаний, состава и зон контролируемого оборудования в традиционных системах СПНИ, применение новых методов и средств измерений (полупроводниковые датчики пульсаций давления и динамических деформаций, автономные измерительные модули термо-тензометрирования и др.). Оптимизированная для повышения надежности компоновка распределенной системы сбора данных, модернизированные ПТК с обновленным ПО, включая применение on-line алгоритмов обратных задач теплопроводности и термоупругости, динамического анализа временных рядов, а также совместное применение СПНИ и СКУД для получения базовой исходной информации диагностического контроля оборудования ЯЭУ предусмотрены при проведении работ по обоснованию прочности, долговечности и надежности несущих элементов конструкций основного оборудования на всех этапах жизненного цикла

Passive sprinkler system of AES 2006 containment

S21

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Sprinklernaya sistema lokalizuyushchej sistemy bezopasnosti AEhS 2006 passivnogo tipa vp.

(RU)

3 refs., 1 fig.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; REACTOR SAFETY; LOSS OF COOLANT; CONTAINMENT SPRAY SYSTEMS; WATER; SPRAY COOLING

Sprinkler system is provided to diminish temperature and pressure of steam in the containment when LOCAs. Cold water drops that are produced by sprinklers cause said diminishing. Important role of the system is radioactive substances washing out the gas phase and fixing in water. The former is enriched by chemicals for this purpose. A risk of pumps failure or feeding pipes damage lowers the reliability of the system and of the containment. The passive sprinkler system is suggested to be incorporated in the staff one with the use of the sprinkler collector at +60 m point. The collector is equipped with the lower feeding pressure sprinklers. The available room and the heights are enough to place the passive system and to provide feeding level assuming fittings hydraulic resistance as calculated. Safety grade is 4 (severe accident control system), seismic grade is 1 for the passive system.

Спринклерные системы охлаждения герметичной оболочки локализирующей системы безопасности (ГО ЛСБ) АЭС предназначены для снижения температуры и давления водяного пара, который может поступать в помещение при разгерметизации различного оборудования (трубопроводов, парогенераторов и т.п.). Для этого в пар впрыскивают капли холодной воды, которые производят форсунки. Важной функцией спринклерной системы является сбор подвижных радиоактивных субстанций и их фиксация в воде, для чего в воду вводят специальные реагенты. Возможности отказа насосов или повреждение питающих трубопроводов снижают надежность устройства. Предлагается встраивать пассивную спринклерную систему в штатную с использованием коллектора на отметке +60. Коллектор комплектуется форсунками с уменьшенным давлением питания. Расчетами установлена достаточность располагаемых высот для обеспечения гидростатического напора с учетом требуемого уровня рабочего расхода жидкости и гидравлического сопротивления пассивной системы. Класс устройства по безопасности определен как 4 (средство управления запроектной аварией (ЗПА)), по сейсмостойкости класс 1

Researches of interassembly interaction of coolant between neighbouring FA of WWER-1000 reactor core of atomic power station #Left Double Quotation Mark#Temelin#Right Double Quotation Mark#

S21

Borodina, V.E.; Dmitriev, S.M.; Dobrov, A.A.; Doronkov, D.V.; Polozkova, E.N.; Pronin, A.N.; Khrobostov, A.E. (Nizhegorodskij Gosudarstvennyj Tehnicheskij Univ. im. R.E. Alekseeva, Nizhnij Novgorod (RU))

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Issledovaniya mezhkassetnogo vzaimodejstviya teplonositelya mezhdu sosednimi TVSA aktivnoj zony reaktora VVEhR-1000 AEhS #Left Double Quotation Mark#Temelin#Right Double Quotation Mark#

vp.

(RU)

6 refs., 16 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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TEMELIN-1 REACTOR; REACTOR SAFETY; REACTOR CORES; FUEL ASSEMBLIES; COOLANTS; MASS TRANSFER; OPTIMIZATION

On the first stage researches of local hydrodynamic characteristics of coolant flow have been conducted into a bunch of fuel rods of model. These researches consisted in measurement of velocity vector module, total pressure and static pressure. Measurement have been carried out by means of fivechannel pneumatic probe. Velocity vector was measured in characteristics zones both of FA and of interassembly gap of a core WWER-1000. On the second stage researches of mass-transfer of coolant have been carried out using the admixture diffusion method (the tracer-gas method). The tracer have been injected in characteristic cells of experimental model and than redistribution of tracer in cross-section and lengthwise of model have been monitored. The obtained experimental information has been generalized, and a data bank for verifying CDF codes and computer programs for cell-wise calculations for reactor cores with FA has been established. Investigations results are used to specify local hydrodynamics and mass transfer characteristics of coolant flow when assessing heat engineering reliability of WWER reactor core

На первом этапе проводились исследования локальных гидродинамических характеристик потока теплоносителя внутри трубного пучка модели, заключающиеся в измерении модуля вектора скорости, полного и статического давлений. Измерения проводились пятиканальным пневмометрическим зондом. Вектор скорости измерялся в характерных зонах как ТВСА, так и межкассетного зазора активной зоны реактора ВВЭР-1000. На втором этапе проводились исследования массообмена теплоносителя с использованием метода диффузии трассера. Трассер подавался в характерные ячейки экспериментальной модели, и далее отслеживалось перераспределение трассера в поперечном сечении модели и по ее длине. Полученные сведения позволили выявить особенности межкассетного взаимодействия теплоносителя в активной зоне реактора ВВЭР-1000. Результаты могут быть использованы в качестве базы экспериментальных данных для верификации CFD-кодов и программ детального поэлементного расчета активных зон реакторов ВВЭР с целью уменьшения консерватизма при обосновании надежности установок данного типа

Experiments with mixing of helium as hydrogen simulant with steam-gas atmosphere in containment of the KMS large-scale test facility

S21

Migrov, Yu.A.; Efimov, V.K.; Zasukha, V.K.; Gorshkov, A.I. (FGUP #Left Double Quotation Mark#NITI im. A.P. Aleksandrova#Right Double Quotation Mark#, Sosnovyj Bor (RU)); Bezlepkin, V.V.; Semashko, S.E.; Krylov, Yu.V. (AO #Left Double Quotation Mark#ATOMPROEKT#Right Double Quotation Mark#, Sankt-Peterburg (RU))

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Ehksperimental'nye issledovaniya protsessov peremeshivaniya geliya (kak imitatora vodoroda) v parogazovoj srede kontejnmenta krupnomasshtabnogo stenda KMS

vp.

(RU)

8 refs., 15 figs., 2 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; HELIUM; CONTAINMENT SYSTEMS EXPERIMENT;
REACTOR SAFETY; CONTAINMENT; BENCHMARKS; THERMAL HYDRAULICS;
REACTOR ACCIDENT SIMULATION

The experiment pressures inside the containment were up to 0.5 kPa (abs) and the experiment temperatures of the steam-gas mixture were up to 150#Degree Celsius#. Helium was injected in the following containment areas: - the bottom head, at steam injection point; - the dome space. Analysis of results demonstrates that in all experiment conditions the helium concentration eventually becomes uniform over the containment areas. The time taken to reach a uniform concentration profile depends on whether convection or diffusion prevails. Therefore, no conditions for local buildup of helium to explosive-for-hydrogen concentrations exist

Исследования проводились при давлении внутри контейнента стенда до 0,5кПа (абс) и температуре парогазовой среды до 150 #Degree Celsius# при различных координатах точки подачи гелия: - в нижнее помещение контейнента, в точке подачи пара; - в подкупольное пространство контейнента. Полученные результаты позволяют сделать вывод, что при различных условиях проведения экспериментальных режимов в конечном итоге происходит постепенное (с различной скоростью в зависимости от преобладания конвективных или диффузионных процессов) выравнивание концентрации гелия по всем помещениям контейнента стенда, что свидетельствует об отсутствии условий для образования повышенных концентраций гелия (взрывоопасных для водорода) в отдельных полостях и в помещениях контейнента стенда

Improving the efficiency of computational and experimental evaluation of seismic resistance of NPP equipment with the use of a database of seismic attestation

S21

Saakov, Eh.S.; Ryasnyj, S.I.; Zajkin, I.I.; Kas'yanov, K.G.; Kaznovskij, P.S.; Kaznovskij, A.P.; Shchugorev, A.V. (AO #Left Double Quotation Mark#Atomtekhehnergo#Right Double Quotation Mark#, Mytishchi, Moskovskaya Obl. (RU))

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Povyshenie ehffektivnosti raschetno-ehksperimental'noj otsenki sejsmostojkosti oborudovaniya AEhS s primeneniem bazy dannykh sejsmicheskoy attestatsii

vp.

(RU)

16 refs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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vp.

WWER TYPE REACTORS; NUCLEAR POWER PLANTS; RISK ASSESSMENT;
EARTHQUAKES; REACTOR SAFETY; DATA ANALYSIS; COMPUTER
CALCULATIONS; SAFETY STANDARDS; MULTI-PARAMETER ANALYSIS

Increasing the efficiency of using the results of a computational and experimental assessment of the seismic resistance of nuclear power plant equipment can be achieved using the seismic certification database as a tool to reduce the list of equipment subject to dynamic testing, reduce the amount of calculation work, identify and use statistical regularities inherent in the characteristics of the equipment's own oscillations under conditions of its actual state. A set of parameters for inclusion in the Database of seismic certification of equipment, developed on the basis of experience and results of previous work, was determined. The possibility of using the Database ... to evaluate the seismic resistance of Russian NPP equipment by indirect methods is estimated. A set of parameters is determined when using the developed database for the purposes of systematization and generalization of the results. The urgency of the refinement of the influence of the level of the experimental influence on the estimation of damping of oscillations under conditions of real seismic action using the methods of classical modal analysis

Повышение эффективности использования результатов расчетно-экспериментальной оценки сейсмостойкости оборудования АЭС может достигаться с применением базы данных сейсмической Аттестации в качестве инструмента для сокращения перечня оборудования, подлежащего динамическим испытаниям, сокращения объемов расчетных работ, выявления и практического использования статистических закономерностей, присущих характеристикам собственных колебаний оборудования в условиях его фактического состояния. Определен набор параметров для включения в Базу данных сейсмической аттестации оборудования, разрабатываемую на основе опыта и результатов предшествующих работ. Оценена возможность использования Базы данных... для оценки сейсмостойкости оборудования российских АЭС косвенными методами. Определен набор параметров при использовании разрабатываемой базы данных для целей систематизации и обобщения результатов. Определена актуальность уточнения влияния уровня экспериментального воздействия на оценку затухания колебаний в условиях реального сейсмического воздействия с применением методов классического модального анализа

Technologies and procedures of NPP's buildings, structures and equipment seismic margin assessment

S21

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Tekhnologiya i protsedury otsenki zapasa sejsmostojkosti zdaniy, sooruzhenij i oborudovaniya AЕhS

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8 refs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; NUCLEAR POWER PLANTS; SEISMICITY; RISK ASSESSMENT; SAFETY ANALYSIS; PROBABILISTIC ESTIMATION; REACTOR SAFETY; RECOMMENDATIONS; SAFETY MARGINS

Features of the application of Seismic Margin Assessment (SMA) technology and procedures at Russian power units are: - the inability to apply US screening criteria; - the impossibility of estimating the values #Zero Width Space##Zero Width Space#characterizing the high reliability of the low probability of failure (HCLPF) by comparing the boundary spectra of the seismic resistance with response spectra for seismic effects on structures, systems and elements (CSE). To ensure the possibility of using SMA technology and procedures on Russian power units, it is required to create its own seismic equipment certification database. Estimation of seismic stability of CSE at Russian NPPs prior to the creation of a database and the development of criteria for screening and boundary spectra based on it can be carried out only by the calculation and experimental method

Особенностями применения технологии и процедур Seismic Margin Assessment (SMA) на российских энергоблоках являются: - невозможность применения американских критериев скрининга; - невозможность оценки значений, характеризующих высокую достоверность низкой вероятности отказа (HCLPF) путем сопоставления граничных спектров сейсмического сопротивления со спектрами ответа на сейсмические воздействия на конструкции, системы и элементы (КСЭ). Для обеспечения возможности применения технологии и процедур SMA на российских энергоблоках требуется создание собственной базы сейсмической аттестации оборудования. Оценка сейсмостойкости КСЭ на российских АЭС до создания базы данных и разработки на ее основе критериев скрининга и граничных спектров может проводиться только расчетно-экспериментальным методом

A water-tube corium catcher

S21

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WWER TYPE REACTORS; NUCLEAR POWER PLANTS; MELTDOWN; REACTOR CORES; LOSS OF COOLANT; OPTIMIZATION; REACTOR SAFETY; HEAT TRANSFER; TUBES; WATER

Experience gained from operation of power-generating reactors shows that, despite the availability of well-developed safety systems, loss of reactor core cooling still may occur, followed by buildup of core temperature due to decay heat release, which leads to melting the fuel assemblies, since not each and every negative factor can be taken in consideration in the design. In severe BDBAs involving loss of coolant, the reactor core is destructed and melt. As a result, the molten elements of the reactor internal structures and core shift to the RPV bottom, so that high-temperature corium is formed in the RPV lower part. It presents the design of a water-tube device for confining the melt (water-tube corium catcher, WTCC) for WWER-1000 reactors, which incorporates design measures for limiting the impact of the above-mentioned effects. It should be specially pointed out that the WTCC is accommodated in the limited volume of reactor cavity and does not require redesigning of the premise beneath the reactor. This makes it possible to carry out modernization and enhance the safety level of existing NPPs equipped with WWER-1000 reactors

Опыт эксплуатации энергетических реакторов показывает, что, несмотря на развитые системы безопасности, в силу невозможности учесть при проектировании негативных факторов, происходит прекращение охлаждения активной зоны реактора и последующий ее разогрев за счет остаточного энерговыделения, который приводит к расплавлению ТВС. В тяжелых ЗПА с потерей теплоносителя происходит разрушение и расплавление активной зоны. Вследствие этого расплавленные элементы конструкции реактора и активной зоны РУ перемещаются на днище корпуса реактора, что приводит к формированию в нижней части корпуса реактора высокотемпературного кориума. Представлена конструкция водотрубного устройства локализации расплава (ВДУ) для ВВЭР-1000, в котором предусмотрены конструктивные меры для ограничения воздействия указанных выше эффектов. Следует особо отметить, что размещение ВДУ производится в ограниченном объеме шахты реактора и не требует реконструкции подреакторного помещения. Это позволяет провести модернизацию и повысить безопасность существующих АЭС с ВВЭР-1000

Modeling the RPV high-temperature creep processes under the conditions of a beyond-design-basis accident in a WWER-based reactor plant

S21

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Modelirovanie protsessov vysokotemperaturnoj polzuchesti korpusa reaktora pri zaproektnoj avarii RU VVEhR

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International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; COMPUTERIZED SIMULATION; TEMPERATURE RANGE 1000-4000 K; CREEP; DEFORMATION; REACTOR SAFETY; REACTOR ACCIDENT SIMULATION; MELTDOWN; REACTOR VESSELS; HEAT TRANSFER; CORE CATCHERS

The project of an NPP equipped with WWER reactors incorporates a corium catcher (CC) as a means for control of beyond-design-basis accidents (BDBAs). The CC design includes a guiding plate, which is placed under the RPV bottom. The plate consists of a few cylindrical shell rings and 16 support ribs. The ribs are specially designed for holding the vessel bottom when it is detached and becomes plastically deformed. An analysis of the applicability of the creep models included in the software complex ANSYS is carried out. Based on these models, numerical calculations of the high-temperature creep processes in the metal of the reactor vessel are performed. The results obtained are used to analyze the effect of plastic deformation and the lowering of the body on the edges of the guide plate. Of special interest are the data obtained by varying the temperature boundary conditions, the excess pressure, and the geometry of the model

В проекте АЭС с реакторами ВВЭР для управления запроектными авариями (ЗПА) с плавлением активной зоны (АЗ) предусмотрено устройство локализации расплава (УЛР). Конструкция УЛР включает в себя направляющую плиту, которая располагается под днищем корпуса реактора. Плита состоит из нескольких цилиндрических обечаек и 16 опорных ребер. Ребра специально предназначены для удержания днища корпуса при его

отрыве и пластическом деформировании. Проведен анализ применимости моделей ползучести, включенных в программный комплекс ANSYS. На основе этих моделей выполнены численные расчеты процессов высокотемпературной ползучести в металле корпуса реактора. Полученные результаты используются для анализа влияния пластического деформирования и опускания корпуса на ребра направляющей плиты. Особый интерес представляют данные, полученные при варьировании температурных граничных условий, избыточного давления и геометрии модели

Substantiation of radiation safety under severe accident conditions in Leningradskaya NPP-2 design

S21

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Obosnovanie radiatsionnoj bezopasnosti v proekte Leningradskoj AEhS-2 pri tyazhelykh avariyakh

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LENINGRAD-2 REACTOR; REACTOR ACCIDENTS; REACTOR ACCIDENT SIMULATION; REACTOR SAFETY; PROGRAMMING; POPULATIONS; RADIATION EFFECTS; WEATHER; FORECASTING; RECOMBINATION; HYPOTHETICAL ACCIDENTS

In order to analyse radiological consequences of severe BDBA, based on meteorological data for the period of 2007-2013 obtained at NW FSHEM weather station situated on LNPP site, maximum emergency release dispersion/deposition factors of a probability not less than 95 % have been calculated for short distances with use of software #Left-Pointing Double Angle Quotation Mark#RiskZone v.1.0#Right-Pointing Double Angle Quotation Mark#. In order to estimate maximum possible radiation effect on population from the considered severe accident at

long distances software #Left-Pointing Double Angle Quotation Mark#RECASS NT#Right-Pointing Double Angle Quotation Mark# developed for NPP safety evaluation with respect to transboundary transfer has been used. Based on calculation results maximum nuclide concentrations, deposition and radiation exposure are determined for an hypothetical accident in 2013 at distances of 30-1000 km in 8 directions from LNPP-2. The calculation results confirm that dose criteria for severe accidents of the considered class are fulfilled, and LNPP-2 design complies with the international recommendations

Для анализа последствий тяжелой ЗПА по данным метеостанции СЗ УГМС на площадке строительства ЛАЭС-2 за период 2007-2013 гг. ПС #Left Double Quotation Mark#RiskZone v.1.0#Right Double Quotation Mark# выполнен расчет максимальных факторов разбавления/осаждения аварийного выброса в ближней зоне обеспеченности не менее 95 %. Программно-технический комплекс #Left Double Quotation Mark#RECASS NT#Right Double Quotation Mark#, предназначенный для оценки безопасности АЭС в трансграничном контексте, применен к исследованию максимально возможного радиационного воздействия на население при рассмотренной тяжелой аварии на дальних расстояниях. По результатам расчетов выполнен прогноз максимально возможных значений концентраций, выпадений и радиационного воздействия на население при аварии в период 2013 г. на расстояниях от 30 до 1000 км в 8 направлениях вокруг ЛАЭС-2. Результаты расчетов подтвердили выполнение дозовых критериев при тяжелых авариях данного класса и соответствие проекта ЛАЭС-2 международным рекомендациям

Boric acid mass transfer processes in emergency operation modes of NPP equipped with WWER reactors

S21

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BORIC ACID; MASS TRANSFER; WWER TYPE REACTORS; REACTOR SAFETY; CORE CATCHERS; CORIUM; MELTDOWN; ABUNDANCE

A number of models have been developed and implemented in software to describe the dynamics of boric acid concentration both in the vessel of the reactor in the emergency processes at the stage of shutdown cooling, and in the operation of the molten core catcher. The calculation of phase transitions-crystallization/dissolution has been realized, changes in the density of a solution of boric acid from the density of the solvent and concentration have been taken into account. The presence of a solid phase was taken into account, the positions of coolant levels in subregions were determined, and the dependence of the limiting concentration of boric acid solution on temperature was used. It is necessary to clarify (confirm) many characteristics of a solution of boric acid, including the thermophysical properties of a solution of boric acid at limiting concentrations: the heat of phase transition, the dependence of viscosity on concentration, the mechanics of solid phase formation in the studied processes. These works must be preceded by multidimensional calculations for emergency processes

Разработан ряд моделей и выполнена их реализация в программных средствах для описания динамики концентрации борной кислоты как в сосуде реактора в аварийных процессах на стадии расхолаживания, так и при функционировании устройства локализации расплава. Реализован расчет фазовых переходов - кристаллизация/растворение, учтены изменения плотности раствора борной кислоты от плотности растворителя и концентрации. Учтено наличие твердой фазы, определены положения уровней теплоносителя в подобластях и использованы зависимости предельной концентрации раствора борной кислоты от температуры. Следует уточнять (подтверждать) многие характеристики раствора борной кислоты, включая теплофизические свойства раствора борной кислоты при предельных концентрациях: теплота фазового перехода, зависимость вязкости от концентрации, механика образования твердой фазы в изучаемых процессах. Эти работы должны предшествовать проведению многомерных расчетов для аварийных процессов

The justification of AES-2006 NPS core catcher efficiency

S21

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WWER TYPE REACTORS; CORE CATCHERS; MELTDOWN; COMPUTER CALCULATIONS; HEAT TRANSFER; COOLING; REACTOR SAFETY; ECCS

To substantiate the efficiency of the molten core catcher, a calculation analysis of physicochemical processes was carried out in the interaction of the corium melt with sacrificial materials, while simulating radiant heat exchange with the concrete protections of the molten core catcher and the console farm. Passive principles are used to cool molten core catcher. The response time of the passive valve, which provides the supply of water to the surface of the melt, is determined by the rates of heating of the thermal shields. The results of calculation and experimental work on the validation of the operational capability of the AES-2006 project are presented

Для обоснования эффективности устройства локализации расплава был выполнен расчетный анализ физико-химических процессов при взаимодействии расплава кориума с жертвенными материалами с одновременным моделированием лучистого теплообмена с бетонными защитами корпуса устройства локализации расплава и фермы-консоли. Для охлаждения устройства локализации расплава используются пассивные принципы. Время срабатывания пассивного клапана, обеспечивающего подачу воды на поверхность расплава, определяется темпами разогрева тепловых защит. Представлены результаты расчетных и экспериментальных работ по обоснованию работоспособности устройства локализации расплава проекта АЭС-2006

Computational and experimental researches of in-vessel corium melt retention possibility in WWER reactors

S21

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LENINGRAD-2 REACTOR; COMPUTER CALCULATIONS; CORE CATCHERS;
MELTDOWN; S CODES; CONFINEMENT

Justification of the in-vessel corium melt retention possibility is a complex task requiring for its solution of computational researches with application of special codes and experimental investigations for various physical and chemical phenomena. The results of the calculation analysis for the Russian heavy-duty code SOKRAT of the effectiveness of such a measure for the design of a reactor similar to that used in the LAES-2 project (RU WWER-1200) are considered. The obtained results made it possible to evaluate the necessity of using other technical solutions for the in-body retention of the corium in the WWER-1200 reactor. Also, the results of experimental studies carried out in support of the concept of in-vessel confinement

Обоснование возможности удержания расплава внутри корпуса реактора является комплексной задачей, требующей выполнения расчетного и экспериментального обоснования различных физических и химических явлений. Рассмотрены результаты расчетного анализа по российскому тяжелоаварийному коду СОКРАТ эффективности такой меры для конструкции реактора, аналогичной использованной в проекте ЛАЭС-2 (РУ ВВЭР-1200). Полученные результаты позволили оценить необходимость использования других технических решений для внутрикорпусного удержания кориума в РУ ВВЭР-1200. Также приводятся результаты экспериментальных исследований, выполненных в поддержку рассматриваемой концепции внутрикорпусного удержания

Variation of boric acid solution concentration in the course of corium catcher operation

S21

Shmal', I.I. (AO #Left Double Quotation Mark#Atomenergoproekt#Right Double Quotation Mark#, Moscow (RU))

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Izmenenie kontsentratsii rastvora bornoj kisloty v protsesse funktsionirovaniya ULR

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International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomenergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomenergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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BORIC ACID; WWER TYPE REACTORS; CORE CATCHERS; CORIUM; REACTOR
COOLING SYSTEMS; TEMPERATURE DEPENDENCE; WATER; REACTOR SAFETY;
ABUNDANCE

Variation of boric acid solution concentration in the gap between the corium catcher (CC) casing outer surface and the reactor cavity during the boiling of water arrived into the gap between the CC casing outer surface and the reactor cavity inner surface. The maximal mass of boric acid solid phase precipitating in the reactor cavity bottom part has been determined. It is assumed that the boric acid solid phase emerges in the boric acid solution on reaching the limiting concentration for the corresponding value of temperature

Рассмотрено изменение концентрации раствора борной кислоты в зазоре между наружной поверхностью корпуса устройства локализации расплава (УЛР) и внутренней поверхностью шахты реактора в процессе выкипания воды, поступившей в результате разрыва трубопровода первого контура в зазор между наружной поверхностью корпуса УЛР и внутренней поверхностью шахты реактора. Получено максимальное значение массы твердой фазы борной кислоты, выпадающей в донной части шахты реактора. Принято, что твердая фаза борной кислоты образуется в растворе борной кислоты по достижению предельной концентрации для соответствующего значения температуры

Thermophysical and nuclear safety analysis of spent fuel pool during an accident with full nuclear power plant blackout

S21

Bezborodov, A.N.; Borisenkov, A.Eh.; Vasil'ev, A.D.; Zvonarev, Yu.A.; Merkulov, V.V.; Tebin, V.V.; Osadchij, A.I. (Natsional'nyj Issledovatel'skij Tsentr #Left Double Quotation Mark#Kurchatovskij Inst.#Right Double Quotation Mark#, Moscow (RU))

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Теплофизический анализ и анализ ядерной безопасности бассейна выдержки при аварии с полным длител'ным обесточиванием AEhS

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WWER TYPE REACTORS; FUEL STORAGE POOLS; REACTOR SAFETY; LOSS OF
COOLANT; MONTE CARLO METHOD; COMPUTER CODES; THREE-DIMENSIONAL
CALCULATIONS; CRITICALITY

The paper presents the results of nuclear safety calculation for the severe accident with full station blackout and leak in the fuel storage pond (FSP) for nuclear plant with WWER-1200. The calculation of K_{eff} regard to the non-uniformity water level during the coolant boiling in FSP. Thermophysical analysis of the FSP drying off is carried out using the best estimate code SOKRAT. Calculation of K_{eff} is carried out by SAPFIR-2006 software, which is based by the Monte-Carlo method, with three-dimensional full-scale modelling of the FSP rack

Представлены результаты расчета критичности для различных этапов протекания запроектной аварии с некомпенсируемой течью и длительным полным обесточиванием в бассейне выдержки АЭС с реактором ВВЭР-1200. Расчет $K_{эфф}$ учитывает неравномерности уровня воды при кипении теплоносителя в бассейне выдержки. Теплофизический анализ осушения БВ проводится при помощи тяжелоаварийного кода улучшенной оценки СОКРАТ. Расчеты $K_{эфф}$ ведутся с использованием программного комплекса САПФИР-2006, реализующего метод Монте-Карло, с трехмерным полномасштабным моделированием секций стеллажей бассейна выдержки

Experimental studies under the substantiation of representativeness the temperature control core of the reactor WWER-TOI

S21

Churkin, A.N.; Bezrukov, Yu.A.; Vasil'chenko, I.N.; Supronenko, M.N.; Lobachev, S.M.; Lisenkov, E.A.; Kushmanov, S.A. (AO OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark#, Podol'sk (RU))

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2 refs., 8 figs.

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WWER TYPE REACTORS; REACTOR CORES; THERMAL HYDRAULICS;
TEMPERATURE DEPENDENCE; BENCH-SCALE EXPERIMENTS; FUEL ASSEMBLIES;
REACTOR CONTROL SYSTEMS; TEMPERATURE CONTROL

To justify the representativity of measurements of the temperature at the outlet of the WWER-TOI was carried out the complex of scientific-research works, including: calculation of heat-release in the KNIT and in the structural elements of the reactor core, thermohydraulic calculation of coolant in the area of location GT with the KNIT, experimental research. Experimental studies were carried out on the test facility OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark# with non-heated rods bundle with full-scale cross-section, with length 1230 mm. During the experiments using the model of the KNIT was recorded the change on height of the temperature difference between the main flow and the more cool coolant flowing in GT. The temperature difference was created by feeding the coolant flow into the model GT coolant with temperature on 12-15#Degree Celsius# lower than the temperature of the coolant in space of fuel rods

Для обоснования представительности измерений температуры на выходе из ТВС ВВЭР-ТОИ был проведен комплекс научно-исследовательских работ, включающих в себя: расчет тепловыделений в каналах измерения нейтронного потока и температуры (КНИТ) и в элементах конструкции активной зоны, расчет теплогидравлики теплоносителя в зоне расположения направляющих каналов (НК) с КНИТ, экспериментальные исследования. Экспериментальные исследования проводились на стенде ОКБ #Left Double Quotation Mark#ГИДРОПРЕСС#Right Double Quotation Mark# с необогреваемой моделью пучка ТВС, полномасштабной в поперечном сечении, длиной 1230 мм. В ходе экспериментов с помощью модели КНИТ фиксировалось изменение по высоте разности температур между основным потоком и более холодным теплоносителем, текущим в НК. Разность температур создавалась подачей в модель НК теплоносителя с температурой на 12-15#Degree Celsius# ниже, чем температура теплоносителя в межтвэльном пространстве

Autonomous thermosiphon based PHRS for reactor coolant circuit

S21

Sviridenko, I.I. (Sevastopol'skij Gosudarstvennyj Univ., Sevastopol' (RU)); Shevelev, D.V. (VNIIEhS, Moscow (RU))

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Avtonomnaya SPOT pervogo kontura s termosifonnym teploobmennym oborudovaniem

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WWER TYPE REACTORS; REACTOR SAFETY; PRIMARY COOLANT CIRCUITS; OUTAGES; LOSS OF COOLANT; STEAM GENERATORS

The design characteristics of the passive heat removal system (PHRS R), which removes the residual heat of the core of the WWER core from the coolant of the primary circuit in case of an accident with a complete long-term de-energization in the conditions of a dense primary circuit, and also for small leaks of the primary circuit are presented. Comparative characteristics of the efficiency of PHRS R with heat removal from the heat exchanger-condenser by atmospheric air and water in the tanks of emergency heat removal

Представлены расчетные характеристики системы пассивного отвода теплоты (СПОТ Р), осуществляющей отвод остаточного тепловыделения активной зоны ВВЭР от теплоносителя первого контура при аварии с полным длительным обесточиванием в условиях плотного первого контура, а также при малых течах первого контура. Приводятся сравнительные характеристики эффективности СПОТ Р с теплоотводом от теплообменника-конденсатора по атмосферному воздуху и воде, находящейся в баках аварийного отвода теплоты

Analysis of strength and durability of reinforced concrete structures of nuclear power plant

S21

Modestov, V.S.; Lukin, A.V.; Popov, I.A.; Smirnov, A.B. (FGBOU VPO SPbGPU, Sankt-Peterburg (RU))

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Analiz zhivuchesti i otkazoustojchivosti zhelezobetonnykh konstruksij reaktornogo otdeleniya AEhS

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WWER TYPE REACTORS; NUCLEAR POWER PLANTS; FINITE ELEMENT METHOD;
REINFORCED CONCRETE; COMPUTER CODES; COMPUTERIZED SIMULATION;
DYNAMIC LOADS; WEAR RESISTANCE; SERVICE LIFE; THICKNESS; IMPACT
STRENGTH; CONTAINMENT

Finite elements method was used as a modeling technique. Various verification problems were solved in different CAE systems (ANSYS, ABAQUS, LS-DYNA). Nonlinear concrete material models were used. Reinforcement of the outer containment wall was taken into consideration in several ways. Methods used for reinforced concrete structures modeling show high level of adequacy to experimental data. Therefore it is considered that modeling technique used in these studies is appropriate to be used for modelling of problem of aircraft impact with NPP outer containment wall. Assesment of the NPP concrete walls safe thicknesses is possible based on proposed strength and durability criteria. Floor response spectrums calculation method was developed based on accelerograms obtained for each building floor

Применялся метод конечных элементов. Построение моделей и расчеты осуществлялись с использованием различных прикладных программных систем (ANSYS, ABAQUS, LS-DYNA). Использовались нелинейные модели деформирования бетона. Арматура в бетоне моделировалась явно, что позволило оценить степень ее деформирования. Используемые методы моделирования процессов нелинейного деформирования бетона демонстрируют высокий уровень соответствия экспериментальным данным, что оправдывает их применение для моделирования задач столкновения самолета с внешней защитной оболочкой здания реактора АЭС. Возможна оценка безопасных толщин защитных железобетонных конструкций АЭС на основе предложенных критериев прочности и живучести. Разработана методика вычисления поэтажных спектров отклика в различных расчетных точках здания реактора на основе полученных поэтажных акселерограмм

Experimental studies of air-cooled finned heat exchanger operation in WWER passive safety system

S21

Morozov, A.V.; Kalyakin, D.S.; Ragulin, S.V.; Sakhigareev, A.R.; Soshkina, A.S.; Shlepkin, A.S. (GNTs RF - Fiziko-Ehnergeticheskij Inst. imeni A.I. Lejpunskogo, Obninsk (RU))

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Ehksperimental'noe issledovanie raboty vozdukhookhlazhdaemogo orebrennogo teploobmennika v passivnoj sisteme bezopasnosti AEhS s VVEhR

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WWER TYPE REACTORS; REACTOR SAFETY; REACTOR CORES; HEAT EXCHANGERS; PRIMARY COOLANT CIRCUITS; PILOT PLANTS; PRESSURE RANGE MEGA PA 01-10; VERIFICATION

To select the basic parameters of the heat exchanger and for the definition of his operation characteristics the experimental facility with model heat exchanger has been constructed in IPPE. The test program involved two phases of the experiments: for plain tubes and pipes with longitudinal fins. Experiments have been carried out on #Left Double Quotation Mark#pure#Right Double Quotation Mark# steam, and with steam-gas mixtures of different composition at a pressure of 0.36-0.37 MPa. The results obtained can be used for the verification of computer codes and modeling of the emergency processes in WWER reactor facility taking into account the operation of the complex of passive safety systems, including GE-2, SPOT, and device for non-condensable gases removing

Для выбора основных параметров теплообменника и определения особенностей его работы в ГНЦ РФ-ФЭИ создана экспериментальная установка с модельным теплообменником. Программа экспериментов предусматривает проведение двух этапов экспериментов: на гладких трубах и трубах с продольным оребрением. Опыты проводились на #Left Double Quotation Mark#чистом#Right Double Quotation Mark# паре, а также с подачей парогазовой смеси различного состава при давлении 0,36-0,37 МПа. Полученные результаты можно использовать для верификации расчетных кодов и моделирования аварийных процессов в реакторной установке ВВЭР с учетом работы комплекса систем пассивной безопасности, включающих в себя систему GE-2, СПОТ, а также устройство удаления неконденсирующихся газов

Experimental study of DNB on the WWER FA model with mixing grids with MCP disconnection

S21

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Ehksperimental'nye issledovaniya krizisa teploobmena na modeli TVS VVEhR s peremeshivayushchimi reshetkami pri otklyuchenii GTsN

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vp.

WWER TYPE REACTORS; FUEL ASSEMBLIES; HEAT TRANSFER; CIRCULATING SYSTEMS; REACTOR SAFETY; BENCH-SCALE EXPERIMENTS; PROCESS DEVELOPMENT UNITS; REACTOR COOLING SYSTEMS

Prepared for unsteady experimental research is the 19-rod model FA WWER-1200 with a heated length of 3.73 m with uniform heat release across the section. Along with standard spacing gratings, the model is equipped with two intermediate lattices-intensifiers of the #Left Double Quotation Mark#vortex#Right Double Quotation Mark# type and two-type #Left Double Quotation Mark#sweep#Right Double Quotation Mark#. Experimental studies of the heat transfer crisis were carried out in the worst case, involving the shutdown of all the working MCPs. It is found that in the nonstationary regimes studied, with a decrease in the flow rate, the critical heat flux is 4% higher than in steady-state regimes with the same parameters (flow rate, pressure, inlet temperature). Thus, the correlation for stationary regimes is conservative in the calculation of reserves before the crisis when the MCP is shut off

Подготовлена к нестационарным экспериментальным исследованиям 19-стержневая модель ТВС ВВЭР-1200 с обогреваемой длиной 3,73 м при равномерном тепловыделении по сечению. Наряду со стандартными дистанционирующими решетками модель оснащена двумя промежуточными решетками-интенсификаторами типа #Left Double Quotation Mark#вихрь#Right Double Quotation Mark# и двумя - типа #Left Double Quotation Mark#прогонка#Right Double Quotation Mark#. Проведены экспериментальные исследования кризиса теплообмена при худшем варианте, связанным с отключением всех работающих ГЦН. Получено, что в исследованных нестационарных режимах при снижении расхода критические тепловые потоки на 4 % выше, нежели в стационарных режимах при тех же параметрах (расходе, давлении, температуре на входе). Таким образом, корреляция для стационарных режимов является консервативной при расчете запасов до кризиса при отключении ГЦН

Experimental researches of hydraulic friction and levelling capacity of submerged perforated sheet in PGV test facility (EREC)

S21

Melikhov, O.I.; Elkin, I.V.; Melikhov, V.I.; Nikonov, S.M.; Parfenov, Yu.V.; Emel'yanov, D.A.; Nerovnov, A.A. (AO #Left Double Quotation Mark#EhNITs#Right Double Quotation Mark#, Ehlektrogorsk (RU))

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Ehksperimental'nye issledovaniya gidrosoprotivleniya i vyravnivayushchej sposobnosti PDL na stende PGV (EhNITs)

vp.

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9 refs., 9 figs., 2 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; STEAM GENERATORS; HEAT TRANSFER; REACTOR COOLING SYSTEMS; BENCH-SCALE EXPERIMENTS; PERFORATION; HYDRODYNAMICS; REACTOR SAFETY

The possibility of the increasing of the steam equalization ability of the submerged perforated sheet with the help of the submerged perforated sheet with non-uniform perforation was studied. The PGV test facility was constructed in JSC #Left Double Quotation Mark#EREC#Right Double Quotation Mark# for the experimental investigation of this problem. Several series of experiments were performed in PGV test facility in 2010-2013. The constructive shortcomings of the test facility have been identified and corrected in the course of this work. After removing of all constructive shortcomings in the test facility experimental researches were performed with using submerged perforated sheets with uniform and non-uniform perforations. Experimental data obtained in the experiments are reported. The hydraulic resistance of the submerged perforated sheet with uniform perforation for different flow rates of the steam and two-phase correction factor were determined on the basis of the experimental data analysis. The steam equalization ability of the SPS was evaluated on the basis of the experimental data analysis by different methods

Исследовались возможности повышения выравнивающей способности погруженного дырчатого листа (ПДЛ) с помощью применения ПДЛ с неравномерной перфорацией. Для экспериментального исследования этого вопроса в АО #Left Double Quotation

Mark#ЭНИЦ#Right Double Quotation Mark# был сооружен стенд ПГВ. В течение 2010-2013 годов на стенде ПГВ были проведены несколько серий экспериментов. В ходе этой работы выявлялись и устранялись конструктивные недостатки стенда. После проведения всех модификаций стенда были выполнены экспериментальные исследования с использованием ПДЛ с равномерной и неравномерной перфорацией. Изложены и систематизированы опытные данные, полученные в экспериментах. С помощью анализа опытных данных были определены гидравлические сопротивления ПДЛ с равномерной перфорацией при различных расходах подаваемого пара, и получена поправка на двухфазность потока через ПДЛ. На основе опытных данных выполнена оценка выравнивающей способности ПДЛ по различным методикам

Results of physics start-up tests of Mochovce and Bohunice units with 2-nd generation Gd fuel (average enrichment 4.87 %)

S21

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AEhS s VVEhR

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MOCHOVCE-1 REACTOR; MOCHOVCE-2 REACTOR; NUCLEAR POWER PLANTS; MIXED OXIDE FUELS; START-UP; REACTOR CORES; COMPUTER CALCULATIONS; COMPUTER CODES; STATISTICS

There are presented main features of the fuel and the list of experimental neutron-physical characteristics measured during physics start-up tests. All together there were carried out 14 physics start-ups at Bohunice and Mochovce Units with the new type of fuel. Differences between theoretical and experimental neutron-physical characteristics were statistically processed and compared with the tests acceptance criteria. There are summarized results of reactor physics start-ups with 2-nd generation Gd fuel usage

Представлены основные особенности этого топлива и перечень экспериментальных нейтронно-физических характеристик, измеренных во время испытаний при физпуске.

Всего было проведено 14 испытаний при физпусках на блоках АЭС Богунице и АЭС Моховце с новым типом топлива. Различия между теоретическими и экспериментальными нейтронно-физическими характеристиками были статистически обработаны, и было проведено сравнение с приемочными критериями испытаний. Представлены обобщенные результаты по испытаниям при физпуске с использованием гадолиниевого топлива второго поколения

Possible beyond design-basic accident analysis at handling and transportation of reactor WWER-1200 fuel assemblies

S21

Sikorin, S.N.; Polozov, S.A.; Domorad, Yu.V. (Gosudarstvennoe Nauchnoe Uchrezhdenie #Left Double Quotation Mark#Ob"edinennyj Inst. Ehnergeticheskikh i Yadernykh Issledovanij - Sosny#Right Double Quotation Mark# Natsional'noj Akademii Nauk Belarusi, Minsk (BY)), e-mail: polazau@sosny.bas-net.by

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Analiz vozmozhnykh zaproektnykh avarij pri obrashchenii i transportirovanii TVS reaktora VVEhR-1200

vp.

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4 refs., 3 figs., 2 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; REACTOR ACCIDENTS; FUEL ASSEMBLIES; M CODES; REACTOR SAFETY; CRITICALITY; RADIATION ACCIDENTS

By means of program codes MCUPD and MCNP the analytical researches were executed on criticality of multiple systems on the basis of fuel rods, contained in one fuel assembly. The criticality calculations of fuel rod systems dipping into the water and a steam-and-water mix are presented. The opportunity to create nuclear-dangerous system with Keff exceeding 1,09 in the water are shown. Technical decisions for exception of super-criticality systems formation and maintenance of nuclear safety requirements are offered

С помощью программных кодов MCUPD и MCNP выполнены расчетные исследования и представлены результаты расчетов по критичности размножающих систем на основе

ТВЭЛОВ, составляющих одну ТВС при помещении их в водную среду и пароводяную смесь. Показаны варианты возможности создать в водной среде ядерно-опасную надкритическую систему с Кэфф, превышающим 1,09. Предложены технические решения для исключения образования надкритических систем и обеспечения требований ядерной безопасности

Experience of developing WWER-440 fuel loading patterns during operation at the updated power level

S21

Adeev, V.A.; Melenchuk, I.S.; Panov, A.E. (Kol'skaya AEhS, Polyarnye Zori (RU))

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Opyt formirovaniya toplivnykh zagruzok reaktorov VVEhR-440 pri rabote na povyshennom urovne moshchnosti

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International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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KOLA-2 REACTOR; KOLA-3 REACTOR; KOLA-4 REACTOR; NUCLEAR FUELS; FUEL ASSEMBLIES; REACTOR FUELING; REACTOR OPERATION; REACTOR SAFETY; FUEL RODS

The Kola NPP continues commercial operation of 2nd generation fuel (FA-2) and trial operation of 3rd generation fuel (FA-3), which has a number of design features providing the best operational characteristics. The results of WWER-440 core operation with FA-2 and FA-3 with enrichment increased up to 4.87%, and at the power level updated to 107% of nominal power level are given. Quantitative dependencies of change of FA multiplication properties are derived which indicate the actual change of power of the 1st year assemblies during burn-up, and can be used to design next fuel loading. Special technique of fresh fuel loading as well as clarification of features of nature of change in the 1st year assemblies' multiplication properties allowed to achieve 107% power level at the Kola NPP Unit 4

На Кольской АЭС проводится промышленная эксплуатация топлива 2-го поколения (ПК-2) и опытно-промышленная эксплуатация топлива 3-го поколения (ПК-3), имеющего ряд

конструктивных особенностей, обеспечивающих лучшие эксплуатационные характеристики. Приведены результаты эксплуатации активной зоны реактора ВВЭР-440 с РК-2 и РК-3 повышенного до 4.87% обогащения на повышенном до 107% от номинального уровня мощности. Получены количественные зависимости изменения размножающих свойств РК, которые отражают реальное изменение мощности кассет первого года в процессе выгорания и могут быть использованы при проектировании следующей топливной загрузки. Особый способ загрузки свежих кассет наряду с уточнением характеристик изменения свойств кассет 1-го года позволил достичь 107% мощности на блоке 4 Кольской АЭС

Investigation of LWR reflooding parameters taking into consideration the model fuel assemblies scale factor and cladding thermo-physical properties

S21

Bazyuk, S.S.; Parshin, N.Ya.; Popov, E.B.; Soldatkin, D.M. (FGUP NII NPO #Left Double Quotation Mark#LUC#Right Double Quotation Mark#, Podol'sk (RU)); Kuzma-Kichta, Yu.A.; Meshchanov, A.A. (FGBOU VPO #Left Double Quotation Mark#NIU MEI#Right Double Quotation Mark#, Moscow (RU))

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Issledovanie kharakteristik povtornogo zaliva model'nykh TVS legkovodnykh reaktorov s uchetom masshtabnogo faktora i teplofizicheskikh svoystv imitatora tvehla

vp.

(RU)

11 refs., 4 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; FUEL RACKS; FUEL ASSEMBLIES; BENCH-SCALE EXPERIMENTS; WETTING HEAT; REACTOR SAFETY; P CODES; THERMAL HYDRAULICS; HEAT TRANSFER

To substantiate the earlier generalizing dependence on the wetting rate on model assemblies with a large number of fuel rods, the results of experiments at the FLECHT stand were processed. A model is proposed and a calculation program is developed to determine the characteristics of the reflooding, built into the calculation code PARAM - TG and based on the heat balance. The model and the program were tested by comparing the results of the calculation and the

experiment in simulating a reflooding at the PARAMETER stand on single-rod and 19-rod assemblies, which showed a satisfactory agreement of the wetting time data. In a number of experiments, the experimental and calculated data on the position of the wetting front in the upper part of the assembly sharply diverge

Для обоснования полученной ранее обобщающей зависимости по скорости фронта смачивания на модельных сборках с большим количеством ТВЭЛов обработаны результаты экспериментов на стенде FLECHT. Предложена модель и разработана расчетная программа для определения характеристик повторного залива, встроенная в расчетный код PARAM - TG и основанная на балансе тепла. Модель и программа апробированы путем сопоставления результатов расчета и эксперимента при моделировании повторного залива на стенде ПАРАМЕТР на однотвэльной и 19-стержневой сборках, что показало удовлетворительное согласование данных по времени смачивания. В ряде экспериментов опытные и расчетные данные по положению фронта смачивания в верхней части сборки резко расходятся

Experiment calculated research of plate-type spacer characteristics o WWER FA S21

Luzan, Yu.V.; Malakhov, A.A.; Aksenov, P.M.; Lerner, A.E.; Kochergin, V.M.; Tsirin, S.I. (EhLEMASH - OAO #Left Double Quotation Mark#Mashinostroitel'nyj Zavod#Right Double Quotation Mark#, Ehlektrostal' (RU))

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Raschetno-ehksperimental'noe issledovanie kharakteristik distantsioniruyushchej reshetki plastinchatoj konstruktsii dlya TVS AEhS VVEhR

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WWER TYPE REACTORS; SPACERS; FUEL RODS; FUEL ASSEMBLIES; REACTOR LATTICES; REACTOR CELLS; ZIRCONIUM ALLOYS; MECHANICAL PROPERTIES

A Zr alloy E110 plate-type spacer design for WWER FAs was elaborated. Experimental models of WA WWER-440 and FA WWER-1000 spacers were manufactured based on gas-laser cut

technology. Experiment calculated comparative characteristic research of the new and standard spacers of WA WWER-440, second generation, were held. Spacers geometry (fit in diameters, spacer pace) measurements underwent statistical treatment. Spacer flexible/elastic deformation reserve was determined, as well as dummy fuel rod pushing force, flexural stiffness and carrying capacity while compression in spacers plane. Proved that the proposed plate-type spacer all mentioned characteristics are better than the standard cell-type

Разработана конструкция дистанционирующей решетки пластинчатого типа для ТВС АЭС типа ВВЭР из циркониевого сплава Э110. С использованием технологии газолазерной резки изготовлены опытные образцы дистанционирующих решеток РК ВВЭР-440 и ТВСА ВВЭР-1000. Проведены сравнительные расчетно-экспериментальные исследования характеристик предлагаемой и штатной дистанционирующих решеток для РК ВВЭР-440 второго поколения. Проведена статистическая обработка измерений геометрических параметров дистанционирующих решеток (вписанные диаметры, шаг ячеек). Определены запас упругих деформаций ячеек и усилие проталкивания имитаторов твэлов, жесткость на изгиб и несущая способность при сжатии в плоскости дистанционирующих решеток. Показано, что предлагаемая дистанционирующая решетка пластинчатой конструкции по всем указанным параметрам имеет лучшие характеристики по сравнению со штатной дистанционирующей решеткой ячеистого типа

Experiment #Left Double Quotation Mark#Steady-state crisis#Right Double Quotation Mark# and its result

S21

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Ekhsperiment #Left Double Quotation Mark#Krizis statsionarnyj#Right Double Quotation Mark# i ego rezul'taty

vp.

(RU)

4 refs., 8 figs., 1 tab.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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MIR REACTOR; FUEL ASSEMBLIES; CONTAINMENT; REACTOR SAFETY; THERMAL HYDRAULICS; FUEL CANS; FUEL ELEMENTS; HEAT TRANSFER

The experiment #Left Double Quotation Mark#Steady-state crisis#Right Double Quotation Mark# was conducted in the MIR reactor. The results of the reactor experiment and post-test data analysis are presented. The possibility of fixing the moment of the onset and development of the heat-transfer crisis of the first kind on the fuel cladding is shown. The possible limits of the parameters that must be realized in the RIA experiment with the heat transfer crisis on the shell are determined. After completion of the tests, all the fuel elements remained sealed. Based on the analysis of the experimental data and the results of the calculation analysis, a conclusion was made about the principle possibility of modeling the first stage of ultimate design-basis accident in the channel of the MIR reactor without installing additional equipment for the cooling circuit of experimental FA

В реакторе МИР проведен эксперимент #Left Double Quotation Mark#Кризис стационарный#Right Double Quotation Mark#. Приведены результаты реакторного эксперимента и посттестового расчетного анализа данных. Показана возможность фиксации момента начала и развития кризиса теплоотдачи I рода на оболочке ТВЭЛОВ. Определены возможные границы параметров, которые необходимо реализовать в эксперименте RIA с кризисом теплообмена на оболочке. После завершения испытаний все ТВЭЛЫ остались герметичными. На основании анализа данных эксперимента и результатов расчетного анализа сделано заключение о принципиальной возможности моделирования I стадии МПА в канале реактора МИР без установки дополнительного оборудования на контур охлаждения экспериментальной ТВС

Improvement of safety and techno-economic characteristics of light water reactors due to micro-fuel use - aggregated data of the experimental works definition of the corrosion resistance of coatings

S21

Agul'nik, M.A.; Grishanin, E.I.; Fal'kovskij, L.N.; Fomichenko, P.A.; Fonarev, B.I. (NITs #Left Double Quotation Mark#Kurchatovskij Inst.#Right Double Quotation Mark#, Moscow (RU))

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9-ya mezhdunarodnaya nauchno-tekhnicheskaya konferentsiya Obespechenie bezopasnosti AEhS s VVEhR

Povyshenie bezopasnosti i tekhniko-ehkonomicheskikh kharakteristik legkovodnykh reaktorov za schet ispol'zovaniya topliva v vide mikrotvehlov - obobshchennye dannye ehksperimental'nykh rabot po opredeleniyu korrozionnoj stojkosti pokrytij

vp.

(RU)

8 refs., 15 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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HTGR TYPE REACTORS; FUEL ELEMENTS; PYROLYTIC CARBON; SILICON CARBIDES; CORROSION PROTECTION; FUEL CANS; FUEL ASSEMBLIES; REACTOR ACCIDENTS; TEMPERATURE RANGE 1000-4000 K; TEMPERATURE RANGE OVER 4000 K

In the presented experimental work HTGR microfuel elements were investigated for the purpose of definition of corrosion stability and integrity of their PyC and SiC coatings in light water coolant at nominal and emergency conditions (including heavy and hypothetical accidents). Non-irradiated MFE with core diameter from 0.5 to 1.5 mm and thickness of outer coatings from 55 to 125 microns were investigated. Tests were carried out at the nominal parameters in water and steam of appropriate state. Tests at superhigh temperatures were carried out in a gas-vapor medium formed by the products of combustion of propane in oxygen. The results of the experiments indicate that the obtained limiting temperature of the corrosion resistance of microfuel elements exceeds the temperature of applicability of the structural materials of the fuel assembly

В представленных результатах экспериментальных работ исследовались микротвэлы ВТГР с целью определения коррозионной стойкости и целостности их покрытий, выполненных из пироуглерода (PyC) и карбида кремния (SiC), в среде легководного теплоносителя при номинальных и аварийных режимах АЭС (включая тяжелые и гипотетические аварии). Исследовались необлученные МТЭ с диаметром зерна от 0,5 до 1,5 мм и толщиной наружных покрытий от 55 до 125 мкм. Испытания при номинальных параметрах проводились в воде и паре соответствующего качества. Испытания при сверхвысоких температурах проводились в парогазовой среде, образованной продуктами сгорания пропана в кислороде. Результаты экспериментов свидетельствуют о том, что полученная предельная температура коррозионной стойкости микротвэлов превышает температуру применимости конструкционных материалов тепловыделяющей сборки

Numerical estimation of FP release from the corium during localization and cooling in core catcher of WWER-TOI

S21

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Raschetnaya otsenka vykhoda radioaktivnykh produktov deleniya iz koriuma v protsesse ego lokalizatsii i okhlazhdeniya v ULR VVEhR-TOI

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(RU)

6 refs., 9 figs., 7 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; FISSION PRODUCT RELEASE; CORIUM; REACTOR ACCIDENTS; CORE CATCHERS; REACTOR CORES; REACTOR VESSELS

The calculation is based on the results of mathematical modeling of the operation of device at out-vessel stage of severe accident. The core catcher is intended to reduce to a safe level of radiation consequences of severe accidents with the release of the radioactive molten material of core outside the reactor vessel. The results of the basic calculations of FP release from core catcher allow the following conclusions: volatile radionuclides, including telluride, completely release from core catcher, release of non-volatile radionuclides from the melt pool, located in the core catcher is substantially less than their release at the stage of core degradation. It can be explained by a decrease of temperature of corium when it interacts with the sacrificial material of core catcher. As well as it can be explained by a rapid decrease of temperature of upper layer of the melt pool while cooling of the reactor vessel by water from surface

Расчет основан на результатах математического моделирования функционирования устройства на внекорпусной стадии тяжелой аварии. Устройство локализации расплава (УЛР) предназначено для уменьшения до безопасного уровня радиационных последствий тяжелых ЗПА с выходом радиоактивного расплавленного тепловыделяющего материала активной зоны (АЗ) за пределы корпуса реактора. Полученные результаты базовых расчетов выхода продуктов деления из УЛР позволяют сделать следующие выводы: летучие радионуклиды, включая теллуриды, полностью выходят из УЛР, выход слаболетучих радионуклидов из бассейна расплава, находящегося в УЛР, существенно меньше по сравнению с их выходом на стадии деградации АЗ. Это объясняется снижением температуры кориума при его взаимодействии с жертвенным материалом УЛР, а также быстрым снижением температуры верхнего слоя бассейна расплава при охлаждении корпуса реактора водой с поверхности

Experience in operation of nuclear fuel at the concern #Left Double Quotation Mark#Rosenergoatom#Right Double Quotation Mark# NPPs with WWER S21

Anufriev, D.G. (AO #Left Double Quotation Mark#Kontsern Rosehnergoatom#Right Double Quotation Mark#, Moscow (RU))

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AEhS s VVEhR

Опыт эксплуатации ядерного топлива на АЕhS s VVEhR ОАО #Left Double Quotation Mark#Концерн Росэнергоатом#Right Double Quotation Mark#

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(RU)

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WWER TYPE REACTORS; FUEL CYCLE; REACTOR OPERATION; FUEL ASSEMBLIES; TIME DEPENDENCE; POWER DENSITY; NUCLEAR POWER PLANTS

The steps have been determined to implement the problems related to uprating of thermal power up to 104% of nominal power at NPP Units with WWER-1000 and up to 107% of nominal power at NPP Units with WWER-440, as well as to transition of NPPs with WWER-1000 to operation with 18-months fuel cycles within the framework of the Programme for increase in electric power generation at the operating Units of the #Left Double Quotation Mark#Concern Rosenergoatom#Right Double Quotation Mark# NPP for 2011-2015 approved by the State Corporation #Left Double Quotation Mark#Rosatom#Right Double Quotation Mark#. The experience of operation of new types of fuel assemblies under operating conditions at high power in long fuel cycles, as well as ways to solve problematic issues, is considered. The statistics on the failed (unsealed) fuel assemblies at the power units of the nuclear power plant with WWER are given

В рамках утвержденной Госкорпорацией #Left Double Quotation Mark#Росатом#Right Double Quotation Mark# Программы по увеличению выработки электроэнергии на действующих энергоблоках АЭС ОАО #Left Double Quotation Mark#Концерн Росэнергоатом#Right Double Quotation Mark# на 2011-2015 годы определены шаги по реализации задач повышения тепловой мощности энергоблоков АЭС с ВВЭР-1000 до 104% номинальной и АЭС с ВВЭР-440 до 107% номинальной, а также перевода АЭС с ВВЭР-1000 на эксплуатацию в 18-ти месячных топливных циклах. Рассмотрен опыт эксплуатации новых видов ТВС в условиях эксплуатации на повышенной мощности в длительных топливных циклах, а также пути решения проблемных вопросов. Приведена статистика по отказавшим (негерметичным) ТВС на энергоблоках АЭС с ВВЭР

Methods of evaluation of sensitivity and uncertainty of reactor core neutron and physical model

S21

Adeev, V.A.; Marakulin, K.I. (Kol'skaya AEhS, Polyarnye Zori (RU))

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Metody otsenki chuvstvitel'nosti i neopredelennosti nejtronno-fizicheskoy modeli aktivnoj zony reaktora

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(RU)

8 refs., 1 tab.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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KOLA-4 REACTOR; REACTOR CORES; SENSITIVITY; REACTOR FUELING; REACTOR OPERATION; MONTE CARLO METHOD; DATA COVARIANCES; REACTOR SAFETY; OPTIMIZATION

The Nuclear Safety and Reliability Department of the Kola NPP has developed two methods for analyzing the sensitivity of the uneven energy release of the core from accidental fuel burn-up deviations. Using the developed techniques, several load calculations were performed to compare the accuracy of the results obtained by different methods and to assess the effect of asymmetric loading on the sensitivity of the energy release field. A comparative analysis of two downloads is made. As a basis for comparing accuracy, the Monte Carlo method with a large volume of typed statistics was chosen. In a comparative analysis of two downloads in order to demonstrate the validity of using sensitivity criteria as an optimization parameter in the design of new loads, the loads of the 4th unit of the Kola NPP were used

В Отделе ядерной безопасности и надежности Кольской АЭС разработаны две методики анализа чувствительности неравномерности энерговыделения активной зоны от случайных отклонений выгорания топлива. При помощи разработанных методик проведены расчеты нескольких загрузок с целью сравнения точности результатов, получаемых по разным методикам и оценки влияния асимметричности загрузки на чувствительность поля энерговыделения. Проведен сравнительный анализ двух загрузок. В качестве базы сравнения точности был выбран метод Монте-Карло с большим объемом набранной статистики. При сравнительном анализе двух загрузок с целью демонстрации обоснованности использования критериев чувствительности в качестве параметра оптимизации при проектировании новых загрузок, использовались загрузки 4-го блока Кольской АЭС

Operational experience with blowdown on PGV-440 steam generators on NPP

Dukovany

S21

Joch, L.; Krautschneider, R.; Junek, L. (Inst. of Applied Mechanics Brno, Ltd., Brno (CZ)); Cancura, Z. (CEZ Group, NPP Temelin, Temelin (CZ)); Svancara, J. (CEZ Group, NPP Dukovany, Dukovany (CZ))

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WWER TYPE REACTORS; NUCLEAR POWER PLANTS; STEAM GENERATORS; BLOWDOWN; PRIMARY COOLANT CIRCUITS; REACTOR SAFETY; THERMAL HYDRAULICS; FEEDWATER

Operational experience with blowdown on PGV-440 steam generators on NPP Dukovany is described. Based on operational observations and measurements additional numerical analyses were done with the aim to reveal possible anomalies in flow from blowdown nozzles. Some analyzes of the steam generator primary collector pocket were also done. Those analyzes had to show if the blowdown from the pocket area can be improved in effectiveness

Дано описание опыта эксплуатации по продувке парогенераторов PGV-440 на АЭС Дукованы. Основываясь на наблюдениях и измерениях, проведенных в процессе эксплуатации, были проведены дополнительные численные анализы с целью выявления возможных аномалий в расходе из патрубков продувки. Были также проведены несколько анализов кармана коллектора первого контура парогенератора. Эти анализы должны были показать, можно ли повысить эффективность продувки из зоны кармана

Thermal shock analysis of steam generator collectors due to SG blowdown system operation

S21

Junek, L.; Ambroz, J.; Cancura, Z.; Ambroz, J.; Chanchura, Z.; Terletskij, Yu. (CEZ GROUP, AEhS #Left Double Quotation Mark#Temelin#Right Double Quotation Mark#, Temelin (CZ));

Yunek, L. (Inst. Prikladnoj Mekhaniki, Brno (CZ))

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AEhS s VVEhR

Analiz termicheskogo udara v kollektorakh PG vsledstvie podklyucheniya sistemy produvki PG

vp.

(RU)

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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TEMELIN-1 REACTOR; BLOWDOWN; STEAM GENERATORS; NUCLEAR POWER PLANTS; COOLING; WELDED JOINTS; REACTOR SAFETY; THERMAL HYDRAULICS; REACTOR START-UP

Operational experience with steam generator blowdown system of PGV-1000 at nuclear power plant #Left Double Quotation Mark#Temelin#Right Double Quotation Mark# is presented . In analyzing measurements of SG No. 4 of the first power unit, cooling of the SG lower part from blowdown system after power unit start-up was detected. On the basis of in-service measurements, calculations were carried out with the purpose of determination of a degree of influence of cooling the SG lower part on possibility of damage of weld No. 111 and critical radius R10 in SG pockets

Представлен эксплуатационный опыт с системой продувки парогенераторов типа ПГВ-1000 на атомной станции #Left Double Quotation Mark#Темелин#Right Double Quotation Mark#. В рамках анализа измерений на ПГ #Numero Sign# 4 первого энергоблока было выявлено подхолаживание нижней части ПГ от системы продувки после пуска энергоблока. На основании эксплуатационных измерений были проведены расчеты с целью определения степени влияния подхолаживания нижней части ПГ на возможность повреждения сварного шва #Numero Sign# 111 и критического радиуса R10 в карманах ПГ

In vessel melt retention strategy for WWER 1000/320 unit

S21

Zdarek, J.; Krhounek, V.; Batek, D. (Inst. Yadernykh Issledovaniy, Rzhesh (CZ))

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AEhS s VVEhR

Strategiya uderzhaniya vnutrikorpusnogo rasplava dlya blokov s VVEhR 1000/320

vp.

(RU)

24 figs.

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WWER TYPE REACTORS; REACTOR ACCIDENTS; NRC KURCHATOV INSTITUTE; BENCHMARKS; COMPUTER CODES; REACTOR COOLING SYSTEMS; SURFACE PROPERTIES; REACTOR VESSELS

First analytical results were received from the Kurchatov Institute, based on those results and input deck we initiated support from the EC and JRC Petten to perform International Benchmark on computer code calculations on IVMR for WWER 1000, with target of providing preliminary results on the feasibility of this mitigation strategy in case of severe accident. In parallel extensive experimental work is carried out on small scale experimental facility at UJV Rez to obtain CHF curve for different surface structure of the RPV steel and with different cooling media composition. Large scale experimental facility, similar to ULPU and FIRM is also under preparation

Первые аналитические результаты работы были получены из Института Курчатова, основанные на тех результатах и входных данных, которые получили от ЕС и JRC Petten в качестве основы, с тем, чтобы провести International Benchmark по расчетам УВР с помощью компьютерных кодов для ВВЭР 1000, с целью обеспечения предварительных результатов по практической осуществимости стратегии ослабления последствий в случае тяжелой аварии. Параллельно проводится обширная экспериментальная работа на маломасштабной экспериментальной установке в UJV Реж, чтобы получить кривую запаса до кризиса кипения для различной поверхностной структуры стали корпуса реактора и с различным составом сред охлаждения. Крупномасштабная экспериментальная установка, подобная ULPU и FIRM, также находится в процессе подготовки

Control of service life of WWER-1000/1200 reactor plant based on degradation evaluation implemented in SAKOR

S21

Merkun, A.V.; Bogachev, A.V.; Semishkin, V.P. (AO OKB #Left-Pointing Double Angle Quotation Mark#GIDROPRESS#Right-Pointing Double Angle Quotation Mark#, Podol'sk (RU))
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Upravlenie resursom RU VVEhR-1000/1200 na osnove realizovannykh v SAKOR otsenok degradatsii

vp.

(RU)

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomenergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomenergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWR TYPE REACTORS; COMPUTERIZED CONTROL SYSTEMS; REACTOR CONTROL SYSTEMS; REACTOR INSTRUMENTATION; REACTOR OPERATION; REACTOR SAFETY; FATIGUE; CRACK PROPAGATION

A probability of crack occurrence in E&P components of WWER-1000/1200 reactor plant, referring to safety classes 1 and 2, is insignificant with absence of corrosion. Nevertheless, according to the valid regulatory documents, when the cyclic damage ability reaches a value less than 1 it means that there is a postulated small crack in a component. Within the frame of SAKOR a cyclic crack extension is considered that were revealed by NDT. The calculated extension of postulated cracks and those revealed by NDT is an important control function for making the solutions on the scope and interval of inspection of both the base metal, and weld metal

Вероятность появления трещин в компонентах ОиТ РУ ВВЭР-1000/1200, относящихся к 1 и 2 классам безопасности, при отсутствии коррозии незначительна. Тем не менее, в соответствие с существующими нормативными документами при достижении циклической повреждаемости некоторого значения меньше 1 означает присутствие в компоненте постулируемой малой трещины. Однозначно в рамках САКОР рассматривается циклический рост трещин, выявленных по результатам НМК. Расчетный рост постулируемых и обнаруженных НМК трещин служит важной управляющей функцией для принятия решений по объему и периодичности контроля как основного металла, так и металла сварных соединений

Elaboration and design justification of the corium confinement and reactor vessel cooling system for active and prospective NPP with WWER RP

S21

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others)

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O razrabotke i raschetnom obosnovanii sistemy vnutrikorpusnogo uderzhaniya rasplava dlya dejstvuyushchikh i proektiruemykh AEhS s RU VVEhR

vp.

(RU)

18 refs., 5 figs., 3 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; CORIUM; REACTOR ACCIDENTS; REACTOR CORES; REACTOR SAFETY; REACTOR VESSELS; CORE FLOODING SYSTEMS; CONFINEMENT; REACTOR CONTROL SYSTEMS; REACTOR INSTRUMENTATION

In order to implement the concept of reactor corium confinement for active NPP with WWER-440 and WWER-1000 RP it is proposed to control severe accidents by flooding the reactor cavity with water arranging its circulation around the reactor pressure vessel. This condition ensures corium and internals confinement inside the reactor vessel and prevention of through-damage of the reactor vessel due to heat removal by water to the ultimate heat sink. In order to ensure corium confinement the following is suggested for implementation: #Left Double Quotation Mark#Device for reactor vessel external cooling#Right Double Quotation Mark# (UNOKR) in WWER-440 RP designs and #Left Double Quotation Mark#Corium confinement and reactor vessel cooling system#Right Double Quotation Mark# (SUROK) in WWER-1000 RP designs. SUROK and UNOKR are means of control of severe beyond design basis accidents aimed at maintaining integrity of the reactor vessel and confinement. Introduction of SUROK and UNOKR will make it possible to decrease emission of radioactive fission products outside NPP in case of severe beyond design basis accidents

Для реализации концепции удержания расплава внутри корпуса реактора для действующих АЭС с РУ ВВЭР-440 и РУ ВВЭР-1000 предлагается управление тяжелой аварией посредством залива водой шахты реактора с организацией ее циркуляции вокруг корпуса реактора. Данное условие обеспечивает удержание расплава активной зоны и ВКУ внутри корпуса реактора и предотвращение сквозного повреждения корпуса реактора за счет отвода тепла водой к конечному поглотителю. Для обеспечения внутрикорпусного удержания расплава предлагается реализовать в проектах АЭС с РУ ВВЭР-440 #Left Double Quotation Mark#Устройство наружного охлаждения корпуса реактора#Right Double Quotation Mark# (УНОКР) и #Left Double Quotation Mark#Систему

удержания расплава и охлаждения корпуса реактора#Right Double Quotation Mark# (СУРОК) в проектах АЭС с РУ ВВЭР-1000. СУРОК и УНОКР являются средствами управления тяжелыми запроектными авариями, направленными на сохранение целостности корпуса реактора и гермообъема. Внедрение СУРОК и УНОКР позволит сократить выход радиоактивных продуктов деления за пределы АЭС в случае возникновения тяжелых запроектных аварий

Polymeric compositions for #Left Double Quotation Mark#dry#Right Double Quotation Mark# decontamination of NPP equipment and premises

S38

Voronik, N.I.; Toropova, V.V. (GNU #Left Double Quotation Mark#Ob"edinennyj Inst. Ehnergeticheskikh i Yadernykh Issledovanij - Sosny#Right Double Quotation Mark#, Minsk (BY))

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Polimernye retseptury dlya #Left Double Quotation Mark#sukhoj#Right Double Quotation Mark# dezaktivatsii oborudovaniya i pomeshchenij AEhS

vp.

(RU)

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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DECONTAMINATION; POLYMERS; OPTIMIZATION; SURFACE COATING; COMPOSITE MATERIALS; LEACHING; CESIUM 137; COBALT 60

In JIPNR #En Dash# #Left Double Quotation Mark#Sosny#Right Double Quotation Mark# NASB developed decontaminating polymeric compositions based on binder #En Dash# polyvinyl alcohol solution with active additives such as nitric and borohydrofluoric acids, 1-hydroxyethylidene diphosphonic acid and its salts, detergents and fillers - natural tripoli; tripoli modified by ferrocyanides of nickel and copper; pulverized dolomite modified by manganese oxides, ferrocyanides of nickel and copper; clinoptilolite modified by iron chlorides (III) and calcium sodium phosphate and potassium ferrocyanide; hydrolytic lignin. It is shown that the developed decontaminating polymeric compositions (pastes) possess high decontaminating capacity (FD 102 #En Dash# 103) and low adhesion to the surfaces of stainless and carbon steels, including painted, plastic, self-leveling floors, teflon-surface. Prolonged leaching method allowed determine the chemical resistance of #Left Double Quotation Mark#dry#Right Double

Quotation Mark# decontamination wastes, strength of ^{137}Cs and ^{60}Co fixations in wastes obtained in result of using new decontamination pastes

В ОИЭЯИ - Сосны НАН Беларуси разработаны дезактивирующие полимерные композиционные рецептуры на основе связующего - раствора поливинилового спирта с активными добавками азотной и бородифтористоводородной кислот, 1-гидроксиэтилендифосфоновой кислоты и ее триаммонийной соли, синтетического моющего средства и наполнителей - природного трепела; трепела, модифицированного ферроцианидами никеля и меди; пылевидного доломита, модифицированного оксидами марганца, ферроцианидами никеля и меди; клиноптилолита, модифицированного хлоридами железа (III) и кальция, фосфатом натрия и ферроцианидом калия; гидролизного лигнина. Показано, что разработанные дезактивирующие полимерные композиционные материалы (пасты) обладают высокой дезактивирующей способностью (Кдез 102 - 103) и малой адгезией к исследованным поверхностям нержавеющей и углеродистой сталей, в том числе окрашенных, пластика, наливных полов, тефлона. Методом длительного выщелачивания определена химическая устойчивость и прочность фиксации радионуклидов ^{137}Cs и ^{60}Co в отходах #Left Double Quotation Mark#сухой#Right Double Quotation Mark# дезактивации, полученных при использовании новых дезактивирующих паст

Application of a low pressure containment filtered venting system based on the dry filtered method

S21

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WWER TYPE REACTORS; CONTAINMENT; VENTILATION BARRIERS; FILTRATION; VAPOR-DOMINATED SYSTEMS; RADIATION PROTECTION; PRESSURE RANGE MEGA PA 10-100; DECONTAMINATION

The basic premise of CFVS is that, independent of the state of the reactor itself, the catastrophic failure of the containment structure can be avoided by filtered discharging of steam, air and incondensable gases like hydrogen to the atmosphere. Filtering the steam prior to its release significantly reduces both the surrounding population risk of exposure to radiation and the potential for land contamination near the site. The application of DFM technology to low pressure confinements is presented and discussed in detail. Furthermore an example of a low pressure DFM design for a WWER-440 is given

Основная предпосылка для ВФСК заключается в том, что независимо от состояния реактора, катастрофическое разрушение конструкции контейнента можно предотвратить посредством фильтрованного отвода пара, воздуха и неконденсирующихся газов, таких как водород, в атмосферу. Фильтрация пара перед его выбросом значительно уменьшает риск облучения населения, проживающего в окрестностях АЭС, а также уменьшает возможность загрязнения земли поблизости от площадки. Детально рассматривается применение технологии на основе метода сухой фильтрации к защитным негазоплотным оболочкам низкого давления. Кроме того, приводится пример проекта на основе этого метода для ВВЭР-440

Forecasting of resonances vibration equipment with elastic waves coolant and with the external periodic loads on NPP with WWER

S21

Proskuryakov, K.N.; Zaporozhets, M.V. (NIU #Left Double Quotation Mark#MEH#Right Double Quotation Mark#, Moscow (RU)); Fedorov, A.I. (Novovoronezhskaya AEhS, Novovoronezh (RU))

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Prognozirovanie vozniknoveniya na AEhS s VVEhR rezonansov vibratsij oborudovaniya s uprugimi volnami teplonositelya i vneshnimi periodicheskimi nagruzkami

vp.

(RU)

7 refs., 4 figs., 2 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomenergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomenergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWR TYPE REACTORS; FORECASTING; REACTOR VESSELS; PRIMARY COOLANT CIRCUITS; OSCILLATIONS; REACTOR SAFETY; DYNAMIC LOADS; EARTHQUAKES

Forecasting are carried out for external loads in relation to the main circulation circuit - dynamic loads caused by the rotation of the MCP, dynamic loads caused by the earthquake, dynamic loads caused by damage to the MCP in the earthquake. A comparison of the response spectrum of one of the variants of the base of the NPP, with the frequency vibration of the primary circuit equipment for NPP with WWER-1000 and self-frequency of elastic waves in the fluid. Analysis of the comparison results shows that the frequency of vibration of the main equipment of the reactor plant and elastic waves are in the frequency band in the spectrum response corresponding to the maximum amplitude of the seismic action

Прогнозирование проведено для внешних нагрузок по отношению к главному циркуляционному контуру - динамических нагрузок, вызванных вращением ГЦН, динамических нагрузок, вызванных землетрясением, динамических нагрузок, вызванных повреждением ГЦН при землетрясении. Проведено сопоставление спектра отклика одного из вариантов основания сооружений АЭС с частотами вибраций оборудования первого контура АЭС с ВВЭР-1000 и собственными частотами упругих волн в теплоносителе. Анализ сопоставляемых результатов показывает, что частоты вибраций основного оборудования реакторной установки и упругих волн попадают в полосу частот в спектре отклика, соответствующую максимальным значениям амплитуды сейсмического воздействия

The influence of high pressure heater working modes over the temperature patterns at the entrance of a reactor core fuel assembly and thermo-physical properties of WWER-1000 reactor system of Kalinin NPP

S21

Baj, V.F.; Bogachek, L.N.; Makarov, S.V. (Kalininskaya AEhS, Udomlya (RU))

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Vliyanie rezhimov raboty PVD na temperaturnye polya na vkhode v TVS aktivnoj zony i TFKh RU VVEhR-1000 Kalininskoj AEhS

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2 refs., 21 figs., 3 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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KALININ-1 REACTOR; FEEDWATER HEATERS; REACTOR COOLING SYSTEMS; FUEL ASSEMBLIES; REACTOR OPERATION; REACTOR CORES; REACTOR SAFETY; TEMPERATURE MEASUREMENT; TEMPERATURE CONTROL

Temperature patterns at the entrance of a reactor core fuel assembly were defined according to the operational data of the units with the standard working scheme of a high pressure heater (HPH) and by the operation with one group of HPH. The coolant at the 1st Unit is taken down into the elliptic bottom of a reactor without skewing (the operational modes of the 29th, 28th, 27th, 26th, 25th, 24th, 22th campaigns with the various makeup of the pull-out parts of the Main Circulating Pump were analyzed). At the 2nd Unit the coolant skews at the angle of 30-40 degrees counter clockwise in the down-coming area of the reactor inlet chamber. The coolant in the down-take ring area of the reactor (Unit 3) inlet chamber streams alike to the coolant at the 1st Unit. At the 4th Unit the coolant in the down-coming chamber skews at 60-80 degrees counter clockwise. For the 4th Unit the given data correlates well with the results of the tests carried out during the pre-commissioning activities

На основании эксплуатационных данных работы блоков при штатной схеме работы подогревателей высокого давления (ПВД) и при работе с одной группой ПВД определены температурные поля теплоносителя на входе в ТВС активной зоны. На блоке 1 теплоноситель опускается в эллиптическое днище реактора без закрутки (рассмотрены режимы работы в 29, 28, 27, 26, 25, 24, 22 топливные кампании при различном составе выемных частей ГЦН). На блоке 2 происходит разворот теплоносителя на опускном участке входной камеры реактора на 30-40 градусов против часовой стрелки. Характер течения теплоносителя в опускном кольцевом участке входной камеры реактора блока 3 подобен течению теплоносителя на блоке 1. На блоке 4 закрутка теплоносителя в опускной камере реактора достигает 60-80 градусов против часовой стрелки. Для блока 4 данные результаты хорошо коррелируют с данными испытаний, проведенными в ПНР

Correlation measurements of the primary coolant flowrate by nitrogen-16 activity at Kalinin NPP

S21

Bogachek, L.N.; Kuz'min, V.V. (Kalininskaya AEhS, Udomlya (RU))

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Korrelyatsionnye izmereniya raskhoda teplonositelya pervogo kontura po aktivosti "1"6N na Kalininskoj AEhS

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bezopasnosti AEhS s VVEhR. Nauchno-tehnicheskoe ehlektronnoe izdanie

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vp.

KALININ-2 REACTOR; PRIMARY COOLANT CIRCUITS; NITROGEN 16; FLUID FLOW; COOLANT LOOPS; CORRECTIONS; MASS TRANSFER; REACTOR SAFETY; REACTOR CONTROL SYSTEMS

Analysis of the data from Kalinin Unit 2 shows that the main benefits of the correlation method of flow measurement are the following: potential for keeping the inaccuracy of primary flow measurements on the level of 2% which is much lower than the factual inaccuracy of measurements on the pressure drop in MCP and design head-flow characteristics, using sensors located inside the pipeline, extremely high reliability and failure-free performance of the sensors and secondary equipment of the measurement system. As sensors are not directly connected to the primary circuit pipelines, the system can be easily installed on the operating units and on the newly commissioned WWER units. The results of long-term operation which show that along with the correlation method of coolant flow rate detection measuring of gamma activity ^{16}N can be used for the on-line control of the core thermal power

Анализ данных, полученных на блоке 2 Калининской АЭС, показывает, что основными преимуществами корреляционного метода измерения расхода являются: возможность обеспечить погрешность измерения расхода в I контуре на уровне 2% , что значительно ниже, чем реальная погрешность его измерения по перепаду давления на ГЦН и паспортным напорным характеристикам, использование датчиков, устанавливаемых снаружи трубопровода, исключительно высокая надежность и безотказность датчиков и вторичной аппаратуры измерительной системы. Тот факт, что датчики не имеют непосредственного контакта с трубой I контура, позволяет сравнительно легко монтировать систему как на вновь вводимых, так и на действующих блоках с реакторами типа ВВЭР. Представлены результаты длительной эксплуатации, показывающие, что, помимо определения расхода теплоносителя корреляционным методом, измерение γ -активности азота-16 может быть использовано для оперативного контроля тепловой мощности петель РУ

Perfecting of means and technologies of eddy current testing of WWER NPP equipment

S21

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Sovershenstvovanie sredstv i tekhnologij vikhretokovogo kontrolya oborudovaniya AEhS s VVEhR

vp.

(RU)

4 refs., 8 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; REACTOR CONTROL SYSTEMS; REACTOR COMPONENTS;
EDDY CURRENT TESTING; STEAM GENERATORS; REACTOR SAFETY;
COMPUTERIZED CONTROL SYSTEMS

Last developments of specialized scanning devices and eddy current probes executed by COMVIS Company essentially expand an application area of flaw detector COMVIS LM. Manipulator SKR-140/170 is intended for ECT of threaded holes on flanges of reactor housings WWER-1000 and WWER-440. Scanning device SKR-64N is intended for ECT of threaded and smooth surfaces of studs (up to M64). Rotating eddy current probe PNV-4 is intended for ECT of weld joints of heat-exchanging tubes and steam generator collector. COMVIS LM software provides full interacting of flaw detector with scanning devices

Последние разработки специализированных сканирующих устройств и вихретоковых преобразователей, выполненные компанией КОМВИС, существенно расширяют область применения дефектоскопа КОМВИС ЛМ. Манипулятор СКР-140/170 предназначен для ВТК резьбовых отверстий на фланцах корпусов реакторов ВВЭР-1000 и ВВЭР-440. Сканирующее устройство СКР-64Н предназначено для ВТК резьбовой и гладкой поверхностей шпилек (до М64). Вращающийся вихретоковый преобразователь ПНВ-4 предназначен для ВТК сварных швов приварки теплообменных труб к коллектору парогенератора. Программные средства КОМВИС ЛМ обеспечивают полное взаимодействие дефектоскопа со сканирующими устройствами

On extension of information and diagnostic functions of thermal control system of NPS with WWER

S21

Prijmak, S.V.; Zajtsev, P.A.; Georgievskij, V.N.; Mel'nikov, G.N.; Usachev, V.B.; Fomenko, V.V. (FGUP #Left Double Quotation Mark#NII NPO #Left Double Quotation Mark#LUCh#Right Double Quotation Mark#, Podol'sk (RU))

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AEhS s VVEhR

O rasshirenii informatsionno-diagnosticheskikh funktsij sistemy termokontrolya AEhS s VVEhR
vp.

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7 refs., 4 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern ROsehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; REACTOR SAFETY; REACTOR CONTROL SYSTEMS; THERMOCOUPLES; TEMPERATURE MEASUREMENT; NONDESTRUCTIVE TESTING; REGRESSION ANALYSIS

the results of design and testing of a device for commissioning measurements and operational diagnostics of thermocouples, research on operational factors influence on TII, practicing of a method of thermocouples installation quality express-diagnostics during periodic large scale replacements under PPM, research on the accuracy of the accounting method of a thermocouple radiation heating up, development of a calculation model for diagnostics of the coolant temperature ripple amplitude are presented. Accounting of thermocouples TII at measurement reference will ensure realization of functional capabilities of the WWER thermal control system, including reliability of data on temperature margin in case of NPS energy extension

Представлены результаты разработки и испытаний прибора для пусконаладочных измерений и эксплуатационной диагностики ПТИ термодатчиков, исследований влияния эксплуатационных факторов на ПТИ, отработки способа экспресс-диагностики качества монтажа термодатчиков при периодической массовой замене в ходе ППР, исследований точности методики учета радиационного разогрева термодатчика, разработки расчетной модели диагностики амплитуд пульсаций температуры теплоносителя. Учет ПТИ термодатчиков в измерительных позициях обеспечит реализацию функциональных возможностей системы термоконтроля ВВЭР, в том числе достоверность данных запаса по температуре при повышении мощности АЭС

Approaches for accounting and prediction of fast neutron fluence on WWER pressure vessels and results of validation of calculational procedure

S21

Borodkin, P.G.; Khrennikov, N.N. (FBU #Left Double Quotation Mark#NTTs YaRB#Right Double Quotation Mark#, Moscow (RU)); Ryabinin, Yu.A. (AO #Left Double Quotation Mark#Kontsern ROsehnergoatom#Right Double Quotation Mark#, Moscow (RU)); Adeev, V.A. (Kol'skaya AEhS, Polyarnye Zori (RU))

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AEhS s VVEhR

Podkhody po uchetu i kontrolyu flyuensa bystrykh nejtronov na korpusakh reaktorov VVEhR i rezul'taty testirovaniya protsedury raschetnogo opredeleniya flyuensa

vp.

(RU)

9 refs., 6 figs., 1 tab.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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KOLA-2 REACTOR; KOLA-3 REACTOR; NEUTRON FLUENCE; REACTOR VESSELS; REACTOR CORES; REACTOR SAFETY; REACTOR KINETICS; REGULATIONS

A description is given of the universal procedure for calculation of fast neutron fluence (FNF) on WWER vessels. Approbation of the calculation procedure was carried out by comparing the calculation results for this procedure and measurements on the outer surface of the WWER-440 and WWER-1000 vessels. In addition, an estimation of the uncertainty of the settlement procedure was made in accordance with the requirements of regulatory documents. The developed procedure is applied at Kola NPP for independent fast neutron fluence estimates on the WWER-440 reactor vessels when planning core loads taking into account the introduction of new fuels. The results of the pilot operation of the procedure for calculating FNF at the Kola NPP were taken into account when improving the procedure and its application to the calculations of FNF on the WWER-1000 vessels

Приведено описание универсальной процедуры расчетов флюенса быстрых нейтронов (ФБН) на корпусах ВВЭР. Апробация процедуры расчета проведена путем сравнения результатов расчета по данной процедуре и измерений на внешней поверхности корпусов ВВЭР-440 и ВВЭР-1000. Кроме того, проведена оценка неопределенности расчетной процедуры, в соответствии с требованиями нормативных документов. Разработанная процедура применяется на Кольской АЭС для независимых оценок флюенса быстрых нейтронов на корпусах реакторов ВВЭР-440 при планировании загрузок активной зоны с учетом внедрения новых видов топлива. Результаты опытной эксплуатации процедуры расчета ФБН на Кольской АЭС учтены при совершенствовании процедуры и ее применения к расчетам ФБН на КР реакторов ВВЭР-1000

Development of technique for determining the weighted average coolant temperature in the primary circuit hot legs of the WWER-1000

S21

Saunin, Yu.; Dobrotvorskiy, A.N.; Semenikhin, A.V. (AO #Left Double Quotation Mark#Atomtekhenergo#Right Double Quotation Mark#, Novovoronezhskij Filial,

Novovoronezh (RU)

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AEhS s VVEhR

Razrabotka metodiki opredeleniya srednevzveshennoj temperatury teplonosatelya v goryachikh nitkakh petel' pervogo kontura RU s VVEhR-1000

vp.

(RU)

23 refs., 4 figs., 1 tab.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; PRIMARY COOLANT CIRCUITS; COOLANT LOOPS; TEMPERATURE CONTROL; REACTOR SAFETY; TEMPERATURE DISTRIBUTION; ACCURACY

Development of industry regulations #Left Double Quotation Mark#The Technique for determining the weighted average coolant temperature in the primary circuit hot legs of the WWER-1000#Right Double Quotation Mark# performed by the task of JSC #Left Double Quotation Mark#Concern Rosenergoatom#Right Double Quotation Mark#, is associated with the need to adopt organizational and technical measures to improve the accuracy of operational determining the coolant temperature in the primary circuit hot legs. The basic preconditions and approaches to the Technique development are presented. They are developed on the basis of the executed numerical and experimental researches taking into account the analysis of the extensive material received at carrying out full-scale tests at commissioning of WWER-1000 power units, and of operational data obtained from several power units with different fuel loadings

Разработка отраслевого нормативного документа #Left Double Quotation Mark#Методика определения средневзвешенной температуры теплоносителя в горячих нитках петель первого контура реакторных установок с ВВЭР-1000#Right Double Quotation Mark#, выполняемая по заданию АО #Left Double Quotation Mark#Концерн Росэнергоатом#Right Double Quotation Mark#, связана с необходимостью принятия организационных и технических мер для повышения точности оперативного определения температуры теплоносителя в горячих нитках петель первого контура. Представлены основные предпосылки и подходы к разработке Методики, выработанные на основании выполненных расчетно-экспериментальных исследований с учетом анализа обширного материала, полученного при проведении натурных испытаний при вводе в эксплуатацию энергоблоков с ВВЭР-1000 разных проектов, и эксплуатационных данных по отдельным топливным кампаниям на нескольких энергоблоках

Problems of exploitation of the deaerator system recharge and boron regulation with packed columns

S21

Litvinenko, L.D.; Mitryukhin, A.G.; Amelyushina, A.G. (AO #Left Double Quotation Mark#ATOMPROEKT#Right Double Quotation Mark#, Sankt-Peterburg (RU))

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Problemy ehkspluatatsii deaeratorov sistemy podpitki i bornogo regulirovaniya s nasadochnymi kolonnami

vp.

(RU)

5 refs., 4 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; FEEDWATER; DEAERATORS; WATER TREATMENT; THERMAL HYDRAULICS; PRIMARY COOLANT CIRCUITS; REACTOR SAFETY; NOZZLES

There is reason to suspect that the design of the recharge and boron regulation deaerators does not provide the necessary reserves to flood. Analysis of operation and calculations performed using the thermo-hydraulic code KORSAR/B3 allow to offer the following solutions: switch to another type of unordered nozzles elements, for example, omega-shaped elements, calculation methods are available, divide the flow of gas mixture (steam) as follows: 30 kg/h through a pressure plate of small nozzles, 100 kg/h of a large nozzle volume. The decisions must be justified and confirmed by the experimental studies

Анализ эксплуатации и проведенные расчеты с помощью теплогидравлического кода КОРСАР/В3 позволяют предложить следующие пути решения проблемы: перейти на другой тип элементов неупорядоченных насадок, например, на омегаобразные, методики расчета которых имеются, разделить расход парогазовой смеси (выпар): 30 кг/ч - через напорную тарелку малой насадки, 100 кг/ч - из объема большой насадки. Принятые решения должны быть обоснованы и подтверждены экспериментальными исследованиями

WANO-MC Regional Crisis Center for nuclear power plants with WWER reactors

S21

Loktionov, S.A. (Moskovskij Tsentr VAO AEhS, Moscow (RU))

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Regional'nyj krizisnyj tsentr AS s rektorami VVEhR Moskovskogo tsentra VAO AEhS
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5 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; REACTOR SAFETY; NUCLEAR POWER PLANTS; RUSSIAN ORGANIZATIONS; REGIONAL COOPERATION; MANAGEMENT; DECISION MAKING; OPTIMIZATION

The purpose and the main task of the RCC is to provide advice and technical assistance in the event of a site area emergency, general site emergency at WANOMC WWER plants as well as spreading the information on safety relevant events at NPPs among its members. WANO-Moscow Centre#Right Single Quotation Mark#s Regional Crisis Center, has agreed on co-operation based on following principles: 1. Continuous Readiness; 2. Information Flow Optimization; 3. Prompt Notification; 4. Confidentiality; 5. Expert Support; 6. Providing Logistical, Material and Technical Support; 7. Using Accumulated Knowledge; 8. Conducting Exercises and Drills; 9. RCC arranges regular emergency exercises and drills; 10. Voluntariness

Цель и основная задача РКЦ заключается в предоставлении консультаций и технической помощи в случае чрезвычайной ситуации на площадке, общей аварийной ситуации на объектах WANOMC WWER, а также распространения информации о событиях, связанных с безопасностью на АЭС, среди ее членов. Региональный кризисный центр ВАО АЭС-Москва согласовал сотрудничество на основе следующих принципов: 1. Непрерывная готовность; 2. Оптимизация информационного потока; 3. Оперативное оповещение; 4. Конфиденциальность; 5. Экспертная поддержка; 6. Обеспечение логистической и материально-технической поддержки; 7. Использование накопленных знаний; 8. Проведение тренингов и учебно-тренировочных занятий; 9. Организация РСС регулярных экстренных учений и тренировок; 10. Добровольность

Automated process safety parameters monitoring system

S21

Iyudina, O.S.; Solov'eva, A.G.; Syrov, A.A. (ZAO #Left Double Quotation Mark#Diakont#Right Double Quotation Mark#, Sankt-Peterburg (RU))

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Avtomatizirovannaya sistema monitoringa parametrov bezopasnosti tekhnologicheskikh protsessov

vp.

(RU)

2 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; NUCLEAR POWER PLANTS; REACTOR SAFETY; REACTOR CONTROL SYSTEMS; REACTOR MONITORING SYSTEMS; AUTOMATION; REACTOR OPERATION

Basing on the expertise in upgrading and creation of control systems for NPP process equipment, #Left Double Quotation Mark#Diakont#Right Double Quotation Mark# has developed the automated process safety parameters monitoring system project. The monitoring system is a set of hardware, software and data analysis tools based on a dynamic logical-and-probabilistic model of process safety. The proposed monitoring system can be used for safety monitoring and analysis of the following processes: reactor core reloading; spent nuclear fuel transfer; startup, loading, on-load operation and shutdown of an NPP turbine

ЗАО #Left Double Quotation Mark#Диаконт#Right Double Quotation Mark# на основе опыта модернизации и создания систем управления технологического оборудования АЭС разработан проект автоматизированной Системы мониторинга параметров безопасности технологических процессов. Система мониторинга представляет собой комплекс программно-технических и информационно-аналитических средств, основой которого является динамическая логико-вероятностная модель безопасности технологического процесса. Предлагаемая Система мониторинга может быть использована для контроля и анализа безопасности следующих технологических процессов: перегрузка активной зоны реактора; транспортирование отработавшего ядерного топлива; пуск, нагружение, работа на мощности и останов турбины АЭС

Use of acoustic anisotropy parameter for the analysis of damage accumulation in the area of SW111 during production and operation of steam generators PGV-1000

S21

Kamyshev, A.V.; Pasmanik, L.A.; Smirnov, V.A. (OOO #Left Double Quotation Mark#INKOTES#Right Double Quotation Mark#, Nizhnij Novgorod (RU)); Modestov, V.S.; Pivkov, A.V. (FGBOU VPO SPbGPU, Sankt-Peterburg (RU))

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Ispol'zovanie parametra akusticheskoy anizotropii dlya analiza nakopleniya povrezhdenij v zone SS111 pri izgotovlenii i ehkspluatatsii parogeneratorov serii PGV-1000

vp.

(RU)

4 refs., 5 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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ACOUSTIC MONITORING; REACTOR INSTRUMENTATION; REACTOR MONITORING SYSTEMS; WWER TYPE REACTORS; ANISOTROPY; REACTOR SAFETY; STRESS ANALYSIS; HIGH ALLOY STEELS; STEAM GENERATORS

Results of experimental application of the acoustoelasticity method for evaluation of the metal condition at the junction of fluid collector with pipe DN1200 (JCP) of steam generator PGV-1000 are presented. This node is characterized by tendency for cracks appearance in the heat-affected zone of the seam welding of the collector to the inlet of the steam generator (SW111). Presented results of laboratory studies of the dependence of intrinsic acoustic anisotropy on the degree of plastic deformation and the presence of scattered micro-damage in steel 10MnNi2MoVN, which is used to produce JCP. It is shown that if intrinsic acoustic anisotropy value exceeds the threshold value of 0.7%, then damage accumulation to the metal structure can lead to appearance of macro cracks with substantial depth. It is shown that for those JCP, in which large tensile stresses under thermal and force loading were identified by use of calculated-instrumental method based on the definition of force boundary conditions by the method of acoustoelasticity, the values of the intrinsic acoustic anisotropy of the metal in the zone above the SW111 indicates the presence of significant signs of damage accumulation

Представлены результаты опытного применения метода акустоупругости для оценки состояния металла узла сопряжения коллектора теплоносителя (УСКП) с патрубком Ду1200 парогенераторов ПГВ-1000. Выбор был обусловлен склонностью узла к трещинообразованию в зоне термического влияния шва приварки коллектора к патрубку парогенератора (СС #Numero Sign#111). Показаны результаты лабораторных исследований зависимости величин собственной акустической анизотропии от степени пластической деформации и наличия рассеянных микроповреждений в стали 10ГН2МФА, из которой изготавливаются УСКП. Показано, что при превышении собственной акустической анизотропии 0,7% накопленные повреждения структуры металла могут привести к появлению макротрещин значительной глубины. Показано, что для тех УСКП, в которых расчетно-инструментальным методом, основанном на определении силовых граничных условий методом акустоупругости, выявлены большие растягивающие напряжения при термосиловом нагружении, значения собственной акустической анизотропии металла в зоне выше СС #Numero Sign#111 указывают на наличие существенных признаков накопления повреждений

Evaluation of stress-strain state JCP under thermal and force loading of steam generators PGV-1000 by means of the calculated-instrumental method with the definition of force boundary conditions by the acoustoelasticity method

S21

Kamyshev, A.V.; Pasmanik, L.A.; Smirnov, V.A. (OOO #Left Double Quotation Mark#INKOTES#Right Double Quotation Mark#, Nizhnij Novgorod (RU)); Modestov, V.S.; Pivkov, A.V. (FGBOU VPO #Left Double Quotation Mark#SPbGPU#Right Double Quotation Mark#, Sankt-Peterburg (RU))

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AEhS s VVEhR

Otsenka napryazhenno-deformirovannogo sostoyaniya USKP pri termosilovom nagruzhenii parogeneratorov serii PGV-1000 raschetno-instrumental'nym metodom s opredeleniem silovykh granichnykh usloviy metodom akustouprugosti

vp.

(RU)

4 refs., 10 figs., 2 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; STEAM GENERATORS; DYNAMIC LOADS; STRESSES; DEFORMATION; MECHANICAL TESTS; ULTRASONIC TESTING

The numerical model in terms of the description of the interactions of the steam generator elements under thermal and force loading and take into account these interactions for evaluation of the stress-strain state of junction of collector fluid with pipe DN1200 (JCP) are determined. As it follows from results obtained, under thermal and force loading of JCP additional axial forces and moments occurs, indicating the presence of interactions between steam generator structural elements, which are not fully taken into calculation model. Research on operating blocks showed a significant difference between thermal and force loads for JCP of different steam generators, which correlates well with the history of exploitation and damageability

Уточнялись расчетная модель в части описания взаимодействия элементов парогенератора при термосиловом нагружении и учет этих взаимодействий при оценке напряженно-деформированного состояния узла сопряжения коллектора теплоносителя (УСКП). Из полученных результатов следует, что при термосиловом нагружении в УСКП возникают дополнительные осевые силы и моменты, указывающие на наличие взаимодействий элементов конструкции парогенератора, не полностью учитываемых расчетной моделью. Исследования на эксплуатируемых блоках показали существенное различие термосиловых нагрузок для УСКП разных парогенераторов, которое хорошо коррелирует с историей эксплуатации и повреждаемостью

Calculation of steam-gas mixture parameters in WWER-1000/V-320 containment during severe accident taking into account operation of filtered venting system

S21

Zvonarev, Yu.A.; Budaev, M.A.; Kobzar', V.L.; Konobeev, A.V.; Shmel'kov, Yu.B. (NITs #Left Double Quotation Mark#Kurchatovskij Inst.#Right Double Quotation Mark#, Moscow (RU))

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Raschet parametrov parogazovoj sredy v zashhitnoj obolochke VVEhR-1000/V-320 pri tyazhelej zaproektnoj avarii s uchetom raboty sistemy avariynogo fil'truemogo sbrosa gazov vp.

(RU)

3 refs., 15 figs., 2 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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BALAKOVO-4 REACTOR; REACTOR SAFETY; CONTAINMENT SHELLS; REACTOR COOLING SYSTEMS; FILTRATION; GAS DISCHARGE TUBES; LOSS OF COOLANT; BLOWDOWN

The considered accident is a double-ended break of the cold leg with equivalent diameter of 850 mm accompanied with simultaneous total loss of power supply. The break is situated near the reactor inlet. No operator actions are assumed. Calculation analysis of the processes in reactor and containment were performed by SOKRAT V.1 and ANGAR codes. Containment of Unit #Numero Sign# 4 Balakovo NPP was used for calculation of steam-gas mixture parameters in containment during severe accident. For specified algorithm of operating of filtered venting system, parameters of steam-gas mixture in containment and decay heat capacity of fission products retained by this system were defined

Рассматривалась авария с разрывом ГЦТ Ду 850 мм полным сечением на входе в реактор, с одновременным отказом всех источников переменного тока, включая дизель-генераторы, на длительный период без вмешательства оперативного персонала. Расчетный анализ процессов в РУ и в ЗО проводился с использованием кодов СОКРАТ В.1 и АНГАР. Расчет параметров парогазовой среды в ЗО при данной аварии проводился на примере ЗО энергоблока #Numero Sign# 4 Балаковской АЭС. Для заданного алгоритма работы системы аварийного фильтруемого сброса газов определены параметры парогазовой среды в ЗО и мощность остаточного энерговыделения ПД, поступающих в систему сброса

A computational and experimental technique of critical local cleavage stress determination for different brittle fracture origins in SE(B) type specimens

S21

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9-ya mezhdunarodnaya nauchno-tehnicheskaya konferentsiya Obespechenie bezopasnosti AEhS s VVEhR

Raschetno-ehksperimental'noe opredelenie kriticheskogo lokal'nogo napryazheniya otrывa dlya razlichnykh istochnikov khрупкого razrusheniya v obraztsakh tipa SE(B)

vp.

(RU)

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; REACTOR COMPONENTS; FRACTURE PROPERTIES;
CRACKS; STRESSES; DEFORMATION; TIME DEPENDENCE; CLEAVAGE

Fractographic studies of the tested SE(B) specimens allow to reveal types and locations of brittle fracture origins in samples. For calculation of specimen stress-strain state in terms of normal stress components, stress and plastic strain intensities a computational and experimental technique comprising the finite element models of the described samples was developed. Based on this model the critical local cleavage stress (local criterion for brittle fracture) determination method was created. It was found that the typical values of critical local cleavage stress in case of structural boundaries as origin for irradiated samples are achieved for smaller times (#Swung Dash#140 000 h) compared to the specimens after only thermal exposure where the same values are achieved over larger times (#Swung Dash#200 000 h)

Проведение фрактографических исследований испытанных образцов типа SE(B) позволяет выявить типы, а также расположение источника зарождения хрупкой трещины в образце. Для расчета напряженно-деформированного состояния образца в виде распределений нормальных компонентов напряжений, интенсивности напряжений и интенсивности пластических деформаций была разработана расчетно-экспериментальная модель, содержащая конечноэлементные модели описанных выше образцов. На основе этой модели была создана методика определения величины критического локального напряжения отрыва - локального критерия хрупкого разрушения образца. Установлено, что в облученных образцах значения критического локального напряжения отрыва, характерные для структурных границ, достигаются за меньшие времена (~140 000 ч), в то время как в образцах после только термического воздействия такие же значения достигаются за большие времена (200 000 ч)

Dependence of limits on activity of fission products in the primary coolant of WWER-1200, established for modes of normal operation and category 2, from limiting sizes of unorganized coolant leak in containment and steam generator S21

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Zavisimost' predelov po aktivnosti produktov deleniya v teplonositele pervogo kontura VVEhR-1200, ustanavlivaemykh dlya rezhimov NEh i kategorii 2, ot predel'nykh velichin neorganizovannykh protechek pervogo kontura v GO i PG

vp.

(RU)

2 refs., 3 figs., 8 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO

ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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FISSION PRODUCTS; WWER TYPE REACTORS; PRIMARY COOLANT CIRCUITS; LEAKS; FISSION PRODUCT RELEASE; REACTOR SAFETY; RADIATION PROTECTION

The establishment in the design not enough rigid requirements regarding safety limits on unorganized leaks involves increase of requirements to reliability of fuel rods regarding their tightness to provide performance of acceptance radiating criteria. Excessively rigid regulation of limits for depressurization of fuel rods can lead in practice to the raised restrictions of operation of the power unit and decrease its capacity operating ratio, and also occurrence of additional complexities at implementation of new kinds of nuclear fuel and fuel cycles. The interrelation of limits of safety on leaks from the first circuit in in volume of containment and steam generator with limits on activity of FP in primary coolant in modes of normal operation and categories 2 is illustrated on an example of the AES-2006 with two-units with WWER-1200

Установление в проекте недостаточно жестких требований в части пределов безопасности по неорганизованным протечкам влечет за собой повышение требований к надежности твэлов в части их герметичности для того, чтобы обеспечить выполнение приемочных радиационных критериев. Чрезмерно жесткое регламентирование пределов разгерметизации твэлов может на практике привести к повышенным ограничениям эксплуатации энергоблока и снижению его КИУМ, а также появлению дополнительных сложностей при внедрении новых видов топлива и топливных циклов. Взаимосвязь пределов безопасности по протечкам из первого контура в объем ГО и ПГ с пределами по активности ПД в ТПК в режимах нормальной эксплуатации и категории 2 иллюстрируется на примере двухблочной АЭС-2006 с ВВЭР-1200

Accounting of the knowledge-based actions and the rules-based actions in frames of accident management guidelines development

S21

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Uchet dejstvij, osnovannykh na znaniyakh, i dejstvij, osnovannykh na pravilakh pri razrabotke rukovodstv po upravleniyu avariymi

vp.

(RU)

6 refs., 2 figs., 1 tab.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; ACCIDENT MANAGEMENT; REACTOR ACCIDENTS; RADIATION PROTECTION; REACTOR SAFETY; RELIABILITY; REACTOR OPERATORS; HUMAN FACTORS; OCCUPATIONAL SAFETY; RECOMMENDATIONS

The main approaches used in the development of the Safety Guide (SG) #Left Double Quotation Mark#Recommendations to the structure and content of the manual for the management of beyond-design-basis accidents, including severe accidents#Right Double Quotation Mark# (BDBA MG) are described. The manual was developed taking into account the provisions of the current IAEA standards relevant to the affected area, taking into account the specifics of the Russian nuclear power industry. In the draft SG, three types of behavior of personnel are considered - based on skills, rules and knowledge. When developing BDBA MG, it is recommended to give priority to a knowledge-based approach. At the same time, when performing well-designed and worked-out activities, work is possible based on rules and skills (for example, using step-by-step procedures). The SG project provides for a unified organizational structure for managing beyond-design-basis accidents, both at the stage of preventing severe damage to the core, and at the stage of managing a heavy accident. In SG the order of management of beyond-design-basis accidents for both of the indicated stages examined in detail

Описаны основные подходы, использовавшиеся при разработке Руководства по безопасности (РБ) #Left Double Quotation Mark#Рекомендации к структуре и содержанию руководства по управлению запроектными авариями, в том числе тяжелыми авариями#Right Double Quotation Mark# (РУЗА). Руководство разработано с учетом положений современных стандартов МАГАТЭ, относящихся к затрагиваемой области, учитывает специфику российской атомной энергетики. В проекте РБ рассмотрены три типа поведения персонала - основанные на навыках, правилах и знаниях. При разработке РУЗА рекомендовано отдавать приоритет подходу, основанному на знаниях. В то же время, при выполнении хорошо прогнозируемых и отработанных действий, допускается работа на основе правил и отработанных навыков (например, с использованием пошаговых процедур). Проект РБ предусматривает единую организационную структуру управления запроектными авариями как на стадии предотвращения тяжелого повреждения активной зоны, так и на стадии управления тяжелой аварией. В РБ подробно рассмотрены особенности порядка управления запроектными авариями для обеих из указанных стадий

To the question on vibration strength of the main circulating pumps of reactor

WWER-1000 at influence of the non-stationary hydrodynamic stream

S21

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AEhS s VVEhR

K voprosu o vibroprochnosti glavnykh tsirkulyatsionnykh nasosov reaktora VVEhR-1000 pri vozdejstvii nestatsionarnogo gidrodinamicheskogo potoka

vp.

(RU)

20 refs., 4 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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vp.

WWER TYPE REACTORS; CENTRIFUGAL PUMPS; HYDRODYNAMICS; DYNAMIC LOADS; MATHEMATICAL MODELS

The problem of vibration strength of working shovels of centrifugal pumps is based on definition of dynamic behaviour of shovels on the basis of the theory of flat covers, thus integration of the initial equations of movement is carried out by the modified method consecutive approach. The problem of forced and natural fluctuations of a shovel is solved, which makes it possible to identify the resonant zones of operation of centrifugal pumps of nuclear power, in particular, the main circulation pump GTSN 195M

Проблема вибропрочности рабочих лопаток центробежных насосов основана на определении динамического поведения лопаток на основе теории пологих оболочек, при этом интегрирование исходных уравнений движения осуществляется модифицированным методом последовательных приближений. Решается задача о вынужденных и собственных колебаниях лопатки, что позволяет выявить резонансные зоны работы центробежных насосов ядерной энергетики, в частности, главного циркуляционного насоса ГЦН 195М

Monitoring of neutron field parameters of pressure vessels of russian WWER in compliance with regulatory demands

S21

Borodkin, P.G.; Khrennikov, N.N. (FBU #Left Double Quotation Mark#NTTs YaRB#Right

Double Quotation Mark#, Moscow (RU)); Miroshnichenko, M.I. (Rostekhnadzor, Moscow (RU))

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Raschetno-ehksperimental'nyj monitoring kharakteristik polej nejtronov v korpusakh reaktorov VVEhR s uchetom trebovanij ND

vp.

(RU)

12 refs., 5 figs., 2 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; REACTOR CORES; NEUTRON FLUENCE; CALCULATION METHODS; REACTOR MONITORING SYSTEMS; RECOMMENDATIONS; REACTOR SAFETY

Results of finished calculational-experimental measurements which are performed on the number of Russian power units with WWER-440 and WWER-1000 are presented. The analysis of core power distribution of WWER-1000 and its influence on measurements and calculations of the integral through-vessel neutron leak-age are proposed. It is shown how to get improved estimation of parameters with reasonable uncertainty from actual reactor data (core operational parameters (from in-core monitoring system) and experimentally evaluated the integral leakage at the reactor vessel (neutron activation measurements). In the present paper discusses the results of such investigations and gives the conclusions on the necessity and sufficiency of monitoring of neutron field parameters on RPV

Представлены результаты завершенных расчетно-экспериментальных исследований на ряде российских энергоблоков с ВВЭР-440 и ВВЭР-1000. Также представлены результаты анализа распределения энерговыделения по объему активной зоны ВВЭР-1000 и его влияние на результаты измерения и расчета интегральной утечки нейтронов за корпусом реактора. Показано, что можно получить улучшенную оценку параметров с обоснованной неопределенностью с использованием фактических реакторных данных (эксплуатационных характеристик активной зоны (измеренных СВРК) и экспериментально оцененной интегральной утечки за корпус реактора (нейтронно-активационные измерения)). Обсуждаются результаты таких исследований, а также сделаны выводы о необходимости и достаточности мониторинга радиационной нагрузки корпусов

Experience of works on application of system on-line monitoring of operational damageability of metal in the most critical zones of the NPP main equipment by the example of WJ #Numero Sign#111 of power unit #Numero Sign#5 of Novovoronezh NPP

S21

Bakirov, M.B.; Eremin, A.A.; Levchuk, V.I.; Nikolaev, D.A. (OOO #Left Double Quotation Mark#Nauchno-Sertifikatsionnyj Uchebnyj Tsentr Materialovedeniya i Resursa Komponentov Yadernoj Tekhniki #Left Double Quotation Mark#Tsentr Materialovedeniya i Resursa#Right Double Quotation Mark#, Moscow (RU)); Povarov, V.P.; Gromov, A.F.; Urazov, O.V. (Filial OAO #Left Double Quotation Mark#Kontsern Rosehnergoatom#Right Double Quotation Mark# #Left Double Quotation Mark#Novovoronezhskaya Atomnaya Stantsiya#Right Double Quotation Mark#, Novovoronezh (RU))

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Opyt rabot po ispol'zovaniyu sistemy nepreryvnogo monitoringa ehkspluatatsionnoj povrezhdaemosti metalla v naibolee kriticheskikh zonakh otvetstvennogo oborudovaniya AEhS na primere SS #Numero Sign#111 ehnergobloka #Numero Sign#5 Novovoronezhskoj AEhSV vp.

(RU)

17 refs., 16 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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NOVOVORONEZH-5 REACTOR; WEAR; REACTOR SAFETY; WELDED JOINTS; STEAM GENERATORS; REACTOR MONITORING SYSTEMS; ACOUSTIC MONITORING; ULTRASONIC MACHINING

The system of continuous monitoring of operational damage includes two independent modules: the ultrasonic monitoring of the selected potentially dangerous defect (tracking the most informative parameter of ultrasonic testing - the amplitude of the echo signal from the defect, depending on the operating loading factors with the purpose of assessing the dynamics of the development of the crack at the stage of start-up, operation and stop); acoustic emission antenna for monitoring the moment of crack initiation in the pocket area along the perimeter of the steam generator tube. The results on practical application of the WJ #Numero Sign#111 monitoring system mounted at steam generators on Unit 5 of Novovoronezh NPP, gained during three fuel campaigns, have shown the high effectiveness of the accepted technical decisions. Executed

works have allowed to provide the control of actual defectiveness and to implement the on-line monitoring of metal integrity with the purpose of exclusion of through-wall defects formation

Система непрерывного мониторинга эксплуатационной повреждаемости включает в себя два независимых модуля: ультразвукового контроля выбранного потенциально опасного дефекта (слежение за наиболее информативным параметром УЗ контроля - амплитудой эхо-сигнала от дефекта в зависимости от действующих нагружающих факторов с целью оценки динамики развития трещины на этапе пуска, эксплуатации и останова); акустико-эмиссионной антенны для контроля момента зарождения трещины в зоне кармана по всему периметру патрубка парогенератора. Полученные результаты по использованию системы мониторинга СС #Numero Sign#111 ПГ 5 блока НВАЭС в течение трех топливных компаний показали высокую эффективность принятых технических решений, позволили обеспечить контроль за фактической дефектностью, организовать мониторинг целостности с целью недопущения сквозных повреждений

Implementation of on-line multiparameter monitoring of operational damageability of WJ #Numero Sign#111 by the example of power unit #Numero Sign#5 of Novovoronezh NPP

S21

Bakirov, M.B.; Levchuk, V.I. (OOO #Left Double Quotation Mark#Nauchno-Sertifikatsionnyj Uchebnyj Tsentri Materialovedeniya i Resursa Komponentov Yadernoj Tekhniki #Left Double Quotation Mark#Tsentri Materialovedeniya i Resursa#Right Double Quotation Mark#, Moscow (RU)); Povarov, V.P.; Gromov, A.F.; Urazov, O.V. (Filial OAO #Left Double Quotation Mark#Kontsern Rosehnergoatom#Right Double Quotation Mark# #Left Double Quotation Mark#Novovoronezhskaya Atomnaya Stantsiya#Right Double Quotation Mark#, Novovoronezh (RU))

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Organizatsiya nepreryvnogo mnogoparametricheskogo monitoringa ehkspluatatsionnoj povrezhdaemosti SS #Numero Sign#111 na primere ehnergobloka #Numero Sign#111 Novovoronezhskoj AEhS

vp.

(RU)

16 refs., 9 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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NOVOVORONEZH-5 REACTOR; REACTOR OPERATION; REACTOR SAFETY;
WELDED JOINTS; REACTOR MONITORING SYSTEMS; ACOUSTIC MONITORING;
ULTRASONIC TESTING; DATA COMPILATION

A new approach on on-line multiparameter monitoring of operational damageability of WJ #Numero Sign#111 during the whole fuel campaign is proposed and checked in practice. The approach is based on mutual monitoring of actual thermo-mechanical loads, monitoring of formation and growth of operational defects covering the whole WJ perimeter, as well as numerical computation of accumulated metal damage in the inspected zone using the array of experimental data collected during monitoring. With the purpose of collection of reliable experimental data the system of on-line monitoring of the WJ #Numero Sign#111 was installed on power unit #Numero Sign#5 of Novovoronezh NPP. The system implements collection and processing of data regarding the operational loading factors using the records of temperatures, deformations and displacements in checkpoints, as well as data of on-line monitoring of metal integrity by acoustic and ultrasonic methods

Предложен и апробирован новый подход по непрерывному многопараметрическому мониторингу эксплуатационной повреждаемости СС #Numero Sign#111 в течение всей топливной кампании, включающий мониторинг фактической термосиловой нагруженности, мониторинг за образованием и развитием эксплуатационных дефектов по всему периметру СС, а также численный расчет накопленного повреждения металла контролируемой зоны с использованием массива экспериментальных данных, записываемых в ходе мониторинга. С целью сбора достоверных экспериментальных данных на энергоблоке #Numero Sign#5 Нововоронежской АЭС была установлена система непрерывного мониторинга зоны СС #Numero Sign#111, осуществляющая сбор и анализ данных о нагружающих факторах по результатам измерений температур, деформаций и перемещений в контрольных точках, а также данных непрерывного контроля целостности металла акустическим и ультразвуковым методами

Dissimilar welded joints of equipment and pipelines of nuclear power plants. Status and prospects

S21

Khodakov, V.D.; Khodakov, D.V.; Bazanov, M.A.; Lukicheva, S.V. (AO NPO #Left Double Quotation Mark#TsNIITMASH#Right Double Quotation Mark#, Moscow (RU)); Gutsev, D.F. (AO #Left Double Quotation Mark#Kontsern Rosehnergoatom#Right Double Quotation Mark#, Moscow (RU))

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Raznorodnye svarnye soedineniya oborudovaniya i truboprovodov AEhS. Sostoyanie i perspektivy

vp.

(RU)

7 refs., 9 figs., 1 tab.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO

Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; WELDED JOINTS; AUSTENITIC STEELS; OUTAGES; RELIABILITY; REACTOR SAFETY; CORROSION; CORROSION PROTECTION

The production of NPP equipment inevitably becomes necessary to perform heterogeneous welded joints and, above all, welding parts of pearlite steels and austenitic. By heterogeneous welded joints may also qualify cladding corrosion of internal surfaces in contact with the coolant. When an outage at existing nuclear power plants are found damaged dissimilar weld joints of pipelines and equipment. Presented systematic information about the nature and cause of the aforementioned injuries, technology to eliminate them and measures to prevent their formation

При производстве атомноэнергетического оборудования неизбежно возникает необходимость выполнять разнородные сварные соединения и, прежде всего, сварку деталей из сталей перлитного и аустенитного класса. К разнородным сварным соединениям может быть также отнесена антикоррозионная наплавка внутренних поверхностей, контактирующих с теплоносителем. При проведении ППР на действующих АЭС обнаруживаются повреждения разнородных сварных соединений трубопроводов и оборудования. Представлены систематизированные сведения о характере и причине вышеуказанных повреждений, технологии их устранения и мероприятия по предотвращению их образования

Development of the manual ultrasonic inspection austenitic metal welds in injection and discharge pipelines of compensation pressure system and pressurizer WWER-1000

S21

Razygraev, A.N.; Razygraev, N.P.; Tsukanov, M.V. (AO NPO #Left Double Quotation Mark#TsNIITMAsh#Right Double Quotation Mark#, Moscow (RU))

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Razrabotka metodiki ruchnogo UZK austenitnykh svarnykh soedinenij truboprovodov vpryska i sbrosa sistemy kompensatsii davleniya i korpusa kompensatora davleniya RU VVEhR-1000

vp.

(RU)

4 refs., 8 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; ULTRASONIC TESTING; AUSTENITIC STEELS; WELDED JOINTS; CRACK PROPAGATION; PIPELINES; FLUID INJECTION; REACTOR OPERATION

The main problems encountered in UT austenitic metal welds (AMW) proved the possibility of head (creeping) waves and longitudinal waves with a large angle of probe for UT AMW, the results of the development and testing of specialized transducers for ultrasonic testing of specialized research and tuning samples, and the operation and investigated ways UT in monitoring and evaluating the quality are analyzed. Research on these samples showed the possibility of detection of crack defects that can occur when implementing the required sensitivity, and the possibility of fixing the coordinates of reflectors in welded joints with an accuracy required by the terms of reference. Developed #Left Double Quotation Mark#Ultrasonic testing procedure for pipeline injection and discharge and the cylindrical part of the pressurizer WWER-1000 Balakovo#Right Double Quotation Mark#

Проанализированы основные проблемы, возникающие при УЗК аустенитных сварных соединений (АСС), обоснована возможность применения головных волн и продольных волн с большим углом ввода для УЗК АСС, представлены результаты разработки и испытаний специализированных преобразователей УЗК на специализированных исследовательских и настроечных образцах, а также исследованы операции и способы УЗК при контроле и оценке качества. Исследования на данных образцах показали возможность обнаружения трещиноподобных дефектов, возможных при реализации требуемой чувствительности, а также возможность фиксации координат отражателей в сварных соединениях с погрешностью, требуемой техническим заданием. Разработана #Left Double Quotation Mark#Методика ультразвукового контроля трубопровода впрыска и сброса и цилиндрической части компенсатора давления РУ ВВЭР-1000 Балаковской АЭС#Right Double Quotation Mark#

Determination of steam flow rate in NPP pipelines by velocity gauge

S21

Boltenko, Eh.A.; Korol'kov, B.M.; Basov, A.V.; Davydov, M.V.; Kononenko, I.V. (Ehlektrogorskij Nauchno-Issledovatel'skij Tsentр po Bezopasnosti AEhS (AO #Left Double Quotation Mark#EhNITs#Right Double Quotation Mark#), Ehlektrogorsk (RU))

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Opreделение raskhoda para v paroprovodakh AEhS datchikami skorosti

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(RU)

6 refs., 9 figs., 2 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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BALAKOVO-3 REACTOR; FLOW RATE; VELOCITY; RELIABILITY; REACTOR MONITORING SYSTEMS; STEAM GENERATORS; REACTOR SAFETY; HYDRODYNAMICS

A procedure is proposed for determining the average velocity (flow rate) of water and/or steam for the case of essentially three-dimensional velocity profile in the pipe. The monitored system of steam flow rate measurement (SSRM) consists of three pressure tubes (PT). Monitoring of this system shows its functional reliability and, in particular, absence of erosion and mechanical failures of primary gauges. The signal lag of SSRM is less than 1-2 s. The quantitative differences of readings of pressure tubes located in the same pipeline cross-section were stated. Results of experiments modeling the Balakovo NPP steam pipelines are presented

Предложена методика определения средней скорости (расхода) воды и/или пара для случая существенно трехмерных профилей скорости в трубе. Приведены результаты мониторинга системы измерения расхода пара (СИРП) с тремя пневмометрическими трубками (ПТ). Результаты мониторинга показали, что в целом опыт работы свидетельствует о ее эксплуатационной надежности и, в частности, об отсутствии эрозионных и механических повреждений первичных датчиков. Время транспортного запаздывания СИРП составляет не более 1-2 с. Отмечены количественные различия величин скорости потока по показаниям ПТ, установленных в одном поперечном сечении паропровода. Представлены результаты экспериментальных исследований, моделирующих ситуацию, имеющую место на паропроводах Балаковской АЭС

Measurements of separation characteristics and thermal hydraulic parameters of the steam generator 4SG-2 in Balakovo nuclear power plant with WWER reactors by using a system for additional measurements under bringing the magnitude of 107-110% rated thermal power

S21

Kutdyusov, Yu.F.; Bud'ko, I.O.; Anurkin, R.P.; Khaziev, I.A. (OOO NITsEh #Left Double

Quotation Mark#Tsentrehnergo#Right Double Quotation Mark#, Moscow (RU)); Berkovich, V.Ya.; Sotskov, V.V.; Kharchenko, S.A.; Chuev, V.Yu.; Lakhov, D.A. (OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark#, Podol'sk (RU))

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Rezultaty izmerenij separatsionnykh i teplogidravlicheskih kharakteristik 4PG-2 Balakovskoj AEhS s ispol'zovaniem sistemy dopolnitel'nykh izmerenij v period osvoeniya teplovoj moshchnosti RU 107-110% ot nominal'noj

vp.

(RU)

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BALAKOVO-4 REACTOR; THERMAL HYDRAULICS; REACTOR SAFETY; STEAM GENERATION; EVAPORATION; COMPARATIVE EVALUATIONS; PERFORATION

4SG-2 and 4SG-3 separation tests were performed in Balakovo NPP generating unit 4 at the magnitudes of 75 and 80% rated thermal power of WWER reactor and under operation of three RCPs. The maximum permissible level is determined providing the steam generator (SG) output water ratio lower than 0,2%. A comparison is made of separation characteristics for 4SG-2 and 4SG-3 under various evaporative efficiencies. 4SG-2 thermal hydraulic parameters are determined (pressure drops on submerged perforated sheet, water ratio in SAM sample points) under various thermal power levels of nuclear reactor when separation tests were held. The calculated void fraction below submerged perforated sheet (SPS) is in range of 0-1. The load variation exceeds the calculated value of 1,2 in most relations between SG areas chosen to provide estimations that testifies insufficient pressure equalization of submerged perforated sheet with variable perforation when operated under higher evaporative efficiencies

Выполнены сепарационные испытания 4ПГ-2, 4ПГ-3 энергоблока 4 Балаковской АЭС при тепловой мощности 75%, 80% N-ном ВВЭР и трех работающих ГЦН, определен предельно-допустимый уровень воды, обеспечивающий влажность пара на выходе из ПГ не более 0,2%. Проведен сравнительный анализ сепарационных характеристик 4ПГ-2,3 при разных значениях паропроизводительности. Определены теплогидравлические характеристики 4ПГ-2 (перепады давления на ПДЛ, влажность во влагоотборниках СДИ) на различных уровнях тепловой мощности во время проведения сепарационных испытаний. Рассчитанное истинное объемное паросодержание под ПДЛ изменяется в диапазоне от 0 до 1. Величина рассчитанного коэффициента неравномерности в большинстве соотношений между выбранными для оценки зонами ПГ больше расчетной

величины 1,2, что свидетельствует о недостаточной выравнивающей способности ПДЛ с переменной перфорацией при работе на повышенном уровне паропроизводительности

Design and implementation of forced cooling towers for Loviisa NPP safety- and residual heat removal (RHR) cooling circuits

S21

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LOVIISA-1 REACTOR; LOVIISA-2 REACTOR; NUCLEAR POWER PLANTS; COOLING TOWERS; REACTOR SAFETY; VENTILATION SYSTEMS; HEAT EXCHANGERS; AFTER-HEAT REMOVAL; RHR SYSTEMS

The chosen solution was air-cooled cooling tower connected to the plant cooling systems for RHR and safety systems. The tower capacity and dimensions were iterated to fit the wide range of cooling and operational requirements. Towers are forced draught - type to keep the design compact and reliable. The heat exchangers are of water-to-air - type, consisting of finned tube bundles and connecting piping. The bundles are arranged in two-pass cross-counterflow - formation to allow compact connections with pipes and air fans. The towers are also weather protected to cope with the Finnish winter conditions. The requirements set by the postulated accidents in different operating stages and the arrangement of the RHR and fuel pool cooling led to a design solution of two separate towers for each unit. The capacity, location and the connections for the towers were designed to allow manual operation of the towers together with existing cooling circuits to fulfill the cooling task

Было принято решение, чтобы охлаждаемая воздухом градирня была связана с системами охлаждения систем безопасности и отвода остаточных тепловыделений.

Производительность градирни и ее габариты были вычислены методом последовательных приближений, чтобы выполнить широкий диапазон требований по охлаждению и

эксплуатации. Чтобы проект градирни был компактным и надежным, в нем используется принудительная вентиляция. Теплообменники водо-воздушного типа и состоят из пучков оребренных труб и соединительных трубопроводов. Пучки выполнены в два захода с противотоком - такой проект позволяет осуществить компактные связи с трубами и вентиляторами. Градирни также защищены от неблагоприятных погодных условий, чтобы выдержать финскую зиму. Требования, заданные постулированными авариями в различных режимах эксплуатации, компоновкой системы отвода остаточных тепловыделений и охлаждения топлива, привели к проектному решению в виде двух отдельных градирен для каждого блока

Key success factors for modern, digital I&C in nuclear power plants

S21

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DUKOVANY-1 REACTOR; DUKOVANY-2 REACTOR; DUKOVANY-3 REACTOR; DUKOVANY-4 REACTOR; KOZLODUY-3 REACTOR; KOZLODUY-4 REACTOR; ARMENIAN-2 REACTOR; LOVIISA-1 REACTOR; LOVIISA-2 REACTOR; COMPUTERIZED CONTROL SYSTEMS; REACTOR CONTROL SYSTEMS; REACTOR SAFETY; RELIABILITY; NUCLEAR POWER PLANTS

These can be divided into processes, technology and experience. Processes: life-cycle approach is mandatory to manage complexity, inherent quality assurance is a basic entry criteria, predictable process outcomes, stable and reasonable set of rules and standards are essential, early discussions between vendor, customer and nuclear regulator is essential to clarify interpretation of rules. Technology: configurable, flexible set of building blocks, networks appropriate for nuclear safety applications, well characterized failure modes and test means to detect these failures, SW development processes. Learning from experience on WWER projects: DUKO 1/2/3/4, MO 3/4, LOVIISA 1/2, Armenia 2, Kozloduy 3/4. Discuss technologies, systems, architectures

Факторы можно разделить на показатели, относящиеся к процессам, технологиям и опыту эксплуатации. Процессы: подход к жизненному циклу обязателен для управления комплектностью, обеспечение качества - основной входной критерий, предсказуемые результаты процессов, важен постоянный и необходимый комплект правил и стандартов, заблаговременные обсуждения между продавцом, заказчиком и регулирующим органом по атомной энергии важны для разъяснения трактовки правил. Технологии: АСУ ТП с перестраиваемой архитектурой, гибкая система блоков зданий, сети, подходящие для обеспечения ядерной безопасности, четко охарактеризованные виды отказов и испытательные средства для обнаружения этих отказов, надежность, обоснованная на основе вывода из имеющихся доказательств, процессы разработки ПО. Из опыта по проектам ВВЭР: Дукованы 1/2/3/4, Моховце 3/4, Ловииза 1/2, Армения 2, Козлодуй 3/4. Обсуждение технологий, систем, архитектур

Base solutions and experience of operation functions of protection on local parameters of WWER-1000 reactors

S21

Mil'to, N.V.; Mil'to, V.A.; Lipin, N.V. (NITs #Left Double Quotation Mark#Kurchatovskij Inst.#Right Double Quotation Mark#, Moscow (RU))

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Osnovnye resheniya i opyt ehkspluatatsii funktsii zashchity po lokal'nym parametram reaktorov VVEhR-1000

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WWER TYPE REACTORS; REACTOR SAFETY; REACTOR PROTECTION SYSTEMS; SYSTEMS ANALYSIS; CALIBRATION; REACTOR CORES; FUEL ELEMENTS; SELF-POWERED NEUTRON DETECTORS

The results of the operation of the protection function by local parameters at various WWER-1000 power units are given. The procedure for calibrating the protection function and improving the calibration procedure based on operating experience are described. To justify the possibility

of implementing the protection function by local parameters a set of test calculations was carried out

Приведены результаты эксплуатации функции защиты по локальным параметрам на различных энергоблоках ВВЭР-1000. Описана процедура калибровки функции защиты и усовершенствование процедуры калибровки на основе опыта эксплуатации. Для обоснования возможности реализации функции защиты по локальным параметрам был проведен набор тестовых расчетов

Equipment qualification NPP in harsh environmental conditions occurring during a design basis accident (DBA), beyond design basis accident (BDBA) in accordance with international standards

S21

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Attestatsiya oborudovaniya v zhestkikh usloviyakh okruzhayushchej sredy pri proektnykh i zaproektnykh avariyaх v sootvetstvii s mezhdunarodnymi standartami

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PWR TYPE REACTORS; ENVIRONMENTAL EFFECTS; REACTOR ACCIDENTS; STANDARDS; CERTIFICATION; QUALITY ASSURANCE; EQUIPMENT; RADIATION HAZARDS; SAFETY

The purpose of Equipment Qualification NPP (EQ), safety-related electrical and mechanical equipment is: reducing the probability of equipment failure common cause in connection with the harsh environmental conditions at the design and beyond design basis accidents; to demonstrate that the safety related electrical and mechanical equipment can perform their specific functions related safety in harsh environments in design and beyond design basis accidents. Presentation defines methodological bases of practical EQ in accordance with international standards used for PWR, for demonstration the performance of its safety functions,

which subjected to abnormal and accident conditions, including loss of ventilation systems, breaks feed waterline, steam and cooling water main system and seismic events

Целью аттестации оборудования, связанного с безопасностью электрического и механического оборудования, является: уменьшение вероятности отказов оборудования по общей причине в связи с жесткими условиями окружающей среды при проектных и запроектных авариях; чтобы продемонстрировать, что связанное с безопасностью электрическое и механическое оборудование способно выполнять свои специальные функции, связанные с безопасностью, в жестких условиях окружающей среды при проектных и запроектных авариях. Презентация определяет методические основы практической аттестации оборудования в соответствии с международными стандартами для PWR, чтобы продемонстрировать выполнение своих функций оборудования, связанного с безопасностью, которое подвергается аномальным и аварийным условиям окружающей среды, включая потери систем вентиляции, разрывы линий питательной воды, паропроводов и основной системы охлаждающей воды и сейсмических событий

Fretting wear of the lower fixation core barrel against RPV

S21

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WWER TYPE REACTORS; FRICTION; FRETTING CORROSION; REACTOR OPERATORS; PULSATIONS; REACTOR VESSELS; REACTOR STABILITY; NONLINEAR PROBLEMS

Pressure pulsations generated by MCP cause relative displacement between core barrel and RPV in axial and radial directions. The friction forces and work of these ones may be calculated using

the models of WWER reactors. The equation for estimating fretting-corrosion wear is presented. Numerical example for WWER 440 Model 213 per 30 and 60 years of operation is presented

Пульсации давления, генерируемые ГЦТ, вызывают относительное смещение между ШВК и корпусом реактора в аксиальном и радиальном направлениях. Силы трения и их влияние могут быть рассчитанными с использованием моделей реакторов ВВЭР. Представлено уравнение для оценки фреттинг-коррозионного износа. Также представлен численный пример для ВВЭР 440 тип 213 за 30 и 60 лет эксплуатации

Comparison of strength calculation results of several standard flanges as per GOST, DIN EN and ANSI as per NPP standards

S21

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Sravnienie rezul'tatov rascheta prochnosti nekotorykh standartnykh flantsev po GOST, DIN EN i ANSI po normam AEhS

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WWER TYPE REACTORS; FLANGES; JOINTS; COMPARATIVE EVALUATIONS; STANDARDS; CONFIGURATION; COMPRESSION STRENGTH

Comparison of strength calculations of standard flanges Dnom40, made in accordance with standards GOST, DIN EN and ANSI. The calculations were performed by Standards for strength of KTA. Calculation results have shown that standard flanges both as per GOST and DIN EN and ANSI can be applied up to the selected tightening torque for the indicated flange joint from the viewpoint of strength

Представлено сравнение прочностных расчетов стандартных фланцев Ду40, выполненных по стандартам ГОСТ, DIN EN и ANSI. Расчеты проводились по Нормам прочности КТА. Результаты расчетов показали, что до определенного выбранного момента затяжки для

указанного фланцевого соединения с точки зрения прочности могут быть применены стандартные фланцы как по ГОСТ, так и по DIN EN или ANSI

Fracture mechanics parameters calculation for semielliptical underclad cracks postulated in the cylindrical part of the WWER pressure vessel

S21

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Raschet parametrov mekhaniki razrusheniya dlya postuliruemykh podnaplavochnykh poluehllipticheskikh treshchin v tsilindricheskoj chasti korpusa reaktora VVEhR

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WWER TYPE REACTORS; REACTOR VESSELS; REACTOR SAFETY; FRACTURE MECHANICS; CRACK PROPAGATION; FRACTURE PROPERTIES; NONLINEAR PROBLEMS; REACTOR STABILITY

The requirements of brittle fracture failure resistance are fulfilled if the cracks postulated in RPV structure remain stable within all project cases. This requirements are stated in terms of fracture mechanics parameters such as J-integral or stress intensity factor. The case of semielliptical underclad cracks postulated in a cylindrical part of WWER pressure vessel is investigated. The analytical solution for fracture mechanics parameters estimation along the crack front in the ferritic vessel was developed on the basis of finite element calculation results. The amplification of the elastic stress intensity factor with respect to effect of plasticity is introduced

Сопротивление хрупкому разрушению считается обеспеченным, если во всех проектных режимах работы реакторной установки не происходит страгивания постулируемых дефектов. Это условие выражается в терминах параметров механики разрушения, таких как J-интеграл и коэффициент интенсивности напряжений. Представлен подход для расчета параметров механики разрушения вдоль фронта поднаплавочных полуэллиптических трещин, постулируемых в цилиндрической части корпуса реактора ВВЭР. Используется база данных результатов многовариантного расчета параметров

механики разрушения методом конечных элементов. Рассмотрены случаи упругого и упругопластического поведения материалов корпуса реактора

Beyond design considerations for bonded and un-bonded tendons in nuclear containments

S21

Varpasuo, P. (Fortum Power and Heat Oy, Espoo (FI))

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AEhS s VVEhR

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International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; REACTOR ACCIDENTS; CONTAINMENT; COMPUTERIZED SIMULATION; CONCRETES; PRESSURE DEPENDENCE; CRACK PROPAGATION; DEFORMATION; REACTOR SAFETY

The simplest method to assess the ultimate pressure capacity of the post-tensioned concrete containment vessel (PCCV) is to use analytical formulas. This approach results in pressure-displacement, which gives the mid-height radial displacement in the cylindrical part of the containment plotted against the internal pressure. The curve exhibits five significant corner points, namely: pressure to overcome pre-stress, concrete cracking, deformation of the lining, plastic deformation of the reinforcing bar, tendon yield (ultimate limit state)

Самым простым методом оценки способности к восприятию предельного давления контейнментом из железобетонной конструкции с натяжением арматуры на бетон является использование аналитических формул. Данный подход приводит к смещению под действием давления, которое в свою очередь вызывает средневысотное радиальное смещение в цилиндрической части контейнмента, графически нанесенного по отношению к внутреннему давлению. На кривой показаны пять значительных угловых точек, а именно: давление для преодоления предварительного напряжения, растрескивание бетона, пластическая деформация облицовки, пластическая деформация арматурного стержня, пластическая деформация арматурного пучка (полное предельное состояние)

Development of innovative high temperature cleanup technology for steam generators

S21

Yurmanov, V.A.; Shut'ko, K.I.; Yurmanov, E.V. (OAO #Left Double Quotation Mark#NIKIEhT#Right Double Quotation Mark#, Moscow (RU)); Polevich, A.N.; Kirilina, A.V. (OAO #Left Double Quotation Mark#VTI#Right Double Quotation Mark#, Moscow (RU))

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Razrabotka innovatsionnoj tekhnologii vysokotemperaturnoj paro-khimicheskoj ochistki teploobmennikh trub parogeneratorov

vp.

(RU)

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; REACTOR SAFETY; PILOT PLANTS; TEMPERATURE RANGE 0400-1000 K; DECONTAMINATION; SURFACE CLEANING; STEAM GENERATORS; HEAT EXCHANGERS

An innovative method for steam-chemical treatment (SCT) of pipe surfaces from operating deposits with dosing into organic acid vapor was developed, which was chosen taking into account its thermolysis under the conditions under consideration and corrosion properties. In order to develop this technology for the SG RU BREST-OD-300 modules, a special experimental stand was created, where several cycles of SCT samples with full-scale sediments at temperatures of 400-450 #Degree Celsius#. The results of the experimental development of the SCT technology at the SG RU BREST-OD-300 parameters with model samples confirmed its effectiveness, economy and environmental safety at low corrosion losses of the metal

Разработан инновационный способ паро-химической очистки (ПХО) поверхностей труб от эксплуатационных отложений с дозированием в пар органической кислоты, которая выбрана с учетом ее термолиза в рассматриваемых условиях и коррозионных свойств. С целью отработки данной технологии для модулей ПГ РУ БРЕСТ-ОД-300 создан специальный экспериментальный стенд, на котором проведено несколько циклов ПХО образцов с натурными отложениями при температурах 400-450 #Degree Celsius#. Результаты экспериментальной отработки технологии ПХО при параметрах ПГ РУ

БРЕСТ-ОД-300 с модельными образцами подтвердили ее эффективность, экономичность и экологическую безопасность при низких коррозионных потерях металла

WWER-440 water chemistry optimization to reduce AOA effect

S21

Kritskij, V.G.; Rodionov, Yu.A.; Berezina, I.G.; Gavrilov, A.V. (AO #Left Double Quotation Mark#ATOMPROEKT#Right Double Quotation Mark#, Sankt-Peterburg (RU))

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Vliyanie vodno-khimicheskogo rezhima reaktora VVEhR-440 na proyavlenie ehffekta AOA vp.

(RU)

10 refs., 6 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; WATER CHEMISTRY; COMPUTERIZED SIMULATION; CORROSION PRODUCTS; THERMAL HYDRAULICS; STEAM GENERATORS; DECONTAMINATION; FUEL ASSEMBLIES; REACTOR SAFETY

The results of the performed research and modeling of the corrosion products mass transfer processes allowed to qualify relative contribution of thermohydraulic and chemical parameters in the processes and to fulfill the activities as follows: to perform power units operation at water chemistry with maximum permissible alkali metals content, to increase the coolant flow rate through the core (to do so, throttling orifices were replaced and canister-shields were removed), to reduce the number of steam generators to be decontaminated to 2 per year in a single power unit. As a result deposits accumulation in fuel assemblies has been minimized and there is no leakage in the fuel element; reactor thermal output limitation has been eliminated

Проведенные исследования и моделирование процессов массопереноса продуктов коррозии позволили уточнить относительный вклад теплогидравлических и химических параметров в эти процессы и выполнить следующий ряд мероприятий: осуществлять эксплуатацию энергоблоков при водно-химическом режиме с максимально допустимым содержанием щелочных металлов, увеличить расход теплоносителя через активную зону (проведена замена дроссельных шайб и удалены кассеты-экраны), количество дезактивируемых парогенераторов снизить до 2-х в год на одном энергоблоке. Достигнута

минимизация накопления отложений в тепловыделяющих сборках и отсутствует разгерметизации ТВЭЛ, исключено ограничения тепловой мощности реакторов

Modeling of corrosion products migration in the secondary circuit of NPP with WWER-1200

S21

Kritskij, V.G.; Berezina, I.G.; Gavrilov, A.V.; Motkova, E.A.; Zelenina, E.V.; Prokhorov, N.A. (AO #Left Double Quotation Mark#ATOMPROEKT#Right Double Quotation Mark#, Sankt-Peterburg (RU))

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Modelirovanie migratsii produktov korrozii vo 2-m konture AEhS s reaktorom VVEhR-1200 vp.

(RU)

12 refs., 6 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomenergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomenergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; COMPUTERIZED SIMULATION; WATER CHEMISTRY; FEEDWATER; SECONDARY COOLANT CIRCUITS; CORROSION; STEAM GENERATORS; MASS TRANSFER

An elaborated model and a verified program for calculate coolant pH_T values in the conditions of the ammonia ethanolamine water chemistry is submitted. Presented is pH_T value distribution through the condensate-feed and steam lines. To determine corrosion products concentration in the steam generator feed water, products mass transfer and their distribution in different working medium streams in the secondary circuit was calculated using the physical and chemical model, in which the secondary circuit is considered as an open-cyclic system consisting of a series of interconnected ones. The obtained results allow to evaluate water chemistry quality and the correctness of design solutions when choosing correcting agent concentrations and optimal pH values for the secondary circuit, as well as structural material suitability, and to define the inter-washing period for steam generators, both at the unit start-up and under normal operation conditions

Представлена разработанная модель и верифицированная программа расчета значений pH_T теплоносителя в условиях аммиачно-этанолминового водно-химического режима

(ВХР). Приведено распределение значений pH_T по конденсатно-питательному и паровому трактам. Для определения концентраций продуктов коррозии в питательной воде парогенераторов был выполнен расчет их массопереноса и распределения по различным потокам рабочей среды второго контура с использованием физико-химической модели, в которой второй контур рассматривается как открытая циклическая система, состоящая из ряда взаимосвязанных. Полученные результаты позволяют оценить качество ВХР и правильность проектных решений по выбору концентраций корректирующих реагентов и оптимальных значений pH во втором контуре, а также пригодность конструкционных материалов, и определить межпромывочный период парогенераторов, как при пуске блока, так и при нормальных условиях эксплуатации

Thermal-vacuum drying unit for drying the spent radioactive ion-exchange resins

S12

Prokhorov, N.A.; Babkin, V.V.; Byl'ev, S.Yu.; Khoroshilov, L.I.; Malashkin, S.Yu.; Atrushkevich, A.E.; Sorokin, V.T.; Demin, A.V.; Iroshnikov, V.V. (AO #Left Double Quotation Mark#ATOMPROEKT#Right Double Quotation Mark#, Sankt-Peterburg (RU))

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Ustanovka termovakuumnoj sushki otrabotavshikh radioaktivnykh ionoobmennyykh smol
vp.

(RU)

6 refs., 3 figs., 1 tab.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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RESINS; ION EXCHANGE MATERIALS; NUCLEAR POWER PLANTS; RADIOACTIVE WASTE PROCESSING; DRYING; PILOT PLANTS; WASTE FORMS; SAFETY

The tests description and tests results for the pilot demonstration unit, which satisfies the requirements imposed on the commercial unit in terms of resin volume treated at a time are presented. As a result of the tests, the main parameters of the ion-exchange resin drying process were determined; these parameters ensure producing a product of the required quality in terms of solid radioactive waste criteria. The calculations results for the resin condition during long-term storage and for radiation dose rate emitted by the dried resin containers to confirm storage safety are given

Представлено описание и результаты испытаний опытно-демонстрационной установки, которая по объему единовременно обрабатываемой смолы и производительности соответствует требованиям, предъявляемым к промышленной установке. В результате испытаний определены основные параметры процесса сушки ионообменной смолы, обеспечивающие получение продукта требуемого качества по критериям отнесения к твердым радиоактивным отходам. Приведены результаты расчетов по состоянию смолы в течение длительного хранения и расчетов мощности дозы излучения от контейнеров с осушенной смолой, подтверждающие безопасность хранения

On calculation of the rate of erosion-corrosion wear

S36

Baranenko, V.I.; Naftal', M.M.; Semenova, K.Yu.; Andreenkova, A.V. (AO #Left Double Quotation Mark#VNIAEhS#Right Double Quotation Mark#, Moscow (RU))

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O raschete skorosti ehrozionno-korrozionnogo iznosa

vp.

(RU)

24 refs., 10 figs., 7 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WEAR; EROSION; CORROSION; KINETICS; EROSION CONTROL; TIME DEPENDENCE; CORROSION PRODUCTS; THICKNESS

Three values are used to calculate the thickness change rate: the initial value the thickness of the wall of the pipeline element, the final thickness at the date of inspection and the interval of time between the dates of the determination of thickness. There are two ways to select the interval of time: in the case of one operational control, the time interval is determined between the date of commissioning of the pipeline element and the date operational control, in the case where several operational controls the time interval can be determined between the dates neighboring controls. Various issues related to the definition of erosion-corrosion wear speed, the speed of formation of deposits on the basis of operational controls and indicators based on the use of corrosion with different exposure times are examined

Для расчета скорости изменения толщины используются три величины: начальная толщина стенки элемента трубопровода, конечная толщина на дату контроля и интервал времени между датами определения толщин. Используется два способа выбора интервала времени: в случае проведения одного эксплуатационного контроля интервал времени определяется между датой ввода элемента трубопровода в эксплуатацию и датой проведения эксплуатационного контроля, в случае, когда проведено несколько эксплуатационных контролей, интервал времени может быть определен между датами соседних контролей. Рассматриваются различные вопросы, связанные с определением скорости эрозионно-коррозионного износа, скорости формирования отложений на основе данных эксплуатационного контроля и на основе использования индикаторов коррозии с различным временем экспозиции

Chemical composition effect on WWER-1000 RPV weld metal thermal aging S21

Chernobaeva, A.A.; Kuleshova, E.A.; Gurovich, B.A.; Erak, A.D.; Zabusov, O.O.; Mal'tsev, D.A.; Zhurko, D.A.; Papina, V.B.; Skundin, M.A. (NITs #Left Double Quotation Mark#Kurchatovskij Inst.#Right Double Quotation Mark#, Moscow (RU))

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Vliyanie khimicheskogo sostava na temperaturnoe starenie materialov korpusov VVEhR-1000 vp.

(RU)

6 refs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; AGING; REACTOR VESSELS; TEMPERATURE DEPENDENCE; REACTOR OPERATION; GRAIN BOUNDARIES; FRACTOGRAPHY

New results of WWER-1000 reactor pressure vessels materials thermal aging surveillance specimens are obtained. The maximum holding time reaches 208896 hours. The obtained information did not confirm the mechanism of carbide hardening proposed earlier for the temperature aging. A revision of the weld metal surveillance specimen data base has been performed recently. It is supplemented by the results of fractographic studies and studies of segregations at grain boundaries. This allowed to develop models of temperature aging of reactor vessel materials for long exposures at the operating temperature

Получены новые результаты по температурному старению материалов корпусов реакторов ВВЭР-1000. Максимальное время выдержки при температуре эксплуатации 208896 часов. Полученная информация не подтвердила ранее предложенный для температурного старения механизм карбидного упрочнения. Выполнена ревизия базы данных температурных образцов-свидетелей. Она дополнена результатами фрактографических исследований и исследований сегрегаций на границах зерен. Это позволило разработать модели температурного старения материалов корпусов реакторов для длительных выдержек при температуре эксплуатации

Influence of annealing temperature on the degree of structural state and properties recovery of reactor pressure vessel internals

S21

Gurovich, B.A.; Kuleshova, E.A.; Frolov, A.S.; Mal'tsev, D.A.; Prikhod'ko, K.E.; Fedotova, S.V. (NITs #Left Double Quotation Mark#Kurchatovskij Inst.#Right Double Quotation Mark#, Moscow (RU))

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AEhS s VVEhR

Vliyanie temperatury otzhiga na stepen' vosstanovleniya strukturnogo sostoyaniya i svojstv materiala VKU

vp.

(RU)

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomenergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomenergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; REACTOR COMPONENTS; CHROMIUM-NICKEL STEELS; IRRADIATION; ANNEALING; TEMPERATURE DEPENDENCE; TEMPERATURE RANGE 0400-1000 K; PHYSICAL RADIATION EFFECTS; SWELLING

To assess the degree and mechanisms of stainless steels damage under neutron irradiation structural studies of 12Cr18Ni10Ti and 08Cr18Ni10Ti steel specimens after irradiation in BOR-60 at temperature range 330-400#Degree Celsius# and damaging doses of 30-145 dpa and 08Cr18Ni10Ti (neutron measurement channel of WWER-1000) after irradiation in WWER-1000 conditions (320#Degree Celsius#, 12-14 dpa) were carried out. The possibility of the return of the structural-phase state and properties by means of recovery annealing for 12Cr18Ni10Ti steel (with the swelling level 7% after irradiation in BOR-60 to a damaging dose of 115 dpa) was also studied. It was experimentally shown that recovery annealing in the temperature range of 700-

900#Degree Celsius# promotes the disappearance of radiation defects, secondary phases (G-phase and #alpha#-phase) and radiation-induced segregation. Increasing of the annealing temperature up to 800-1100#Degree Celsius# leads to an abrupt decrease of the primary porosity with the increase of the pore size (more than 1 mm) and a secondary porosity arises with small probably gas-filled pores (2-6 nm). The principal possibility recovery annealing that ensures almost complete recovery of the structure and properties of internal materials. The optimal mode of recovery annealing was chosen: 1000#Degree Celsius# during 120 hours

Для оценки степени и механизмов повреждаемости аустенитных сталей в процессе нейтронного облучения в работе были проведены структурные исследования образцов сталей 12Cr18Ni10Ti и 08Cr18Ni10Ti, облученных в реакторе БОР-60 в диапазоне температур 330-400 #Degree Celsius# и повреждающих доз 30-145 сна, и образцов стали 08Cr18Ni10Ti (чехлы КНИ реактора ВВЭР-1000), облученных в условиях ВВЭР-1000 (320#Degree Celsius#, 12-14сна). Также была исследована возможность возврата структурно-фазового состояния и свойств стали 12Cr18Ni10Ti (с уровнем распухания 7% после облучения в реакторе БОР-60 до повреждающей дозы 115 сна) за счет проведения восстановительного отжига. Экспериментально показано, что проведение изотермических отжигов в температурном интервале 700-900#Degree Celsius# способствует исчезновению радиационных дефектов, вторичных фаз (G-фаза, #alpha#-фаза) и радиационно-индуцированных сегрегаций. При повышении температуры отжига до температур 800-1100#Degree Celsius# происходит резкое снижение первичной пористости при увеличении ее размеров (до более 1 мкм) и возникает мелкая (2-6 нм) вторичная пористость (вероятно, газонаполненная). Показана принципиальная возможность проведения восстановительного отжига ВКУ, обеспечивающего практически полный возврат механических свойств и микроструктуры. Выбран наиболее оптимальный режим восстановительного отжига: 1000#Degree Celsius# в течение 120 ч, обеспечивающий практически полный возврат физико-механических характеристик материала

Prediction of radiation embrittlement for WWER-1000 pressure vessel materials on service lifetime extension

S21

Gurovich, B.A.; Erak, D.Yu.; Zhurko, D.A.; Kuleshova, E.A.; Chernobaeva, A.A.; Papina, V.B.; Zabusov, O.O.; Khodan, A.N.; Skundin, M.A. (NITs #Left Double Quotation Mark#Kurchatovskij Inst.#Right Double Quotation Mark#, Moscow (RU))

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Prognozirovanie radiatsionnogo okhrupchivaniya materialov korpusov rektorov VVEhR-1000 pri prodlenii sroka sluzhby

vp.

(RU)

19 refs., 2 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomenergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomenergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; PHYSICAL RADIATION EFFECTS; EMBRITTLEMENT;
REACTOR OPERATION; SERVICE LIFE; SAMPLING; REACTOR VESSELS; NEUTRON
FLUENCE

A forecast of radiation embrittlement of WWER-1000 RPV materials after the end of design lifetime is terminated; the forecast is built on the database of SS tests and takes into account the findings of investigation of changes of the RPV material fine structure under irradiation is offered. In contrast to the earlier developed dependences the forecast of the ductile-to-brittle transition temperature changes of the WWER-1000 RPV metal under irradiation elaborated in the frame of this work relies on the physically based concept of the total shift of the ductile-to-brittle transition temperature being a sum of hardening and non-hardening components. The comparison with the newly obtained results of the SS metal accelerated irradiation gives grounds for the applicability of the obtained dependences on the extended range of fast neutron fluence values up to 8×10^{23} neutron/m²

На основе анализа базы данных испытания ОС и с учетом результатов исследования изменения тонкой структуры материалов КР под облучением выполнен прогноз радиационного охрупчивания материалов КР ВВЭР-1000 при эксплуатации за пределами проектного срока службы. В отличие от ранее разработанных зависимостей, для прогноза изменения критической температуры хрупкости металла корпуса реактора ВВЭР-1000 под облучением принято физически обоснованное представление общего сдвига критической температуры хрупкости в виде суммы упрочняющей и неупрочняющей компонент. Сопоставлением с вновь полученными результатами опережающего облучения металла ОС обоснована возможность использования полученных зависимостей до значений флюенса быстрых нейтронов $\#Swung\ Dash\# 8 \times 10^{23}$ нейтрон/м²

Mini Charpy specimens for WWER-1000 materials study

S21

Chernobaeva, A.A.; Medvedev, K.I.; Zhurko, D.A.; Kostromin, V.N.; Skundin, M.A.; Erak, D.Yu.; Mikhin, O.V. (NITs #Left Double Quotation Mark#Kurchatovskij Inst.#Right Double Quotation Mark#, Moscow (RU))

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Ispol'zovanie malorazmernykh obraztsov tipa Sharpi dlya issledovaniya materialov korpusov VVEhR-1000

vp.

(RU)

5 refs., 14 figs., 1 tab.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; CHARPY TEST; REACTOR VESSELS; SERVICE LIFE; CHROMIUM-NICKEL-MOLYBDENUM STEELS; JOINTS; BENDING; CORRELATIONS; BRITTLENESS

At present, in connection with the performance of works to justify the extension of the service life of WWER-1000 cases, it became necessary to develop a correlation between the results of small-sized and standard samples for WWER-1000 vessels materials, that is, for steel 15Cr2NiMoVN and its welded joints. A procedure for developing a database for estimating the correlation between the results of tests of standard and small-size Charpy samples aimed at reducing scattering of experimental results is proposed. This made it possible to reduce the scattering of experimental data by a factor of two in comparison with the results of a similar work for WWER-440 materials. The data base, obtained in this work used for development of Russian Standard #Left-Pointing Double Angle Quotation Mark#Procedure of determination of transition temperature of RPV materials using small-size Charpy specimens for impact bend testing#Right-Pointing Double Angle Quotation Mark# MT1..2.1.15.002.0983-2014

В настоящий момент в связи с выполнением работ по обоснованию продления срока службы корпусов ВВЭР-1000 возникла необходимость в разработке соотношения между результатами малоразмерных и стандартных образцов для материалов корпусов ВВЭР-1000, то есть для стали 15Cr2NiMoVN и ее сварных соединений. Предложена процедура разработки базы данных для оценки корреляции между результатами испытаний стандартных и малоразмерных образцов Шарпи, направленная на снижение рассеяния экспериментальных результатов. Это позволило снизить рассеяние экспериментальных данных в два раза по сравнению с результатами аналогичной работы для материалов ВВЭР-440. Полученная база данных позволила разработать #Left-Pointing Double Angle Quotation Mark#Методику определения критической температуры хрупкости материалов корпусов реакторов по результатам испытаний малоразмерных образцов на ударный изгиб#Right-Pointing Double Angle Quotation Mark# MT1.2.1.15.002.0983-2014

Concerning feasibility of after power control in the course of non-design basis accident

S21

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O vozmozhnostyakh kontrolya ostatochnogo ehnergovydeleniya pri zaproektnykh avariyyakh vp.

(RU)

5 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; REACTOR ACCIDENTS; REACTOR CONTROL SYSTEMS; RADIATION DETECTORS; DATA ANALYSIS; NUCLEAR POWER PLANTS; RECOMBINATION

At various NPPs with WWER reactors, studies of arrays of archival data on the currents of communication lines, including signals of integral background SPD, were carried out at different times. The results of the research are presented and discussed. Recommendations are given for prospective detectors and effective placement points

Проведены в разное время на разных АЭС с реакторами ВВЭР исследования массивов архивных данных по токам линий связи, в том числе сигналов интегральных фоновых ДПЗ. Приводятся и обсуждаются результаты исследований. Даны рекомендации по перспективным детекторам и эффективным точкам размещения

Molecular dynamic study of the U-Mo alloy structure

S36

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Issledovanie anizotropii splava U-Mo metodom atomisticheskogo modelirovaniya

vp.

(RU)

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ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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URANIUM ALLOYS; MOLYBDENUM ALLOYS; MOLECULAR DYNAMICS METHOD;
CRYSTALLIZATION; LATTICE PARAMETERS; ANISOTROPY; TRANSITION
TEMPERATURE; ISOTROPY; CRYSTAL STRUCTURE; ENTHALPY

The method of atomistic simulations was used to study the structure of the quasi-equilibrium tetragonal phase U-Mo, formed during crystallization of the melt. Radial distribution function and bond-angle distribution function were calculated for the analysis of uranium and molybdenum structures in the alloy. The lattice constants of the uranium-molybdenum alloy were obtained for different concentrations of molybdenum. Results of calculations are in good agreement with experimental data and confirm the anisotropy of the lattice at low molybdenum concentrations. Temperature of the transition from anisotropic tetragonal phase to a body-centered cubic phase was calculated for different concentrations of molybdenum. It was found that the anisotropy is a consequence of the local arrangement of uranium atoms in the U-Mo alloy structure. It is shown that the anisotropy disappears with increase of molybdenum concentration not due to changes in the uranium atoms arrangement. It disappears because the number of molybdenum atoms #En Dash# #Left Double Quotation Mark#stabilization centers of isotropy#Right Double Quotation Mark# increases. Also dependence of the enthalpy of mixing for uranium-molybdenum alloy on molybdenum concentration was calculated. It is shown that anomalous enthalpy of mixing - molybdenum concentration dependence, known from the experiments, can be obtained only when the atomic structure of the alloy is taking into account.

Выполнено исследование структуры квазиравновесной тетрагональной фазы U-Mo, образованной при кристаллизации расплава. Для анализа подрешеток урана и молибдена в сплаве рассчитывались парно-корреляционная функция и угловая функция распределения. Были получены значения параметров решетки сплава уран-молибден для различных концентраций молибдена. Результаты расчетов хорошо согласуются с экспериментальными данными и подтверждают анизотропию решетки при малых концентрациях молибдена. Рассчитана температура перехода анизотропной тетрагональной фазы в объемно-центрированную кубическую фазу при различных концентрациях молибдена. Обнаружено, что анизотропия является следствием локального расположения атомов урана в структуре сплава U-Mo. Показано, что при увеличении концентрации молибдена анизотропия исчезает не за счет изменения положений атомов урана, а за счет накопления #Left Double Quotation Mark#центров стабилизации изотропии#Right Double Quotation Mark# - атомов молибдена. Рассчитана энтальпия смешения сплава уран-молибден в зависимости от концентрации молибдена. Показано, что только при учете атомистической структуры сплава можно воспроизвести аномальную зависимость энтальпии смешения от концентрации, известную из экспериментов

Main mechanisms of radiation damage of WWER PVI materials and material science issues of PVI longtime operation

S21

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Osnovnye mekhanizmy radiatsionnogo povrezhdeniya materialov VKU i materialovedcheskie problemy ikh dlitel'noj ehkspluatatsii

vp.

(RU)

12 refs., 11 figs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; REACTOR VESSELS; SWELLING; EMBRITTLEMENT; CORROSION; SERVICE LIFE; REACTOR SAFETY; RECOMMENDATIONS

Main mechanisms of radiation damage of materials for WWER pressure vessel internals (PVI) are considered and limit conditions for PVI are formulated in the presented work. It is shown that the following factors limiting PVI lifetime are as follows: radiation swelling leading to change of PVI geometrical sizes and embrittlement of PVI material, and irradiation assisted stress corrosion cracking. Lifetime (operation time) of core baffle for project WWER-TOI is 60 years for realistic (median) estimation of radiation swelling, but only 40 years for conservative estimation. The basic requirements to material (chemical composition, microstructure and mechanical properties) are formulated. These requirements provide internals serviceability up to 60 years and longer for conservative estimation of the radiation swelling. The roadmaps are proposed for developing the new steel

Рассмотрены основные механизмы повреждений материалов внутрикорпусных устройств реакторов ВВЭР, а также сформулированы предельные состояния, определяющие ресурс элементов ВКУ. Показано, что основными факторами, определяющими ресурс ВКУ, являются радиационное распухание, приводящее к формоизменению выгородки и резкому охрупчиванию ее материала, и коррозионное растрескивание высокооблученного материала выгородки. При консервативной оценке радиационного распухания ресурс выгородки ВВЭР ТОИ составляет 40 лет; при более реалистичной - медианной оценке - 60

лет. Сформулированы требования к материалу (по химическому составу, микроструктуре и механическим свойствам), обеспечивающие работоспособность ВКУ не менее 60 лет при консервативной оценке распухания, а также предложены пути реализации разработки новой стали

The guaranteed level of mechanical properties of large-size forgings of steel type 15Kh2MFA and role of the thermal and quenching equipment to obtain it S21

Karzov, G.P.; Teplukhina, I.V.; Tsvetkov, A.S.; Shamraj, E.L. (FGUP #Left Double Quotation Mark#TsNII KM #Left Double Quotation Mark#Prometej#Right Double Quotation Mark#, Sankt-Peterburg (RU))

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Garantirovannyj uroven' mekhanicheskikh svojstv krupnogabaritnykh pokovok iz stali tipa 15Kh2MFA i rol' termicheskogo i zakalochnogo oborudovaniya v ego obespechenii

vp.

(RU)

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; CHROMIUM-NICKEL-MOLYBDENUM STEELS; REACTOR VESSELS; MECHANICAL PROPERTIES; SERVICE LIFE; HEAT TREATMENTS

Safety of the RPV during the operation time provided not only improved mechanical and special properties (increased resistance to irradiation and thermal embrittlement) of the RPV-material, the low rate of its degradation under irradiation and the operating temperature, but the metallurgical quality of forgings, homogeneity and isotropy of the properties throughout the large-sized forgings body. the guaranteed level of mechanical properties in the large-size forgings with heat treatment wall thickness from 200 to 420 mm, made from steel grade 15Kh2MFA during the period from 2008 to 2014 years at #Left-Pointing Double Angle Quotation Mark#OMZ-Special Steels#Right-Pointing Double Angle Quotation Mark# (Saint-Petersburg, Russia), #Left-Pointing Double Angle Quotation Mark#Energomashpetsstal#Right-Pointing Double Angle Quotation Mark# (Kramatorsk, Ukraine) and #Left-Pointing Double Angle Quotation Mark#Volgograd Metallurgical Plant #Left-Pointing Double Angle Quotation Mark#Red October#Right-Pointing Double Angle Quotation Mark# (Volgograd, Russia). Special attention paid to the analysis of factors affecting the obtaining of homogeneous

mechanical properties and structure throughout the large-sized forgings body during heat treatment

Безопасность КР в течение всего срока его эксплуатации обеспечивается не только повышенными механическими и служебными свойствами материала КР, низким темпом их деградации под воздействием облучения и рабочей температуры, но металлургическим качеством заготовок, однородностью и изотропностью свойств по объему заготовок. Произведена оценка гарантированного уровня механических характеристик поковок из стали типа 15X2МФА толщиной под термическую обработку от 200 до 420 мм, произведенных в период с 2008 по 2014 гг. на ООО #Left Double Quotation Mark#ОМЗ-Спецсталь#Right Double Quotation Mark#, ПАО #Left Double Quotation Mark#Энергомашспецсталь#Right Double Quotation Mark# и ЗАО #Left Double Quotation Mark#ВМЗ #Left Double Quotation Mark#Красный Октябрь#Right Double Quotation Mark#. Особое внимание уделено анализу факторов, влияющих на получение однородных механических свойств и структуры по всему сечению в крупногабаритных поковках из реакторных сталей при термической обработке

Stability of the mechanical properties of Cr-Mo-V steels for WWER-type reactor pressure vessels Generation III+ under operating temperature and its structural aspects

S21

Karzov, G.P.; Teplukhina, I.V.; Zotova, A.O.; Zajtseva, O.Yu. (FGUP #Left Double Quotation Mark#TsNII KM #Left Double Quotation Mark#Prometej#Right Double Quotation Mark#, Sankt-Peterburg (RU))

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Stabil'nost' mekhanicheskikh svojstv stalej Cr-Mo-V kompozitsii dlya korpusov reaktorov tipa VVEhR pokoleniya III+ pod vozdejstviem rabochikh temperatur i ee strukturnye aspekty vp.

(RU)

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; CHROMIUM ALLOYS; MOLYBDENUM ALLOYS;
VANADIUM ALLOYS; CHROMIUM-NICKEL-MOLYBDENUM STEELS;
TEMPERATURE RANGE 0400-1000 K; BRITTLENESS; REACTOR VESSELS

A comparative analysis of changes in the fine structure under the thermal ageing is performed for a currently used steels and new steels. When 15Kh2MFA-A mod. A and 15Kh2MFA-A mod. B steels were investigated with electron microscopic it was shown that thermal aging at 350#Degree Celsius# during 500 and 3000 hours do not lead to increasing of quantity and volume distribution density of disperse carbides of vanadium and niobium hardening a steel after high-temperature tempering. Also an increasing of size does not observed for the carbides particles of Me₇C₃- type located along the grain boundaries after the thermal aging. Demonstrated absence of a hardening mechanism of embrittlement under thermal aging RPV-steels with limited nickel

Проведен сравнительный анализ изменений тонкой структуры применяемой и новых сталей под влиянием тепловых выдержек. Электронно-микроскопические исследования тонкой структуры стали марок 15X2MФА-А мод. А и 15X2MФА-А мод. Б показали, что после теплового старения при температуре 350#Degree Celsius# в течение 500 и 3000 часов не наблюдается увеличения количества дисперсных карбидов ванадия и ниобия, упрочняющих сталь после высокого отпуска, а также плотности их объемного распределения. Также не наблюдается увеличения размеров частиц карбидов типа М₇С₃, располагающихся по границам зерен, после тепловых выдержек. Показано отсутствие упрочняющего механизма охрупчивания при тепловом старении корпусных сталей с ограниченным содержанием никеля

Technology development and production of elongated shell for reactor vessel active zone of WWER-TOI project from steel 15Cr2NiMoVN class 1

S21

Shklyayev, S.Eh.; Titova, T.I.; Ratushev, D.V.; Shul'gan, N.A. (NITs OOO #Left Double Quotation Mark#TK #Left Double Quotation Mark#OMZ-Izhora#Right Double Quotation Mark#, Sankt-Peterburg (RU)); Eroshkin, S.B.; Duryinin, V.A.; Efimov, S.V. (OOO #Left Double Quotation Mark#OMZ-Spetsstal'#Right Double Quotation Mark#, Sankt-Peterburg (RU)); Dub, V.S.; Kulikov, A.P.; Romashkin, A.N. (GNTs NPO #Left Double Quotation Mark#TsNIITMASH#Right Double Quotation Mark#, Moscow (RU))

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Razrabotka tekhnologii i proizvodstvo udlinennoj obechajki aktivnoj zony korpusa reaktora proekta VVEhR-TOI iz stali 15Kh2NMFA kl.1

vp.

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4 refs., 8 figs., 2 tabs.

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WWER TYPE REACTORS; REACTOR CORES; REACTOR VESSELS; REACTOR TECHNOLOGY; REACTOR SAFETY; CHROMIUM-NICKEL-MOLYBDENUM STEELS; CONTAINMENT SHELLS

Production process for the elongated shell blank of the active zone of the reactor pressure vessel made from steel 15Cr2NiMoVN Class 1 with finished sizes Dext=4.655 mm, Dint=4.240 mm, H=4.910 mm (height for heat treatment #En Dash# 5.750 mm) is presented. For the first time in Russia in production site of OMZ-Special steel LLC a unique elongated shell blank of the reactor vessel active zone was made from ingot 420.0 t for WWER-TOI project fully meeting the specified requirements in terms of metallurgical quality and set of service properties

Представлен обзор технологии производства заготовки удлиненной ОАЗ КР из стали 15Х2НМФА кл.1 с чистовыми размерами Dнар.=4.655 мм, Dвн.=4.240 мм, H=4.910 мм (высота под термическую обработку 5.750 мм). Впервые в России в производственных условиях ООО #Left Double Quotation Mark#OMZ-Spetsstal'#Right Double Quotation Mark# из слитка 420.0 т изготовлена уникальная заготовка удлиненной ОАЗ КР для проекта ВВЭР-ТОИ, полностью соответствующая заданным требованиям по уровню металлургического качества и комплексу служебных свойств

Study of elongated shell metal quality of reactor vessel active zone for WWER-TOI project from steel 15Cr2NiMoVN class 1

S21

Titova, T.I.; Ratushev, D.V.; Shul'gan, N.A. (NITs ООО #Left Double Quotation Mark#TK #Left Double Quotation Mark#OMZ-Izhora#Right Double Quotation Mark#, Sankt-Peterburg (RU)) (and others)

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Issledovanie kachestva metalla udlinennoj obechajki aktivnoj zony korpusa reaktora proekta VVEhR-TOI iz stali 15Kh2NMFA kl.1

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11 refs., 9 figs., 4 tabs.

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WWER TYPE REACTORS; REACTOR VESSELS; QUALITY CONTROL; MATERIALS TESTING; RELIABILITY; CHROMIUM-NICKEL-MOLYBDENUM STEELS; MECHANICAL PROPERTIES; CONTAINMENT SHELLS; REACTOR CORES; FRACTOGRAPHY

For overall assessment of metal quality for shell made of a supersized ingot (420 t), upon the acceptance tests, additional tests and studies were conducted for the specified shell blank as follows: study of metallurgical quality of the blank metal, including determination of chemical composition and content of harmful impurities and gases, assessment of chemical segregation, structure and microstructure analysis, assessment of contamination with non-metallic inclusions, determination of hardness level directly on the elongated large sized shell surface, study of mechanical homogeneity, including T_{ko} and T_k along the height and thickness of the elongated shell blank, study of mechanical anisotropy, including T_{ko} , depending on the direction of specimens in the elongated shell blank, fractographic study of specimen fractures after mechanical tests. It is shown high metallurgical quality and uniformity of chemical composition, structure and service properties of metal

Для всесторонней оценки качества металла обечайки, изготовленной из сверхкрупного слитка 420 тонн, были проведены дополнительные испытания и исследования указанной заготовки в следующем объеме: исследование металлургического качества металла заготовки, включая определение химического состава, содержания вредных примесей и газов, оценку степени химической ликвации, анализ макро- и микроструктуры, оценку загрязненности неметаллическими включениями, определение уровня твердости непосредственно на поверхности удлиненной крупногабаритной обечайки, исследование однородности механических свойств, включая T_{ko} и T_k , по высоте и толщине заготовки удлиненной обечайки, исследование анизотропии механических свойств, включая T_{ko} , в зависимости от ориентации образцов в заготовке удлиненной обечайки, фрактографические исследования изломов образцов после механических испытаний. Показано высокое металлургическое качество и однородность как химического состава, так и структуры и служебных свойств металла

Application of uncertainty analysis method for calculations of accident conditions for RP AES-2006

S21;S97

Zajtsev, S.I.; Bykov, M.A.; Zakutaev, M.O.; Siryapin, V.N.; Petkevich, I.G.; Siryapin, N.V.; Borisov, S.L.; Kozlachkov, A.N. (OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark#, Podol'sk (RU))

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Primenenie metoda analiza neopredelennostej k raschetam avarijnykh rezhimov dlya RU AEhS-

2006

vp.

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International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; RISK ASSESSMENT; SAFETY ANALYSIS; COMPUTER CODES; REACTOR SAFETY; PROBABILISTIC ESTIMATION

An analysis of some accidents using the uncertainly assessment methods is given. The list of the variable parameters incorporated the model parameters of the computer codes, initial and boundary conditions of reactor plant, neutronics. On the basis of the performed calculations of the accident conditions using the statistical method, errors assessment is presented in the determination of the main parameters comparable with the acceptance criteria. It was shown that in the investigated accidents the values of the calculated parameters with account for their error obtained from TRAP-KS and KORSAR/GP Codes do not exceed the established acceptance criteria. Besides, these values do not exceed the values obtained in the conservative calculations. A possibility in principle of the actual application of the method of estimation of uncertainty was shown to justify the safety of WWER AES-2006 using the thermal-physical codes KORSAR/GP and TRAP-KS, PANDA and SUSA programs

Представлен анализ некоторых аварийных режимов с применением метода анализа неопределенностей. В перечень варьируемых параметров были включены модельные параметры расчетных кодов, начальные и граничные параметры РУ, нейтронно-физические параметры. На основе выполненных расчетов аварийных режимов с применением статистического метода, представлена оценка погрешности определения основных параметров, сравниваемых с приемочными критериями. Показано, что в исследуемых авариях значения расчетных параметров с учетом их погрешности, полученные по программам ТРАП-КС и КОРСАР/ГП, не превышают установленных приемочных критериев. Кроме того, эти значения не превышают значений, полученных в консервативных расчетах. Показана принципиальная возможность практического применения метода оценки неопределенности для обоснования безопасности проектов ВВЭР АЭС-2006 с применением теплофизических кодов КОРСАР/ГП и ТРАП-КС, программ ПАНДА и SUSA

Comparative evaluation of corrosion-mechanical properties and long-term strength of EP 302M

S21

Novichkova, O.V.; Pisarevskij, L.A.; Filippov, G.A. (FGUP #Left Double Quotation

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Sravnitel'naya otsenka korrozionno-mekhanicheskikh svoystv i dlitel'noj prochnosti stali marki EhP 302M

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CHROMIUM-NICKEL-MOLYBDENUM STEELS; CORROSION RESISTANCE; CORROSION RESISTANT ALLOYS; AUSTENITIC STEELS; FAST REACTORS; LIQUID METAL COOLED REACTORS; COMPARATIVE EVALUATIONS; STEAM GENERATORS; REACTOR COOLING SYSTEMS

The differential peculiarities of mechanical properties and corrosion behavior of modified austenitic steel EP 302M in comparison with the austenitic and martensitic steel grades were considered. The steel EP 302M exhibits high resistance to local corrosion types both in the steam-water environments of high parameters (up to 505#Degree Celsius#, 17 MPa) and operational modes of steam-generators. At the accelerated standard testing the steel corrosion characteristics surpass the similar parameters of conventional corrosion-resistant steels and nickel-based alloys. Steel has good processing and welding characteristics. The industrial batch of heat-exchange tubes which can simultaneously be operated in the liquid lead and steam-water medium has been produced at the first time both in the domestic and foreign practice. Tubes are designed to the model of steam-generators for BREST-OD-300 reactor system

Рассмотрены отличительные особенности механических свойств и коррозионное поведение модифицированной аустенитной стали марки ЭП 302М в сравнении со сталями аустенитного и мартенситного классов. Сталь ЭП 302М при испытаниях в паро-водяных средах высоких параметров (до 505#Degree Celsius#, 17 МПа), при режимах эксплуатации парогенераторов, демонстрирует высокую стойкость против локальных видов коррозии. Ее коррозионные характеристики при ускоренных стандартных испытаниях превосходят аналогичные показатели традиционных коррозионно-стойких сталей и сплавов на основе никеля. Сталь обладает хорошими технологическими характеристиками и свариваемостью. Впервые в отечественной и зарубежной практике изготовлена

промышленная партия теплообменных труб, которые могут одновременно эксплуатироваться в жидком свинце и пароводяной среде. Трубы предназначены для модели парогенераторов реакторной установки БРЕСТ-ОД-300

Elaboration of measures on prevention of metal corrosion in the area of collector welding to vessel of steam generator PGV-1000

S21

Zhukov, R.Yu.; Brykov, S.I.; Kharchenko, S.A.; Zubchenko, A.S. (AO OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark#, Podol'sk (RU))

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Razrabotka mer po predotvrashcheniyu korrozii metalla v zone shva privarki kollektora k korpusu parogeneratora PGV-1000

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WWER TYPE REACTORS; CORROSION PROTECTION; STEAM GENERATORS; HEAT RESISTING ALLOYS; REACTOR SAFETY; WELDED JOINTS; CORROSION

The statements are presented on the main corrosion mechanisms in the area of welded joint No. 111 for the steam generators of NPP with WWER-1000. On the basis of the results of the analysis of the mechanisms the process measures are proposed for suppression of metal corrosion in collector #Left Double Quotation Mark#pockets#Right Double Quotation Mark#. Implementation of the procedure of filling the cavities of collector #Left Double Quotation Mark#pockets#Right Double Quotation Mark# with alkaline-reducing solution for steam generators of the operating NPP Units with WWER-1000 before commissioning will allow to exclude the effect of corrosion factor for manifestation of retarding strain corrosion cracking of steel 10MnNi2MoVN

Изложены положения об основных механизмах коррозии в зоне с.с. Numero Sign111 парогенераторов АЭС с ВВЭР-1000. Основываясь на результатах анализа механизмов, предложены технологические меры для подавления коррозии металла #Left Double Quotation Mark#карманов#Right Double Quotation Mark# коллекторов. Внедрение

процедуры заполнения перед пуском полостей #Left Double Quotation Mark#карманов#Right Double Quotation Mark# коллекторов парогенераторов действующих блоков АЭС с ВВЭР-1000 щелочно-восстановительным раствором позволит исключить влияние коррозионного фактора для проявления ЗДКР стали 10MnNi2MoVN

Assessment of efficiency of the measures aimed at prevention of damages of welds #Numero Sign#111 at steam generators of NPP with WWER-1000

S21

Kharchenko, S.A.; Zhukov, R.Yu.; Kurdin, M.E.; Sotskov, V.V. (AO OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark#, Podol'sk (RU))

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International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; REACTOR SAFETY; DAMAGE; WELDED JOINTS; NUCLEAR POWER PLANTS; STEAM GENERATORS

On the basis of the earlier performed studies and activities on revealing of causes of metal damage of collectors in the area of welded joint #Numero Sign#111 it is demonstrated that the damage mechanism is found out. The main factors resulting in damages are formulated. The safety concept justifying impossibility of large-scale damage is developed. The measures are described for improving the operational reliability of the unit at NPP

На основании комплекса проведенных ранее исследований и выполненных работ по выяснению причин повреждения металла коллекторов в зоне с.с. #Numero Sign#111 показано, что механизм повреждения установлен. Сформулированы основные факторы, приводящие к повреждениям. Разработана концепция безопасности, обосновывающая невозможность крупномасштабного разрушения. Указаны меры для повышения надежности эксплуатации узла на АЭС

Applying of small punch technique for evaluation of mechanical properties of primary circuit components after long-term operation

S21

B#Latin Small Letter R With Caron#ezina, M.; Kup#Latin Small Letter C With Caron#a, L.; Petzov#Latin Small Letter A With Acute#, J.; Kapus#Latin Small Letter N With Caron##Latin Small Letter A With Acute#k, M. (VUJE Inc., Trnava (SK))

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MOCHOVCE-1 REACTOR; NUCLEAR POWER PLANTS; SAMPLING; QUALITY CONTROL; REACTOR SAFETY; TENSILE PROPERTIES; YIELD STRENGTH; PRIMARY COOLANT CIRCUITS

To ensure the safe operation of NPP it is necessary to monitor and evaluate changes of mechanical properties. One possibility how to solve this problem is a direct sampling and the assessment of the actual mechanical properties using the small punch test (SPT) technique. By the SPT technique it is possible to evaluate the basic tensile properties such as the ultimate tensile strength and the yield stress of the tested materials as well as the transition temperature. The surface sampling using the special Rolls-Royce equipment SSam-2 for sampling of primary piping and steam generator at Bohunice and Mochovce NPPs is described

Для обеспечения безопасной эксплуатации АЭС необходимо контролировать и оценивать изменения механических свойств. Одной из возможностей решения данной задачи является непосредственный отбор проб и оценка фактических механических свойств с использованием метода проб на пробивку. Данный метод дает возможность оценить основные прочностные характеристики, такие как предел прочности при растяжении и предел текучести испытываемых материалов, а также критическую температуру фазового перехода хрупкости. Описан процесс отбора проб с поверхности с использованием специального оборудования SSam-2 компании Rolls-Royce, предназначенного для отбора проб из системы трубопроводов первого контура и парогенератора на АЭС Богунице и Моховце

Secondary water chemistry optimization of lead cooled reactor BREST-OD-300 S21

Kritskij, V.G.; Prokhorov, N.A.; Styazhkin, P.S.; Rodionov, Yu.A. (AO #Left Double Quotation Mark#ATOMPROEKT#Right Double Quotation Mark#, Sankt-Peterburg (RU)); Yurmanov, V.A. (AO #Left Double Quotation Mark#NIKIEhT#Right Double Quotation Mark#, Moscow (RU))

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Optimizatsiya vodno-khimicheskogo rezhima BREST-OD-300

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LIQUID METAL COOLED REACTORS; FAST REACTORS; WATER CHEMISTRY; SECONDARY COOLANT CIRCUITS; OPTIMIZATION; RESEARCH AND TEST REACTORS; SHIELDING MATERIALS; STEAM GENERATORS

The main tasks of secondary water chemistry are as follows: water chemistry compatibility with SG and cycle component materials, minimization of environmental effect, minimization of deposits growth on heat exchanger surfaces of SGs. Aspects of the selection and optimization of WCR with regard to ensuring its compatibility with structural materials are considered. This project uses a number of innovative chemical technologies based on modern achievements of nuclear and thermal power engineering, including water treatment technology, turbine condensate purification system, deaeration of SG feed water, chemical and corrosion monitoring systems

Основные задачи ВХР второго контура включают: обеспечение коррозионной совместимости ВХР с конструкционными материалами ПГ и элементов второго контура, минимизация роста отложений на поверхностях ПГ, минимизация количества отходов и сбросов в окружающую среду. Рассмотрены аспекты выбора и оптимизации ВХР с учетом обеспечения его совместимости с конструкционными материалами. В рассматриваемом проекте использован ряд инновационных химических технологий, основанных на современных достижениях атомной и тепловой энергетики, включая технологию водоподготовки, систему очистки турбинного конденсата, деаэрацию питательной воды ПГ, системы химического и коррозионного мониторинга

Depleted zinc oxide certificate for safe zinc injection into water coolant of nuclear reactors

S21

Yurmanov, E.V.; Yurmanov, V.A.; Evropin, S.V.; Shut'ko, K.I. (AO #Left Double Quotation Mark#NIKIEhT#Right Double Quotation Mark#, Moscow (RU)); Filimonov, S.V.; Zyryanov, S.M.; Timofeev, D.V. (AO #Left Double Quotation Mark#PO EhKhZ#Right Double Quotation Mark#, Zelenogorsk, Krasnoyarskij Kraj (RU))

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Razrabotka tehnikeskikh trebovanij k kachestvu oksida tsinka dlya dozirovaniya v vodnyj teplonositel' AEhS

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International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; LWGR TYPE REACTORS; ZINC OXIDES; ZINC 64; COBALT 60; SPECIFICATIONS; INJECTION; REACTOR SAFETY; QUALITY CONTROL

The prospects of the introduction of dosing of zinc depleted in the isotope ^{64}Zn into the water coolant of WWER and RBMK are shown with the aim of removing the long-lived radionuclides accumulated in oxide films, including ^{60}Co , which dominates in the radiation situation during parking for repairs and reloading of fuel, on the decommissioning of power units. The developed technical requirements for commercial zinc oxide powder include the allowable ranges of its granulometric characteristics (average and maximum pellet sizes) and isotopic composition (restriction of the share of the isotope ^{64}Zn), its specific surface area and moisture content, as well as restrictions on the content of impurities to ensure its safe use in domestic NPP. The requirements for the purity of zinc oxide powder in the specification are developed taking into account the experience of its production at the enterprises of the State Corporation Rosatom for export deliveries to foreign nuclear power plants

Показана перспективность внедрения дозирования цинка, обедненного по изотопу ^{64}Zn , в водный теплоноситель ВВЭР и РБМК с целью выведения накопленных в оксидных пленках долгоживущих радионуклидов, включая ^{60}Co , который доминирует в радиационной обстановке при стоянках для ремонтов и перегрузки топлива, а также при

работах по выводу энергоблоков из эксплуатации. Разработанные технические требования на товарный порошок оксида цинка включают допустимые диапазоны характеристик его гранулометрического (средний и максимальный размеры гранул) и изотопного состава (ограничение доли изотопа ^{64}Zn), его удельной поверхности и влагосодержания, а также ограничения по содержанию примесей для обеспечения его безопасного применения на отечественных АЭС. Требования к чистоте порошка оксида цинка в ТУ разработаны с учетом опыта его производства на предприятиях Госкорпорации Росатом для экспортных поставок на зарубежные АЭС

Prospectives of supercritical water reactors: the results and work plans on SCWR direction in the frame of international forum Generation-4

S21

Sedov, A.A. (NITs #Left Double Quotation Mark#Kurchatovskij Inst.#Right Double Quotation Mark#, Moscow (RU))

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Perspektivy SKD-reaktorov: rezul'taty i plany rabot po napravleniyu SCWR v ramkakh mezhdunarodnogo foruma Pokolenie-4

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SUPERCritical STATE; PILOT PLANTS; FORECASTING; PLANNING; INTERAGENCY COOPERATION; NUCLEAR FUELS; THERMAL HYDRAULICS; REACTOR SAFETY; WATER CHEMISTRY; CORROSION; RADIOLYSIS; TECHNOLOGY ASSESSMENT

In the beginning of 2000-ths an international project had started on elaboration of new-generation reactors - International Forum Generation 4. Six reactor directions had been chosen for the following studies as prospective. One of them had been recognized the Supercritical Water-Cooled Reactor (SCWR). The first G-4 Road Map on SCWR in support of designs of Pilot Nuclear Power Installations on the concepts appeared in 2002. Arrangements they conduct modeling and experimental studies on supercritical heat transfer and fluid dynamics, critical outflows, thermal hydraulic stability, corrosion and impurities deposition, water chemistry and radiolysis. In the present time a new Project Arrangement on Fuel Qualification Testing (FQT) is

prepared. The problems of fuel behavior, cladding corrosion, radiolysis, water chemistry and heat transfer are believed to be investigated under reactor condition

В начале 2000-ых стартовал международный проект по разработке реакторов нового поколения - Международный форум Поколение-4. Шесть реакторных направлений были отобраны для дальнейшего рассмотрения как перспективные. Одним из таких направлений было признано направление СКВР - реакторы, охлаждаемые сверхкритической водой. Первая дорожная карта НИОКР SCWR G-4 в поддержку разработок проектов пилотных ядерно-производящих установок по данным концепциям появилась в 2002 г. Проводятся эксперименты и моделирование по проблемам теплообмена и гидродинамики СКД-теплоносителей, критического истечения, теплогидравлической устойчивости, окисления и отложений примесей, водной химии и радиолитиза. В настоящее время готовится к подписанию новый Проектный Договор по направлению #Left Double Quotation Mark#Тестирование и квалификация топлива#Right Double Quotation Mark#. Предполагается изучение вопросов поведения топлива, коррозии конструкционных материалов, радиолитиза, водной химии и теплообмена в реакторных условиях

Integral reactor units (RU) with natural water circulation at supercritical pressure #En Dash# RU SCPI

S21

Semchenkov, Yu.M.; Silin, V.A.; Alekseev, P.N.; Chibinyaev, A.V.; Mit'kin, V.V.; Khlopov, R.A. (NITs #Left Double Quotation Mark#Kurchatovskij Inst.#Right Double Quotation Mark#, Moscow (RU))

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Integral'nye reaktornye ustanovki s estestvennoj tsirkulyatsiej vody pri sverkhkriticheskom davlenii - RU SKDI

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WWER TYPE REACTORS; SUPERCRITICAL STATE; CRITICAL PRESSURE; PRIMARY COOLANT CIRCUITS; POWER RANGE 100-1000 MW; EFFICIENCY; REACTOR SAFETY; TECHNOLOGY ASSESSMENT

A possibility to create a nuclear reactor with natural water circulation at super critical pressure (SCP) is considered. The arguments show that it is possible to create a reactor with an electrical output of up to 1000 MW, with an efficiency of up to 41% and one that would have the highest possible safety level. Techno-economic characteristics of such a design strongly exceed characteristics of the loop reactor units

Рассмотрена возможность создания реакторной установки с естественной циркуляцией воды при СКД. Показана возможность создания реакторной установки при единичной электрической мощности до 1000 МВт с к.п.д. до 41% с предельно достижимым уровнем безопасности. Техничко-экономические характеристики такой установки заметно превосходят характеристики петлевых реакторных установок

The experimental databases on heat transfer to water flow at supercritical pressure in a tube

S21

Kirillov, P.L.; Terent'eva, M.I.; Bogoslovskaya, G.P. (GNTs RF Fiziko-Ehnergeticheskij Inst. im. A.I. Lejpunskogo, Obninsk (RU)); Churkin, A.N. (AO OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark#, Podol'sk (RU))

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Banki ehksperimental'nykh dannyx po teplootdache k potoku vody sverkhkriticheskogo davleniya v trube

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International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; SUPERCRITICAL STATE; CRITICAL PRESSURE; VERIFICATION; DATA PROCESSING; HEAT TRANSFER; RESEARCH AND TEST REACTORS; REVIEWS

The review of the existing experimental databases on heat transfer to water flow at supercritical pressure (SCP), which the most part was obtained for vertical tubes. The considered experimental databases are used, first of all, for development, comparative analysis and verification of calculated correlations and the look-up tables on heat transfer to the SCP water flow as applied to the advanced nuclear power plants cooled by SCP water

Дается обзор существующих банков экспериментальных данных (БЭД) по теплоотдаче к потоку воды сверхкритического давления (СКД), большая часть которых была получена на вертикальных трубах. Рассмотренные БЭД применяются, прежде всего, для разработки, сравнительного анализа и верификации расчетных корреляций и скелетных таблиц по теплоотдаче к потоку воды СКД применительно к инновационным реакторным установкам, охлаждаемых водой сверхкритического давления

Research and development of manufacturing technology of welded rotor low-speed turbines for nuclear power plants of high power (WWER-1200)

S21

Nikitin, V.B.; Novikov, V.V.; Kotov, V.A. (OAO #Left Double Quotation Mark#Silovye Mashiny - LMZ#Right Double Quotation Mark#, Sankt- Peterburg (RU)); Khodakov, V.D.; Abrosin, A.A.; Bazanov, M.A. (AO NPO #Left Double Quotation Mark#TsNIITMASH#Right Double Quotation Mark#, Moscow (RU))

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Issledovanie i razrabotka tekhnologii izgotovleniya svarnykh rotorov tikhokhodnykh turbin dlya AEhS bol'shoj moshchnosti (VVEhR-1200)

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7 figs., 2 tabs.

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WWER TYPE REACTORS; NUCLEAR POWER PLANTS; STEAM TURBINES; REACTOR COOLING SYSTEMS; CHROMIUM-NICKEL-MOLYBDENUM STEELS; ROTORS; TECHNOLOGY ASSESSMENT; WELDING

Based on the data on the properties of rotor steels and the results of studies performed for the production of welded rotors of slow-moving turbines for nuclear power plants, steel

26CrNi3Mo2VN was chosen as the base metal. The thermal cycle of manufacturing of welded rotors is determined. The production certification of the developed technology was carried out. The technology was mastered in the conditions of the new plant of OJSC #Left Double Quotation Mark#Power Machines#Right Double Quotation Mark#, two full-scale rotor models of a slow-moving turbine for nuclear power plants

На основании данных о свойствах роторных сталей и результатов выполненных исследований для изготовления сварных роторов тихоходных турбин для АЭС в качестве основного металла была выбрана сталь 26CrNi3Mo2VN. Определен термический цикл изготовления сварных роторов. Выполнена производственная аттестация разработанной технологии. Технология освоена в условиях нового завода ОАО #Left Double Quotation Mark#Силовые машины#Right Double Quotation Mark#, изготовлены два полномасштабных макета ротора тихоходной турбины для АЭС

Possible ways of atomic power development on the base of water cooled reactors S21

Blagoveshchenskij, A.Ya.; Bor, S.M.; Mityukov, V.N. (Sankt-Peterburgskij Gosudarstvennyj Politehnicheskij Univ., Sankt-Peterburg (RU))

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Vozmozhnye puti razvitiya atomnoj ehnergetiki na baze ispol'zovaniya reaktorov s vodyanym teplonositelem

vp.

(RU)

2 refs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; REVIEWS; FORECASTING; TECHNOLOGY ASSESSMENT; NUCLEAR ENGINEERING; REACTOR TECHNOLOGY; REACTOR SAFETY

The prospect of the development of nuclear power with reactors with a water coolant should be based both on the solution of the urgent engineering tasks of today, and on the necessary deep scientific research of fundamentally new directions for the entire range of work in the light of ensuring nuclear safety

Перспектива развития ядерной энергетики с реакторами с водяным теплоносителем должна опираться как на решение насущных инженерных задач сегодняшнего дня, так и на необходимые глубокие научные исследования принципиально новых направлений по всему комплексу работ в свете обеспечения ядерной безопасности

Prospects for development of an innovative water-cooled nuclear reactor for supercritical parameters of coolant

S21

Kirillov, P.L.; Kalyakin, S.G.; Baranaev, Yu.D.; Glebov, A.P.; Bogoslovskaya, G.P. (GNTs RF - Fiziko-Ehnergeticheskij Inst. im. A.I. Lejpunskogo, Obninsk (RU)), e-mail: gpbogoslov@ippe.ru; Nikitenko, M.P.; Makhin, V.M.; Churkin, A.N. (AO OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark#, Podol'sk (RU))

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Perspektivy razrabotki innovatsionnogo vodookhlazhdaemogo yadernogo reaktora so sverkhkriticheskimi parametrami teplonositylya

vp.

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24 refs., 5 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; SUPERCRITICAL STATE; CRITICAL PRESSURE; FORECASTING; REVIEWS; REACTOR SAFETY; TECHNOLOGY ASSESSMENT; BEST AVAILABLE TECHNOLOGY

The state of nuclear power engineering as of February 1, 2014 and the accomplished elaborations of a supercritical pressure water cooled reactor are briefly reviewed, and the prospects of this new project are discussed based on this review. The new project rests on the experience gained from the development and operation of stationary water cooled reactor plants, including WWER, PWR, BWR, and RBMK (their combined service life totals more than 15000 reactor years), and long-term experience gained around the world with operation of thermal power plants the turbines of which are driven by steam with supercritical and ultrasupercritical parameters. The advantages of such reactor are pointed out together with the scientific technical problems that need to be solved during further development of such installations. The knowledge gained for the

last decade makes it possible to refine the concept and to commence the work on designing an experimental small-capacity reactor

На основе краткого обзора состояния атомной энергетики на 01.02.2014 и выполненных разработок водоохлаждаемого реактора на сверхкритическом давлении (ВВЭР СКД) обсуждаются перспективы этого проекта. Проект опирается на опыт создания и эксплуатации стационарных водоохлаждаемых реакторных установок ВВЭР, PWR, BWR, РБМК (более 15000 реакторо-лет), многолетний мировой опыт эксплуатации тепловых электростанций, в турбинах которых используется пар сверхкритических и супер-сверхкритических параметров. Освещаются преимущества такого реактора и научно-технические проблемы, требующие решения при дальнейшей разработке подобных установок. Накопленные за последние 10 лет знания позволяют уточнить концепцию и приступить к проектированию экспериментального реактора небольшой мощности

Experimental and computational investigations of heat exchange in fuel subassembly of reactor core in a substantiation of efficiency and safety for water cooled reactors of new generation

S21

Kirillov, P.L.; Bogoslovskaya, G.P.; Loshchinin, V.M.; Selivanov, Yu.F.; Smirnov, A.M.; Trufanov, A.A.; Pomet'ko, R.S.; Sorokin, A.P. (FGUP #Left Double Quotation Mark#GNTs RF - Fiziko-Ehnergeticheskij Inst. imeni A.I. Lejpunskogo#Right Double Quotation Mark#, Obninsk (RU))

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Ehksperimental'nye i raschetnye issledovaniya teploobmena v TVS aktivnoj zony v obosnovanie ehffektivnosti i bezopasnosti vodookhlazhdaemykh reaktorov novogo pokoleniya

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10 refs., 3 figs., 4 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; HEAT TRANSFER; FUEL ELEMENTS; SUPERCRITICAL STATE; CRITICAL PRESSURE; THERMAL HYDRAULICS; FUEL ASSEMBLIES; REACTOR CORES; REACTOR SAFETY

The analysis results of theoretical and experimental investigations of heat exchange in fuel rod bundles are presented at water flow under supercritical parameters. It is shown, that presence of the wire wrap on the fuel rods promotes to heat exchange intensification. The obtained data are used at thermohydraulic calculation of fuel subassemblies of a reactor core for experimental reactor by power 30 MW thermal (WWER-SWCP-30)

Представлены результаты анализа теоретических и экспериментальных исследований теплообмена в пучках ТВЭЛов при течении воды сверхкритических параметров. Показано, что наличие проволочной навивки на ТВЭлах способствует интенсификации теплообмена. Полученные данные использованы при теплогидравлическом расчете ТВС активной зоны экспериментального реактора мощностью 30 МВт тепловых (ВВЭР-СКД-30)

Conceptual proposals on the test reactor of WWER-SCP

S21

Mokhov, V.A.; Berkovich, V.Ya.; Nikitenko, M.P.; Makhin, V.M.; Churkin, A.N.; Lapin, A.V. (AO OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark#, Podol'sk (RU)); Kirillov, P.L.; Baranaev, Yu.D.; Glebov, A.P. (FGUP #Left Double Quotation Mark#GNTs RF - FEhI#Right Double Quotation Mark#, Obninsk (RU))

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Kontseptual'nye predlozheniya po stendu-prototipu reaktora VVEhR-SKD

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18 refs., 1 fig., 3 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; CRITICAL PRESSURE; TEST REACTORS; REACTOR COOLING SYSTEMS; REACTOR SAFETY; SPECIFICATIONS; OPTIMIZATION; RESEARCH PROGRAMS

A necessity was demonstrated of creation of a prototype of a power reactor plant with supercritical parameters of coolant (small test reactor) for experimental justification of technical solutions and optimization of selection of the design and parameters of operation of WWER-SCP (SCWR) and meeting the requirements of the basic regulatory document #Left Double Quotation Mark#General provisions of ensuring safety of nuclear power plants#Right Double Quotation

Mark# (ОПБ-88/97, p.1.2.5) concerning use of justified and experience-based technical solutions in nuclear engineering. The main technical requirements and conceptual solutions are proposed for the test reactor of reactor WWER SCP as well as proposals on a research program and a long-term economically beneficial use of this facility

Показана необходимость создания прототипа энергетической реакторной установки со сверхкритическими параметрами теплоносителя (small test reactor) для экспериментального обоснования технических решений и оптимизации выбора конструкции и параметров эксплуатации ВВЭР-СКД и выполнения требований основополагающего нормативного документа #Left Double Quotation Mark#Общие положения обеспечения безопасности атомных станций#Right Double Quotation Mark# (ОПБ-88/97, п.1.2.5) по использованию в атомной энергетике обоснованных и проверенных на практике технических решений. Предлагаются основные технические требования и концептуальные решения по прототипу реактора ВВЭР-СКД, а также предложения по программе исследований и по экономически выгодному использованию этой установки в течение длительного времени

Conceptual proposals on reactor WWER-SCW developed on the basis of technologies of WWER and steam-turbine installations at supercritical parameters

S21

Semchenkov, Yu.M.; Sidorenko, V.A.; Alekseev, P.N.; Sedov, A.A.; Silin, V.A. (NITs #Left Double Quotation Mark#Kurchatovskij Inst.#Right Double Quotation Mark#, Moscow (RU)); Mokhov, V.A.; Nikitenko, M.P.; Churkin, A.N.; Makhin, V.M. (AO OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark#, Podol'sk (RU))

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9-ya mezhdunarodnaya nauchno-tehnicheskaya konferentsiya Obespechenie bezopasnosti AEhS s VVEhR

Kontseptual'nye predlozheniya po reaktoru VVEhR-SKD, sozdavaemomu na osnove tekhnologij VVEhR i paroturbinnnykh ustanovok na SKD parametrakh

vp.

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6 figs., 6 tabs.

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomehnergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomehnergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; SUPERCRITICAL STATE; CRITICAL PRESSURE; REACTOR COOLING SYSTEMS; FUEL CYCLE; NUCLEAR POWER PLANTS; CLOSED-CYCLE COOLING SYSTEMS; RECOMMENDATIONS

The Technical Proposals for the design of the NPP with WWER with supercritical parameters of the coolant (WWER-SKD) for three variants of reactors with light-water SKD heat-carrier are formulated. WWER-SKD-1700: single-circuit switchgear with one-and two-pass active zone with a fast resonance neutron spectrum and a loop layout (input: feed water, output - superheated steam); B-670 SKDI: a two-circuit RU with natural circulation of the coolant, with a resonant-thermal spectrum of neutrons in the integrated configuration (input: in-vessel water, output - slightly superheated steam); PSKD-600: a two-circuit RU with a fast neutron spectrum and a loop layout (input: slightly superheated steam, output - superheated steam). The considered variants of the WWER-SCW concepts are briefly described, and their main technical features and characteristics are indicated. The key problems for R&D in support of design of WWER-SCW are specified

Сформулированы Технические Предложения по проекту АЭС с ВВЭР со сверхкритическими параметрами теплоносителя (ВВЭР-СКД) по трем вариантам реакторов с легководным СКД-теплоносителем. ВВЭР-СКД-1700: одноконтурная РУ с одно- и двухзаходной активной зоной с быстрорезонансным спектром нейтронов и петлевой компоновкой (вход: питательная вода; выход - перегретый пар); В-670 СКДИ: двухконтурная РУ с естественной циркуляцией теплоносителя, с резонансно-тепловым спектром нейтронов в интегральной компоновке (вход: внутрикорпусная вода; выход - слабоперегретый пар); ПСКД-600: двухконтурная РУ с быстрым спектром нейтронов и петлевой компоновкой (вход: слабоперегретый пар; выход - перегретый пар). Представлены рассмотренные варианты концепции ВВЭР-СКД, их основные технические особенности и характеристики. Приведен перечень ключевых проблем, сформулированных для проведения НИОКР в поддержку разработки проекта ВВЭР-СКД

Formation of technical requirements to WWER-S

S21

Semchenkov, Yu.M.; Subbotin, S.A.; Alekseev, P.N.; Chibinyaev, A.V. (NITs #Left Double Quotation Mark#Kurchatovskij Inst.#Right Double Quotation Mark#, Moscow (RU)); Mokhov, V.A.; Nikitenko, M.P.; Veselov, D.O. (AO OKB #Left Double Quotation Mark#GIDROPRESS#Right Double Quotation Mark#, Podol'sk (RU)); Kuchumov, A.Yu.; Berkovich, V.M. (AO #Left Double Quotation Mark#Atomenergoproekt#Right Double Quotation Mark#, Moscow (RU))

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Formirovanie tekhnicheskikh trebovanij k VVEhR-S

вр.

(RU)

International Atomic Energy Agency, Vienna (XA); Gosudarstvennaya Korporatsiya po Atomnoj Ehnergii Rosatom, Moscow (RU); AO Atomenergomash, Moscow (RU); AO Kontsern Rosehnergoatom, Moscow (RU); AO Atomenergoproekt, Moscow (RU); AO ATOMPROEKT, Sankt-Peterburg (RU); AO TVEhL, Moscow (RU); NITs Kurchatovskij Inst., Moscow (RU); AO OKB GIDROPRESS, Podol'sk (RU)

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WWER TYPE REACTORS; REACTOR TECHNOLOGY; TECHNOLOGY ASSESSMENT;
FUEL CYCLE; REACTOR CORES; FUEL CONSUMPTION; SPECIFICATIONS

WWER design with its core reactivity spectrum shift to be controlled in the process of burnup (WWER-S) is the evolutionary development line of WWER. Many ways are known to control this spectrum shift by varying either the water-to-fuel ratio, or the light-to-heavy water ratio, or the steam content in water, etc. The basic feature of the WWER-S design is that its core reactivity spectrum shift is controlled by movable water displacers applied to change the water-to-uranium ratio in about a half of its fresh fuel assemblies - this makes it possible to completely exclude liquid-boron control from the operational modes

ВВЭР со спектральным регулированием изменения реактивности активной зоны в процессе выгорания топлива (ВВЭР-С) - это эволюционное развитие технологии корпусных водо-водяных энергетических реакторов. Известны различные способы спектрального регулирования, например, за счет изменения водо-топливного отношения, изменения соотношения тяжелой и легкой воды в активной зоне, изменения паросодержания в воде. Принципиальное отличие ВВЭР-С заключается в спектральном регулировании за счет изменения водно-уранового соотношения примерно в половине более свежих ТВС активной зоны с использованием подвижных вытеснителей воды и полном отказе от жидкостного борного регулирования в эксплуатационных режимах работы

Prospects for the use of the WWER-SKD in a closed fuel cycle

S21

Baranaev, Yu.D.; Glebov, A.P.; Klushin, A.V. (FGUP #Left Double Quotation Mark#GNTs RF-FEhI im. A.I. Lejpunskogo#Right Double Quotation Mark#, Obninsk (RU))

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REACTOR SAFETY; SUPERCRITICAL STATE; CRITICAL PRESSURE; NEUTRON
SPECTRA; ENERGY POLICY; FAST NEUTRONS; REVIEWS

AE development programs in the countries of advanced energy is considered. The features of AE development and CFC implementation in Russia are discussed, as well as their stages, periods of their execution, problems. Substantiation of the use of SCWR reactors with fast neutron spectrum in the systems with CFC is given

Рассматриваются программы развития АЭ в странах с развитой энергетикой. Обсуждаются особенности развития АЭ и реализации ЗТЦ в России, этапы, сроки их выполнения, возникающие проблемы. Обосновывается использование реакторов SCWR с быстрым спектром нейтронов в системах с ЗТЦ

Radioactivity after the nuclear explosions and accidents: effects and ways to overcome. Proceedings of the International scientific-practical conference S54

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

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LEADING ABSTRACT; MEETINGS; RADIATION ACCIDENTS; CONTAMINATION;
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SURFACE AREA; SURFACE CONTAMINATION; SURFACE WATERS; NUCLEAR
EXPLOSIONS

The scientific-practical conference #Left-Pointing Double Angle Quotation Mark#Radioactivity after the nuclear explosions and accidents: effects and ways to overcome#Right-Pointing Double Angle Quotation Mark# dealing with the 30-th anniversary of the Chernobyl accident took place in Obninsk on April 19-21, 2016. On the conference there was a discussion on the questions in the field of monitoring and investigation of radiation situation after nuclear tests, peaceful

nuclear explosions; assessment of consequences of radiation accidents on the South Urals, Chernobyl NPP, Fukushima NPP. The consideration was given to the problems of further development of radiation monitoring system and methods and techniques, database forming; assessment of consequences of radioactive waste burial in the North seas of Russia. The great attention was paid to the ecological aspects of contamination, radioactive substance emissions and discharges; estimation of radiation-ecological risks, rehabilitation of contaminated territories, etc.

Научно-практическая конференция #Left-Pointing Double Angle Quotation Mark#Радиоактивность после ядерных взрывов и аварий: последствия и пути преодоления#Right-Pointing Double Angle Quotation Mark#, посвященная 30-летию аварии на Чернобыльской АЭС, проходила 19-21 апреля 2016 г. в Обнинске. На конференции обсуждались вопросы в области мониторинга и исследований радиационной обстановки после испытаний ядерного оружия, мирных подземных взрывов; оценки последствий радиационных аварий на Южном Урале, Чернобыльской АЭС, АЭС Фукусима. Рассмотрены вопросы дальнейшего развития системы, методов и средств радиационного мониторинга, формирования баз данных; особенностей мониторинга в районах размещения объектов использования атомной энергии; оценки последствий захоронения радиоактивных отходов в северных морях России. Большое внимание уделялось экологическим аспектам радиоактивного загрязнения, выбросов и сбросов радиоактивных веществ; оценке радиационно-экологических рисков, реабилитации загрязненных территорий и т.д.

Estimation of radionuclide contamination of water bodies of the territory of the former Semipalatinsk Test Site

S54

Ajdarkhanova, A.K.; Lukashenko, S.N. (Natsional'nyj Yadernyj Tsentr Respubliki Kazakhstan, Kurchatov (KZ))

Radioactivity after the nuclear explosions and accidents: effects and ways to overcome

Obninsk (RU)

19-21 Apr 2016

Radioaktivnost' posle yadernykh vzryvov i avarij: posledstviya i puti preodoleniya

Otsenka radionuklidnogo zagryazneniya vodnykh ob'ektov territorii byvshego Semipalatinskogo ispytatel'nogo poligona

p. 6-13

(RU)

4 refs., 2 figs., 3 tabs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po

Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

Radioactivity after the nuclear explosions and accidents: effects and ways to overcome.

Proceedings of the International scientific-practical conference

Radioaktivnost' posle yadernykh vzryvov i avarij: posledstviya i puti preodoleniya. Sbornik

trudov Mezhdunarodnoj nauchno-prakticheskoy konferentsii

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Obninsk (RU)

FGBU NPO Tajfun

2016

416 p.

SEMPALATINSK TEST SITE; SURFACE WATERS; CONTAMINATION;
RADIOISOTOPES; RADIONUCLIDE MIGRATION; GAMMA SPECTROSCOPY; BETA
SPECTROSCOPY; ALPHA SPECTROSCOPY; DISTRIBUTION

On the Semipalatinsk test site territory radioactive contamination of artificially and naturally impounded bodies, as well as streams, has been estimated. Content of technogenic radionuclides ^3H , ^{90}Sr , ^{239}Pu , ^{240}Pu , ^{241}Am , ^{137}Cs has been determined. It has been pointed out that at present time the radionuclide contamination of water bodies surfaces has an ambiguous character: high content of radionuclide on the territories of test sites, and they are absent on the figuratively pure ones. Annual monitoring of specified water bodies shows that now there is no change of radionuclide and chemical composition of water

На территории Семипалатинского испытательного полигона проведена оценка радиоактивного загрязнения водоемов техногенного и природного происхождения, а также водотоков. Определялось содержание техногенных радионуклидов ^3H , ^{90}Sr , ^{239}Pu , ^{240}Pu , ^{241}Am , ^{137}Cs . Отмечается, что в настоящее время радионуклидное загрязнение поверхности водных объектов носит неоднозначный характер: на территории испытательных площадок сохраняется высокое содержание радионуклидов, а на условно чистых они отсутствуют. Ежегодный мониторинг указанных водных объектов показывает, что изменение радионуклидного и химического состава воды не наблюдается

Coastal monitoring of the Barents Sea in 2006-2015

S54

Artem'ev, G.B.; Uvarov, A.D.; Valetova, N.K.; Petrenko, G.I.; Polukhina, A.M.; Epifanov, A.O.; Kashirtseva, I.V. (FGBU #Left-Pointing Double Angle Quotation Mark#NPO#Left-Pointing Double Angle Quotation Mark#Tajfun#Right-Pointing Double Angle Quotation Mark#, Obninsk (RU))

Radioactivity after the nuclear explosions and accidents: effects and ways to overcome

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Pribrezhnyj monitoring Barentseva morya v 2006-2015 godakh

p. 14-18

(RU)

4 refs., 2 figs., 2 tabs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po
Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

Radioactivity after the nuclear explosions and accidents: effects and ways to overcome.
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2016

416 p.

SEAS; ARCTIC OCEAN; RADIATION MONITORING; INTERNATIONAL COOPERATION; RUSSIAN FEDERATION; NORWAY; RADIOISOTOPES; COMPARATIVE EVALUATIONS

Data, obtained during integrated radionuclide monitoring of the Barents Sea coast conducted by Russian and Norwegian scientists in 2006-2015, are presented. It is pointed out that radionuclide analysis data of marine environment sampled by Russian coastal monitoring station indicate the absence of any effect of radiation-hazardous objects of Kola Peninsula and radionuclide transport from the disposal sites of such objects. Measured levels of ^{137}Cs , ^{90}Sr , ^{239}Pu , ^{240}Pu and ^3H are within the range which is character for radioactive contamination of the Barents Sea

Представлены данные, полученные в ходе комплексного радионуклидного мониторинга побережья Баренцева моря, проводимого российскими и норвежскими учеными в 2006-2015 годах. Отмечается, что данные радионуклидного анализа проб морской среды, отобранных на российской станции прибрежного мониторинга, свидетельствуют об отсутствии какого-либо влияния деятельности радиационно-опасных объектов Кольского полуострова и переноса радионуклидов с мест захоронения таких объектов. Измеренные уровни содержания ^{137}Cs , ^{90}Sr , ^{239}Pu , ^{240}Pu и ^3H находятся в диапазоне, характерном для радиоактивного загрязнения Баренцева моря

Estimation of cesium-137 migration intensity in typical soils of the Republic of Belarus

S54

Bakarikova, Zh.V.; Zhukova, O.M.; Samsonov, V.L. (Respublikanskij Tsentr po Gidrometeorologii, Kontrolyu Radioaktivnogo Zagryazneniya i Monitoringu Okruzhayushchej Sredy, Minsk (BY))

Radioactivity after the nuclear explosions and accidents: effects and ways to overcome
Obninsk (RU)

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Otsenka intensivnosti migratsii tseziya-137 v tipichnykh pochvakh Respubliki Belarus'
p. 19-25

(RU)

2 refs., 6 figs., 1 tab.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

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416 p.

BELARUS; SOILS; CESIUM 137; RADIONUCLIDE MIGRATION; DISTRIBUTION; FORECASTING; TIME DEPENDENCE

The research results of vertical ^{137}Cs migration in four types of soils specific to the Republic of Belarus are presented. It is shown that after 20 years of the Chernobyl accident diffusion is still the dominating mechanism of vertical migration as it was in the first period after the accident; convective transport now has no significant role in ^{137}Cs redistribution. The main part of radionuclides are fixed in the soil; geochemical barriers (heavy layers of sod cover, humus horizons, clay partings) determine the reduction of migration processes intensity. The bulk of ^{137}Cs is located in the the upper root layer of the soil. The necessity of taking into the account the change of ^{137}Cs migration rate with time at forecasting of its distribution on vertical profile of soils is pointed out

Представлены результаты исследований процессов вертикальной миграции ^{137}Cs в четырех видах почв, характерных для Республики Беларусь. Показано, что спустя 20 лет после Чернобыльской аварии диффузия остается доминирующим механизмом вертикальной миграции, как и в первый период после аварии; конвективный перенос перестал играть значимую роль в перераспределении ^{137}Cs . Большая часть радионуклидов находится в почве в фиксированной форме; наличие геохимических барьеров (мощных слоев дернины, перегнойных горизонтов, прослойки глинистых материалов) обуславливает снижение интенсивности миграционных процессов. Основная доля ^{137}Cs находится в верхнем корнеобитаемом слое почвы. Отмечается необходимость учета изменения с течением времени скорости миграции ^{137}Cs в почве при прогнозировании его распределения по вертикальному профилю почв

Software solution for operational analysis and forecast of emergency radioactive substances distribution in environmental objects

S54

Borodin, R.V.; Kolomeev, M.P.; Pokhil, A.Yu.; Krylova, A.V. (FGBU #Left-Pointing Double Angle Quotation Mark#NPO#Left-Pointing Double Angle Quotation Mark#Tajfun#Right-Pointing Double Angle Quotation Mark#, Obninsk (RU))

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Radioaktivnost' posle yadernykh vzryvov i avarij: posledstviya i puti preodoleniya
Programmnyj kompleks dlya operativnogo analiza i prognoza avariynogo rasprostraneniya
radioaktivnykh veshchestv v ob"ektakh okruzhayushchej sredy

p. 26-40

(RU)

15 refs., 8 figs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po
Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

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416 p.

CONTAMINATION; ENVIRONMENT; RADIATION MONITORING; RUSSIAN FEDERATION; R CODES; DATA BASE MANAGEMENT; DATA PROCESSING; DATA ACQUISITION SYSTEMS; RADIATION PROTECTION; DOSE COMMITMENTS

The RECASS system is the base of the software and hardware suite of the Federal information and analytical center on immediate and forward-looking information in emergencies deal with accidental pollution on the territory of the Russian Federation. The main tasks of the software suit are: continuous monitoring of environmental state and contamination, data processing; on-line analysis of the situation and forecasting of pollutant distribution in emergencies; dose commitments on population; making recommendations on radiation protection for population in emergencies; simultaneous multiuser access to on-line and calculated data; possibility of calculations for trainings and drills. Software and calculational block is constructed on modular concept. The use of the system make it possible to reconstruct spatial-temporal picture of territory contamination after Chernobyl, Siberian Chemical integrated plant, Fukushima accidents with acceptable reliability

Основу программно-технического комплекса Федерального информационно-аналитического Центра по обеспечению оперативной и прогностической информацией в чрезвычайных ситуациях, связанных с аварийным загрязнением окружающей среды на территории России, составляет система RECASS. Основные задачи программного комплекса: обеспечение непрерывного сбора оперативных данных о состоянии и уровне загрязнения окружающей среды, метеорологической информации, организация загрузки, хранения и архивизации поступающих данных; оперативный анализ обстановки и прогноз распространения загрязнения в случае аварийных выбросов в окружающую среду; расчет дозовых нагрузок на население; выработка рекомендаций по проведению защитных мероприятий для населения в случае аварийных ситуаций; обеспечение одновременного многопользовательского доступа к оперативным и расчетным данным; обеспечение возможности проведения учений и тренировок. Программно-расчетный блок системы построен по модульному принципу. Использование системы позволило с приемлемой надежностью восстановить пространственно-временную картину загрязнения территории вследствие аварий на Чернобыльской АЭС, Сибирском Химическом Комбинате, аварии на АЭС Фукусима

Plutonium in the areas of local sources location and its integration into the global circulation

S54

Bulgakov, V.G.; Gnilomedov, V.D.; Katkova, M.N.; Petrenko, G.I.; Bezuglova, N.V. (FGBU #Left-Pointing Double Angle Quotation Mark#NPO#Left-Pointing Double Angle Quotation Mark#Tajfun#Right-Pointing Double Angle Quotation Mark#, Obninsk (RU))

Radioactivity after the nuclear explosions and accidents: effects and ways to overcome
Obninsk (RU)

19-21 Apr 2016

Radioaktivnost' posle yadernykh vzryvov i avarij: posledstviya i puti preodoleniya Plutonij v rajonakh raspolozheniya lokal'nykh istochnikov i ego vovlechennost' v global'nyu tsirkulyatsiyu

p. 41-52

(RU)

7 refs., 12 figs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po

Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)
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PLUTONIUM 238; PLUTONIUM 239; PLUTONIUM 240; ENVIRONMENT;
RADIONUCLIDE MIGRATION; EARTH ATMOSPHERE; RADIOACTIVITY; NUCLEAR
FACILITIES; CONTAMINATION; NUCLEAR EXPLOSIONS; RUSSIAN FEDERATION

The levels of average annual and average monthly volumetric activity (VA) of plutonium isotopes in Chelyabinsk, Kaluga, Kursk regions, where nuclear objects are located, as well as in the territories contaminated after the Chernobyl accident, are considered. In the regions with nuclear objects the maximal values of VA of plutonium isotopes according to years and during the year are distributed in random manner. In the regions contaminated after the Chernobyl accident annually during the warm period the increase of VA of plutonium isotopes is steadily fixed with the maximal concentrations in July-September, apparently due to wind rise increasing in this period. The observed VA of ^{239}Pu and ^{240}Pu in the surface layers of the atmosphere of Obninsk, Bryansk, Kursk was by 6-7 orders and in the area of #Left-Pointing Double Angle Quotation Mark#Mayak#Right-Pointing Double Angle Quotation Mark# Plant by 3-4 orders lower the value of permissible VA for population according to the Radiation safety standards NRB-99/2009 which is equal to $2,5 \times 10^{-3} \text{ Bq/m}^3$

Рассмотрены уровни среднегодовой и среднемесячной объемной активности (ОА) изотопов плутония в Челябинской, Калужской, Курской областях, где расположены объекты атомной промышленности, а также на территориях, загрязненных после Чернобыльской аварии. В районах расположения ядерных объектов максимальные значения ОА изотопов плутония по годам и в течение года распределены случайным образом. В районах, подвергшихся загрязнению после Чернобыльской аварии, ежегодно в теплый период устойчиво фиксируется увеличение ОА изотопов плутония с максимальными концентрациями в июле-сентябре, что, по-видимому, связано с ветровым подъемом, усиливающимся в этот период. Наблюдаемая ОА ^{239}Pu и ^{240}Pu в приземном слое атмосферы Обнинска, Брянска, Курска была на 6-7 порядков, а в районе ПО #Left-Pointing Double Angle Quotation Mark#Маяк#Right-Pointing Double Angle Quotation Mark# - на 3-4 порядка ниже величины допустимой ОА для населения по Нормам радиационной безопасности НРБ-99/2009, равной $2,5 \times 10^{-3} \text{ Бк/м}^3$

Experience in knowledge preservation on radioactive accident consequences and actions on their overcoming in terms of the Chernobyl accident **S54;S96**

Bulgakov, V.G.; Katkova, M.N.; Kupriyanova, I.A.; Ageeva, N.V. (FGBU #Left-Pointing Double Angle Quotation Mark#NPO#Left-Pointing Double Angle Quotation Mark#Tajfun#Right-Pointing Double Angle Quotation Mark#, Obninsk (RU))

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Obninsk (RU)
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Radioaktivnost' posle yadernykh vzryvov i avarij: posledstviya i puti preodoleniya
Opyt sokhraneniya znaniy o posledstviyakh radiatsionnykh avarij i dejstviyakh po ikh
preodoleniyu na primere Chernobyl'skoj avarii

p. 53-59

(RU)

3 figs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po
Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

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REACTOR ACCIDENTS; CHERNOBYLSK-4 REACTOR; DATA BASE MANAGEMENT;
INFORMATION SYSTEMS; ENVIRONMENT; CONTAMINATION; KNOWLEDGE
PRESERVATION; RUSSIAN FEDERATION; DOCUMENTATION

In the Research and Production Association #Left-Pointing Double Angle Quotation
Mark#Tajfun#Right-Pointing Double Angle Quotation Mark# by the year 2015 the Technical
project of information system with large archive, containing 280 units of information, and
databases on the RF territories (24 territorial entries) contaminated after the Chernobyl accident
has been realized. The main aim is the creation of the storage of accurate information on
environmental contamination for wide range of specialists, population and decision-makers of
the regions. Besides, documents collected in one place permit to analyse all overpast actions on
mitigation of consequences of radiation accident of such range

В НПО #Left-Pointing Double Angle Quotation Mark#Тайфун#Right-Pointing Double Angle
Quotation Mark# к 2015 г. реализован Технический проект информационной системы с
большим архивом, содержащим 280 единиц информации, и базами данных о загрязнении
территорий РФ (24 субъекта) после аварии на Чернобыльской АЭС. Основная цель -
создать хранилище достоверной информации о загрязнении окружающей среды, которым
будут пользоваться широкие круги специалистов, населения и лиц, принимающих
административные решения в регионах. Кроме того, собранные в одном месте документы
дают возможность анализировать все прошлые действия по смягчению последствий
радиационной аварии такого масштаба

Results of Roshydromet agencies activities on radiation monitoring of the territories affected the Chernobyl accident (on early stage)

S54

*Vakulovskij, S.M. (FGBU #Left-Pointing Double Angle Quotation Mark#NPO#Left-Pointing
Double Angle Quotation Mark#Tajfun#Right-Pointing Double Angle Quotation Mark#, Obninsk
(RU))*

Radioactivity after the nuclear explosions and accidents: effects and ways to overcome

Obninsk (RU)

19-21 Apr 2016

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Rezul'taty raboty uchrezhdenij Rosgidrometa po monitoringu radiatsionnoj obstanovki na
territoriyakh, postradavshikh vsledstvie avarii na ChAEhS (na rannej stadii)
p. 60-72

(RU)

6 refs., 6 figs., 5 tabs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po
Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

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REACTOR ACCIDENTS; CHERNOBYLSK-4 REACTOR; ENVIRONMENT;
CONTAMINATION; METEOROLOGY; MONITORING; SURFACE WATERS; SOILS;
DECONTAMINATION

The work of the radiometric divisions of Hydrometeorological service on the USSR territory, first aero- and surface radiation surveys just after the Chernobyl accident are described. As a result of these works the map of contamination of the nearest zone of the emergency reactor, map-scheme of soil contamination of Kaluga and Tula Regions with the areas of increased contamination, have been constructed. The first estimations of radioactive contamination of surface waters, possibilities of the territory decontamination have been given. The organization of mass gamma-spectroscopic analysis of soil samples in Kiev, Obninsk, Minsk have been considered

Описана работа радиометрических подразделений гидрометеослужбы на территории СССР, первые авиационные и наземные радиационные разведки в первые дни и недели после аварии на Чернобыльской АЭС. По результатам этих работ были составлены карта радиоактивного загрязнения ближней к аварийному реактору зоны, карта-схема загрязнения почв в Калужской и Тульской областях, где была выявлена зона повышенного загрязнения. Приведены первые оценки радиоактивного загрязнения водных объектов, возможности дезактивации территорий. Рассмотрена организация массовых гамма-спектрометрических анализов проб почв в Киеве, Обнинске, Минске

Water problems and hydrological regime monitoring after nuclear explosions and accidents (the case of the Chernobyl accident)

S54

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Radioactivity after the nuclear explosions and accidents: effects and ways to overcome

Obninsk (RU)

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Radioaktivnost' posle yadernykh vzryvov i avarij: posledstviya i puti preodoleniya

Vodnye problemy i monitoring gidrologicheskogo rezhima posle yadernykh vzryvov i avarij (na primere avarii na ChAEhS)

p. 73-83

(RU)

9 refs., 3 figs., 2 tabs.

Shershakov, V.M. (ed.)

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416 p.

CHERNOBYLSK-4 REACTOR; REACTOR ACCIDENTS; CONTAMINATION; SURFACE WATERS; CONTROLLED AREAS; HYDROLOGY

The results of hydrological measurements conducted in contaminated zone of the Chernobyl NPP and Kiev reservoir in 1986-1991 are presented. It is shown that some measures related with the attempts to prevent surface and subsurface runoff from the Chernobyl accident zone have been realized without due hydrological justification, and it had negative consequences. It is pointed out that all measures on preventing and decreasing radioactive contaminants transport by water flows must have operational hydrological justification at deficient account of peculiarities of water and sediment runoffs forming

Представлены результаты гидрологических наблюдений, проведенных в зоне загрязнения Чернобыльской АЭС и Киевского водохранилища в 1986-1991 годы. Показано, что некоторые мероприятия, связанные с попытками предотвращения стока поверхностных и подземных вод из зоны Чернобыльской АЭС, проводились без должного гидрологического обоснования, что привело к негативным последствиям. Отмечается, что все мероприятия по предотвращению и уменьшению транспорта радиоактивных загрязнений водными потоками должны иметь оперативное гидрологическое сопровождение при недостаточном учете особенностей формирования стока воды и наносов

Detection of technogenic radioactive emissions in the surface layer of the atmosphere during UVF-2 real-time operation

S54

Volokitin, A.A.; Polyanskaya, O.N.; Yakhryushin, V.N. (FGBU #Left-Pointing Double Angle Quotation Mark#NPO#Left-Pointing Double Angle Quotation Mark#Tajfun#Right-Pointing Double Angle Quotation Mark#, Obninsk (RU))

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Obnaruzhenie tekhnogennykh radioaktivnykh vybrosov v prizemnom sloe atmosfery pri rabote UVF-2 v rezhime real'nogo vremeni

p. 84-92

(RU)

7 refs., 8 figs.

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Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po

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**RADIOACTIVE AEROSOLS; AEROSOL MONITORING; SURFACE AIR; AIR FILTERS;
RADIOACTIVITY; ACCURACY; ON-LINE MEASUREMENT SYSTEMS; RADIATION
DETECTORS**

The analysis of data collected in 2007-2012 during operation of the air-filtering unit (AFU) of new generation are presented. AFU measures air volume flow with high accuracy, simultaneously with real-time sampling it measures β -activity and γ -radiation intensity by detection blocks located under accumulative filter. The use of AFU makes it possible to record the moment of the arrival of air contaminated by technogenic radionuclides with precision and to determine the time of its passage. Using the group of AFU it is permissible to determine the location of radiation-hazardous object with accuracy and to identify its emissions. The most interest and significant situations recorded during AFU-2 operation in Obninsk are under consideration

Представлены результаты анализа данных, накопленных в 2007-2012 гг. при эксплуатации воздухо-фильтрующей установки (УВФ) нового поколения. УВФ измеряет объемный расход воздуха с высокой точностью, одновременно с отбором проб измеряет в реальном времени β -активность и мощность дозы γ -излучения с помощью блоков детектирования, расположенных под накопительным фильтром. Использование УВФ позволяет с высокой степенью точности фиксировать момент прихода атмосферного воздуха, загрязненного техногенными радионуклидами, и определять время его прохождения. При использовании группы УВФ можно точно определить местоположение радиационно-опасного объекта и идентифицировать его выбросы. Рассмотрены наиболее интересные и значимые ситуации, которые были зарегистрированы в процессе эксплуатации УВФ-2 в г. Обнинске

Peculiarities of radionuclide migration simulation in sea medium under radiation accidents

S54

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Osobennosti modelirovaniya migratsii radionuklidov v morskoy srede pri radiatsionnykh
avariyakh

p. 93-119

(RU)

41 refs., 1 fig., 7 tabs.

Shershakov, V.M. (ed.)

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416 p.

SEAWATER; CONTAMINATION; RADIONUCLIDE MIGRATION; ADSORPTION;
BIOADSORBENTS; DEPOSITION; RADIOECOLOGY; AQUATIC ECOSYSTEMS

General approaches and regularities which is necessary to radionuclide behaviour simulation in sea medium are analysed. The analysis and estimation of the main parameters characterizing radioecological situation in sea medium are carried out. The suggestions and recommendations on structural and parametric software of model calculations for radioecological situation forecasting, which includes radiation accidents, are worked out. It is shown that in sea ecosystems lithodynamic and biological adsorption are the main factors for radionuclide removal from water. Consideration is given to 8 types of objects which are the sources of real or potential contamination of seawater. It is pointed out that the main attention when selecting parameters of radiological models must be given to the block of radionuclide determination in abiotic and biological components of sea medium

Проанализированы общие подходы и закономерности, необходимые для моделирования поведения радионуклидов в морской среде. Проведены анализ и оценка значений основных параметров, характеризующих радиоэкологическую обстановку в морской среде. Выработаны предложения и рекомендации по структурному и параметрическому обеспечению модельных расчетов для прогнозирования радиоэкологической обстановки, в том числе при радиационных авариях. Показано, что в морских экосистемах основной вклад в выведение радионуклидов из воды дают литодинамическая и биологическая адсорбция. Рассмотрены 8 основных видов объектов, которые являются источниками реального или потенциального радиоактивного загрязнения морской среды. Отмечается, что основное внимание при выборе параметров радиологических моделей следует уделять блоку определения радионуклидов в абиотических и биологических компонентах морской среды

Certain calculations of radioactive contamination of environment and estimation of radiation doses of population from radioactive emissions of Kola NPP

S54

Gaziev, I.Ya.; Kryshev, I.I. (FGBU #Left-Pointing Double Angle Quotation Mark#NPO#Left-Pointing Double Angle Quotation Mark#Tajfun#Right-Pointing Double Angle Quotation Mark#,

Obninsk (RU)

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Otdel'nye raschety radioaktivnogo zagryazneniya okruzhayushchej sredy i otsenki doz
oblucheniya naseleniya ot radioaktivnykh vybrosov Kol'skoj AEhS

p. 120-123

(RU)

4 refs., 1 fig., 4 tabs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po
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**KOLA-1 REACTOR; NUCLEAR POWER PLANTS; CONTROLLED AREAS;
CONTAMINATION; RADIATION DOSES; POPULATIONS**

Radioecological situation in controlled area of the Kola NPP is under consideration. The value of ^{60}Co emissions in 2014 was $4,1 \times 10^6$ Bq, the emissions of other radionuclides were lower than detection limits. It is shown that total radiation dose of population from ^{60}Co emissions of the Kola NPP is equal to $6,6 \times 10^{-1}$ Sv/year. The main part of this dose is radiation from soil

Рассмотрена радиоэкологическая ситуация в санитарно-защитной зоне Кольской АЭС. Величина выбросов ^{60}Co в 2014 г. составила $4,1 \times 10^6$ Бк, выбросы других радионуклидов были ниже пределов обнаружения. Показано, что суммарная доза облучения населения от выбросов ^{60}Co Кольской АЭС составляет $6,6 \times 10^{-1}$ Зв/год. Наибольший вклад в эту дозу вносит облучение от почвы

Statistical and structural properties of radioactive contamination formed by atmospheric precipitations

S54

Grubich, A.O. (ZAO #Left-Pointing Double Angle Quotation Mark#TIMET#Right-Pointing Double Angle Quotation Mark#, Minsk (BY)); Zhukova, O.M. (Respublikanskiy Tsentr po Gidrometeorologii, Kontrolyu Radioaktivnogo Zagryazneniya i Monitoringu Okruzhayushchej Sredy, Minsk (BY))

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Statisticheskie i strukturnye svoystva radioaktivnogo zagryazneniya, obrazovannogo
atmosfernymi vypadeniyami

p. 124-132

(RU)

14 refs., 3 figs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po

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**RADIOACTIVE AEROSOLS; SOILS; SEDIMENTATION; RADIOACTIVITY; SURFACE
CONTAMINATION; SPATIAL DISTRIBUTION; STATISTICAL MODELS; FRACTALS**

The main properties of radioactive deposit (sediment formation where soil is a matrix with interstitial radionuclides) are determined by the process of radioactive aerosols and dust sedimentation on the earth surface. Thus, gamma-emitting radionuclides one can consider as a convenient marker, distribution of which on the earth surface gives information on peculiarities of nonradioactive pollutant distribution. It is pointed out that distribution conditioned by the process of atmospheric precipitations itself is nothing but distribution of activity density. The conclusion is made that lognormal distribution of activity density value is the primary universal property of deposit and multifractal spatial distribution of pollutant is the fundamental property of a deposit

Основные свойства радиоактивного отложения (осадочного образования, в котором почва является матрицей с внедренными радионуклидами) обусловлены процессом оседания радиоактивных аэрозолей и пыли на поверхность земли. Таким образом, радионуклид, распадающийся с испусканием гамма-излучения, можно рассматривать как удобный для выполнения измерений маркер, распределение которого по поверхности земли предоставляет информацию об особенностях распределения нерадиоактивного поллютанта. Отмечается, что распределением, обусловленным собственно процессом атмосферных выпадений, является исключительно распределение плотности активности. Сделан вывод, что логнормальность распределений величины плотности активности является первичным универсальным свойством отложения, а мультифрактальное пространственное распределение поллютанта является фундаментальным свойством отложения

Radioactive rare gases emission at underground nuclear explosions

S54

Dubasov, Yu.V. (AO #Left-Pointing Double Angle Quotation Mark#Radiyevy Inst. im. V.G.

Khlopina#Right-Pointing Double Angle Quotation Mark#, Sankt-Peterburg (RU))

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Istechenie radioaktivnykh blagorodnykh gazov pri podzemnykh yadernykh vzryvakh

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Shershakov, V.M. (ed.)

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NUCLEAR EXPLOSIONS; NOVAYA ZEMLYA; SEMIPALATINSK TEST SITE;

UNDERGROUND EXPLOSIONS; NUCLEAR EXPLOSION DETECTION; RARE GASES;

RADIOISOTOPES; RADIOACTIVITY; RADIATION DOSES

The examples of radioactive rare gases emission at underground nuclear explosions conducted in the USSR on the Novaya Zemlya and Semipalatinsk test sites are considered. It is pointed out that in the case of evasive explosion in vertical wells without apparent radioactive gases emission the samples of subsurface gas must contain the traces of radioactive rare gases. Under the inspection of evasive explosion in horizontal workings of rock massif, one should be guided by the analysis of atmospheric air samples in the inspected area

Рассмотрены примеры истечения радиоактивных благородных газов при подземных ядерных взрывах, проведенных в СССР на Семипалатинском и Новоземельском полигонах. Отмечается, что в случае проведения скрытого взрыва, осуществленного в вертикальной скважине и без явного истечения радиоактивных газов, пробы подпочвенного газа должны содержать следы радиоактивных благородных газов. При инспекции скрытого взрыва, проведенного в горизонтальных выработках горного массива, следует ориентироваться на анализ проб атмосферного воздуха в инспектируемом районе

Radiation control and monitoring in the Republic of Belarus: results and tendencies

S54

Zhukova, O.M.; Germenchuk, M.G. (Respublikanskij Tsentr po Hidrometeorologii, Kontrolyu Radioaktivnogo Zagryazneniya i Monitoringu Okruzhayushchej Sredy, Minsk (BY))

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Radiatsionnyj kontrol' i monitoring v Respublike Belarus': rezul'taty i napravleniya razvitiya

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BELARUS; CONTAMINATION; CONTROLLED AREAS; NUCLEAR POWER PLANTS;
CONTROL SYSTEMS; RADIATION MONITORING; CESIUM 137; AIR; SURFACE
WATERS; SOILS; CONTAMINATION REGULATIONS

The radiation monitoring system of air (79 stations), surface waters (6 stations) and soils (49 stations) of the Republic of Belarus is under consideration. There are 2193 settlements in contaminated areas. Over a period of 1986-2015 the square of the territory contaminated by cesium-137 with levels more than 37 kBq/m² decreased 1,7 times and is equal 13,4% of the total territory of the republic, as of 2016. In the areas of the Chernobyl, Smolensk, Rovno and Ignalina NPPs influence 4 automated radiation monitoring systems are in operation. The program of radiation monitoring system in the area around the Belarus NPP which contains the list of stations and regulations of observations, the list of controlled radioisotopes is developed

Рассмотрена система радиационного мониторинга атмосферного воздуха (79 пунктов), поверхностных вод (6 пунктов) и почв (49 пунктов) в Республике Беларусь. В зонах радиоактивного загрязнения находится 2193 населенных пункта. За период с 1986 по 2015 год площадь территории загрязнения цезием-137 с уровнями более 37 кБк/м² уменьшилась в 1,7 раз и по состоянию на 2016 год составила 13,4% от общей площади республики. В зонах влияния Чернобыльской, Смоленской, Ровенской и Игналинской АЭС функционируют 4 автоматизированные системы радиационного контроля. Разработана программа радиационного мониторинга в районе расположения Белорусской АЭС, которая включает перечень пунктов и регламент наблюдений, перечень контролируемых радионуклидов

**Computer complexes #Left-Pointing Double Angle Quotation
Mark#CHROMOSOMA#Right-Pointing Double Angle Quotation Mark#, #Left-
Pointing Double Angle Quotation Mark#CHROMOSOMA-FISH#Right-
Pointing Double Angle Quotation Mark# for mass cytogenetic examinations of
population in after-Chernobyl period
S63**

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Komp'yuternye komplekсы #Left-Pointing Double Angle Quotation
Mark#KhROMOSOMA#Right-Pointing Double Angle Quotation Mark#, #Left-Pointing Double
Angle Quotation Mark#KhROMOSOMA-FISH#Right-Pointing Double Angle Quotation Mark#

dlya massovykh tsitogeneticheskikh obsledovaniy naseleniya v postchernobyl'skij period
p. 153-158

(RU)

2 refs., 4 figs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po
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CONTAMINATION; CHROMOSOMES; CHROMOSOMAL ABERRATIONS; CYTOLOGY;
GENETICS; NEOPLASMS; DATA PROCESSING; COMPUTER CODES; HUMAN
POPULATIONS

Computerized cytogenetic complexes #Left-Pointing Double Angle Quotation
Mark#CHROMOSOMA#Right-Pointing Double Angle Quotation Mark# and #Left-Pointing
Double Angle Quotation Mark#CHROMOSOMA-FISH#Right-Pointing Double Angle
Quotation Mark# are developed for mass cytogenetic examination of population affected by the
Chernobyl accident. These complexes are multifunctional software and hardware platform for
automated analysis of biological specimens with the use of microscopes and computer 2D
scanner for scanning and digitization of biological specimens on glasses and chips. Computer
algorithms provide for express detection of initial stages of chromosomal diseases, automation of
morphometric parameters measurements and classification of the objects of histologic and
cytologic biopreparations, automated analysis of numerical and structural chromosomal
aberrations, construction of man's karyotype

Компьютерные цитогенетические комплексы #Left-Pointing Double Angle Quotation
Mark#ХРОМОСОМА#Right-Pointing Double Angle Quotation Mark# и #Left-Pointing Double
Angle Quotation Mark#ХРОМОСОМА-FISH#Right-Pointing Double Angle Quotation Mark#
разработаны для массовых цитогенетических обследований населения, пострадавшего от
Чернобыльской аварии. Эти комплексы представляют собой многофункциональную
программно-аппаратную платформу для автоматизированного анализа биологических
препаратов с использованием микроскопов и компьютерного двухкоординатного сканера
для сканирования и оцифровки биопрепаратов на стеклах и чипах. Компьютерные
алгоритмы приборов обеспечивают оперативное выявление начальных стадий
хромосомных болезней, автоматизацию измерений морфометрических параметров и
классификации объектов гистологических и цитологических биопрепаратов,
автоматизированный анализ числовых и структурных aberrаций хромосом, построение
кариотипа человека

Technogenic and natural tritium in atmospheric precipitations on the territory of Russia

S54

Katkova, M.N.; Gnilomedov, V.D.; Shesterikova, E.M.; Tarasenko, A.O.; Filatova, A.N. (FGBU

#Left-Pointing Double Angle Quotation Mark#NPO#Left-Pointing Double Angle Quotation Mark#Tajfun#Right-Pointing Double Angle Quotation Mark#, Obninsk (RU))

Radioactivity after the nuclear explosions and accidents: effects and ways to overcome

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Radioaktivnost' posle yadernykh vzryvov i avarij: posledstviya i puti preodoleniya

Tritij tekhnogenyj i prirodnyj v atmosferykh osadkakh na territorii Rossii

p. 159-163

(RU)

6 refs., 3 figs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po

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416 p.

**RUSSIAN FEDERATION; TRITIUM; EARTH ATMOSPHERE; NUCLEAR EXPLOSIONS;
DISTRIBUTION; NUCLEAR FACILITIES; CONTAMINATION**

Systematic measurements of tritium content in atmospheric precipitations on the Russian Federation territory has been conducted from 1969. The analysis of obtained data allows to make the following conclusions. The main quantity of technogenic tritium was entered the atmosphere during the period of atmospheric nuclear tests up to 1980. During the test period the stratospheric reservoir of technogenic tritium have been formed, now it have not detectable effect on tritium content in precipitations. Effect of NPPs, radiochemical plants, and other objects using nuclear materials is negligible. In warm season with increase of vertical circulation in atmosphere tritium income from stratosphere is intensified. At the present time tritium in atmospheric precipitations has generally the natural origin

Систематические измерения содержания трития в атмосферных осадках на территории России проводятся с 1969 г. Анализ полученных данных позволил сделать следующие выводы. Основное количество техногенного трития поступило в атмосферу в период испытания ядерного оружия в атмосфере, продолжавшегося до 1980 г. В период испытаний сформировался стратосферный резервуар техногенного трития, который в настоящее время не оказывает заметного влияния на содержание трития в осадках. Влияние АЭС, радиохимических предприятий и других объектов, использующих ядерные материалы, незначительно. В теплый период года по мере увеличения вертикальной циркуляции в атмосфере усиливается поступление природного трития из стратосферы. В настоящее время тритий, наблюдаемый в атмосферных осадках, имеет в основном природное происхождение

Meteorologic factors of environment contamination: climate characteristics of inversions in the lower atmosphere for the regions of the Chernobyl NPP and

Fukushima NPP

S54

Kozlova, L.F.; Khokhlova, A.V. (FGBU #Left-Pointing Double Angle Quotation Mark#VNIIGMI-MTsD#Right-Pointing Double Angle Quotation Mark#, Obninsk (RU))

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Meteorologicheskie faktory zagryazneniya okruzhayushchej sredy: klimaticheskie kharakteristiki
inversij v nizhnem sloe atmosfery dlya rajonov Chernobyl'skoj AEhS i AEhS #Left-Pointing
Double Angle Quotation Mark#Fukushima#Right-Pointing Double Angle Quotation Mark#
p. 164-169

(RU)

2 refs., 3 figs., 5 tabs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po
Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

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**CHERNOBYLSK-4 REACTOR; FUKUSHIMA DAIICHI NUCLEAR POWER STATION;
CONTAMINATION; TEMPERATURE INVERSIONS; CLIMATES**

Climate characteristics of temperature inversions near the Chernobyl and Fukushima NPPs in
2011-2015 are analyzed. The assessment of surface and raised inversions frequency by months,
seasons and years are obtained. The calculations of medium power, intensity of surface and
raised inversions, medium altitude of raised inversions are carried out

Проанализированы климатические характеристики температурных инверсий в
окрестностях Чернобыльской АЭС и АЭС #Left-Pointing Double Angle Quotation
Mark#Fukushima#Right-Pointing Double Angle Quotation Mark# в 2011-2015 годах. Получены
оценки повторяемости приземных и приподнятых инверсий по месяцам, сезонам и за год.
Выполнены расчеты средней мощности, интенсивности приземных и приподнятых
инверсий, средней высоты нижней границы приподнятых инверсий

Behaviour of model particles of local precipitations of surface nuclear explosion in food chain and digestive tract of farm animals

S63

*Koz'min, G.V.; Epimakhov, V.G.; Sanzharova, N.I. (FGBNU #Left-Pointing Double Angle
Quotation Mark#Vserossijskij Nauchno-Issledovatel'skij Inst. Radiologii i Agroekologii#Right-
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Povedenie model'nykh chastits lokal'nykh vypadenij nazemnogo yadernogo vzryva v pishchevoj
tsepochke i pishchevaritel'nom trakte sel'skokhozyajstvennykh zhivotnykh
p. 170-201

(RU)

51 refs., 10 figs., 7 tabs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po
Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

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CATTLE; SHEEP; CONTAMINATION; FALLOUT; PARTICLES; GASTROINTESTINAL
TRACT; DISTRIBUTION; FOOD CHAINS; ABSORBED RADIATION DOSES;
MATHEMATICAL MODELS; LOCAL RADIATION EFFECTS; DOSE RATES

The behaviour regularities of radioactive particles - simulators of nuclear surface explosion local fall outs in food chain and gastrointestinal tract (GIT) of farm animals are analyzed. The results show that there is a large difference in transport regularities of radioactive silicate particles and radioactive solutions in GIT. At intake of young fission products high concentrations of radionuclides in GIT content deal with sorption and concentrating of radionuclides on food particles and observe in third stomach, blind gut, terminals of middle and bung guts. Transport regularities of fused radioactive particles depend on digestive apparatus mobility, content consistency and morphological peculiarities of mucosa, which work towards transport slowing and storage of such particles in the part of sheep GIT with minimal dry substance content - abomasum

Проанализированы закономерности поведения радиоактивных частиц - имитаторов локальных выпадений наземного ядерного взрыва в пищевой цепочке и желудочно-кишечном тракте (ЖКТ) сельскохозяйственных животных. Результаты свидетельствуют о существенном отличии закономерностей транспорта в ЖКТ радиоактивных силикатных частиц от транспорта радиоактивных растворов. При поступлении молодых продуктов ядерного деления наиболее высокие концентрации радионуклидов в содержимом ЖКТ связаны с сорбцией и концентрированием радионуклидов на частицах корма и наблюдаются в книжке, слепой кишке, конечной части ободочной кишки и прямой кишки. Закономерности транспорта оплавленных радиоактивных частиц зависят от двигательной функции аппарата пищеварения, консистенции содержимого и морфологических особенностей строения слизистой, способствующих замедлению транспорта и депонированию таких частиц в отделе ЖКТ овец с минимальным содержанием сухого вещества - сычуге

Fukushima and Chernobyl: common and different in radiocesium behaviour
S54

Konoplev, A.V. (*Inst. Radioaktivnosti Okruzhayushchej Sredy, Univ. Fukusimy, Fukusima (JP)*)

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Fukusima i Chernobyl': obshchee i razlicnoe v povedenii radiotseziya

p. 202-218

(RU)

35 refs., 1 fig., 3 tabs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po

Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

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416 p.

FUKUSHIMA DAIICHI NUCLEAR POWER STATION; CHERNOBYLSK-4 REACTOR;
REACTOR ACCIDENTS; ENVIRONMENT; SURFACE WATERS; SOILS; CESIUM 137;
RADIONUCLIDE MIGRATION; NATURAL ATTENUATION; ATMOSPHERIC
PRECIPITATIONS; CLIMATES; COMPARATIVE EVALUATIONS

Comparative analysis of radiocesium (RC) behaviour of the Chernobyl and Fukushima origin in the soil-water system is carried out. It is shown that distribution coefficients of RC in water of Fukushima are 1-2 orders higher than that for rivers of the Chernobyl zone, and standardized coefficients of $\langle \text{Left-Pointing Double Angle Quotation Mark} \text{liquid} \text{Right-Pointing Double Angle Quotation Mark} \rangle$ wash in Chernobyl are 1-2 orders higher in comparison with Fukushima drainage basin. Standardized coefficients of $\langle \text{Left-Pointing Double Angle Quotation Mark} \text{solid} \text{Right-Pointing Double Angle Quotation Mark} \rangle$ wash of RC in drainage basin of Chernobyl are comparable to corresponding values of Fukushima in value. It is shown that in the same period after the accident contaminated territories near Fukushima-1 NPP have higher self-purification capacity, and natural attenuation of environment contamination levels has higher rate than in the Chernobyl zone. This is because of the higher annual precipitation norm, higher yearly average temperature, as well as extreme floods during the period of typhoons. Geomorphologic conditions and predominant types of soils and initial forms of RC in fallouts have substantial significance

Проведен сравнительный анализ поведения радиоцезия (РЦ) Чернобыльского и Фукусимского происхождения в системе почва-вода. Показано, что коэффициент распределения РЦ в водах Фукусимы на 1-2 порядка выше соответствующих значений для рек в Чернобыльской зоне, а нормированные коэффициенты $\langle \text{Left-Pointing Double Angle Quotation Mark} \text{жидкого} \text{Right-Pointing Double Angle Quotation Mark} \rangle$ смыва в Чернобыле на 1-2 порядка больше по сравнению с водосборами Фукусимы. Нормированные коэффициенты $\langle \text{Left-Pointing Double Angle Quotation Mark} \text{твердого} \text{Right-Pointing Double Angle Quotation Mark} \rangle$ смыва РЦ с водосборов Чернобыля сравнимы с соответствующими величинами для Фукусимы. Показано, что через одинаковое время после аварии загрязненные территории в районе АЭС Фукусима-1 характеризуются более высокой самоочищающей способностью, и естественное снижение уровней загрязнения

окружающей среды происходит там быстрее по сравнению с зоной аварии на ЧАЭС. Это обусловлено большей годовой нормой осадков, более высокой среднегодовой температурой, а также экстремальными паводками в период тайфунов. Существенное значение имеют геоморфологические условия и преобладающие виды почв и начальные формы РЦ в выпадениях

Organization of acquisition, processing and representation of radiation monitoring data on the territory of the Russian Federation

S54

Kornejchuk, N.A.; Satyr', S.V.; Kalmykova, O.V.; Mukhalev, V.N. (FGBU #Left-Pointing Double Angle Quotation Mark#NPO#Left-Pointing Double Angle Quotation Mark#Tajfun#Right-Pointing Double Angle Quotation Mark#, Obninsk (RU))

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Organizatsiya sbora, obrabotki i predstavleniya dannykh radiatsionnogo monitoringa na territorii RF

p. 219-222

(RU)

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

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416 p.

RUSSIAN FEDERATION; RADIATION ACCIDENTS; RADIATION MONITORING; ON-LINE CONTROL SYSTEMS; COMPUTER CODES

Unified state automated system for radiation situation monitoring on the territory of the Russian Federation (USASRM) unites departmental monitoring systems of Roshydromet and Rosatom and 18 territorial radiation situation monitoring systems. The main and 8 regional information and analysis centers, as well as 6 data centers are also part of USASRM. The organization of interaction of USASRM subdivisions are under consideration. There is a special software to control data acquisition, it allows to visualize data, autoupdate on-line, give out colour and audio alarm when gamma-radiation dose rate exceeds high and extremely high levels. In 2011 the Fukushima-1 NPP accident showed the ability of USASRM to conduct a mission of emergency response at radiation accidents on the territory of the Russian Federation and neighboring states

Единая государственная автоматизированная система мониторинга радиационной обстановки на территории Российской Федерации (ЕГАСМРО) объединяет ведомственные системы мониторинга Росгидромета и Росатома, а также 18 территориальных систем контроля радиационной обстановки. В ЕГАСМРО входят главный и 8 региональных информационно-аналитических центров, а также 6 центров

сбора и обработки данных. Рассмотрена организация взаимодействия подразделений ЕГАСМРО. Контроль за процессом сбора данных осуществляется с использованием специального программного обеспечения, с помощью которого обеспечивается визуализация данных, автоматическое обновление в режиме реального времени, цветографическая и звуковая сигнализация при превышении высокого и экстремально высокого уровней мощности дозы гамма-излучения. В 2011 г. авария на АЭС Фукусима-1 показала способность ЕГАСМРО выполнять задачу аварийного реагирования при возникновении чрезвычайной ситуации радиационного характера на территории Российской Федерации или сопредельных государств

State-of-the-art software and hardware tools for solving operative radioactive monitoring problems

S54

Kosykh, V.S.; Sarychev, S.A. (FGBU #Left-Pointing Double Angle Quotation Mark#NPO#Left-Pointing Double Angle Quotation Mark#Tajfun#Right-Pointing Double Angle Quotation Mark#, Obninsk (RU))

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Sovremennye programmno-tehnicheskie sredstva dlya resheniya operativnykh zadach
radioaktivnogo monitoringa

p. 223-230

(RU)

5 figs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po
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416 p.

**RUSSIAN FEDERATION; RADIATION MONITORING; ON-LINE SYSTEMS;
RADIATION ACCIDENTS**

The work organization of the Unified state automated system for radiation situation monitoring on the territory of the Russian Federation in normal mode and during the Fukushima accident, then the system assured the reception well-timed assessments of the accident consequences on the territory of the Russian Federation. The operation and possibilities of automatic weather stations, automobile- and aircraft-based laboratories of radiation monitoring are considered

Рассмотрена организация работы Единой государственной автоматизированной системы мониторинга радиационной обстановки на территории Российской Федерации в обычном режиме и во время аварии на АЭС #Left-Pointing Double Angle Quotation Mark#Фукусима#Right-Pointing Double Angle Quotation Mark#, тогда система обеспечила

получение своевременных оценок последствий аварии на территории Российской Федерации. Обсуждается работа и возможности автоматических метеостанций, автомобильных и воздушных лабораторий радиационной разведки

Comparative analysis of doses on aquatic biota in water bodies exposed to radioactive contamination

S54

Kryshev, A.I.; Sazykina, T.G. (FGBU #Left-Pointing Double Angle Quotation Mark#NPO#Left-Pointing Double Angle Quotation Mark#Tajfun#Right-Pointing Double Angle Quotation Mark#, Obninsk (RU))

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Srvnitel'nyj analiz doz na vodnyu biotu v vodoemakh, podvergshikhsya radioaktivnomu
zagryazneniyu

p. 231-245

(RU)

29 refs., 2 figs., 2 tabs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po

Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

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**COOLING PONDS; TECHA RIVER; LAKES; RADIATION ACCIDENTS; RADIATION
DOSES; DOSE RATES; AQUATIC ECOSYSTEMS; BIOLOGICAL RADIATION EFFECTS;
COMPARATIVE EVALUATIONS; DOSE-RESPONSE RELATIONSHIPS; FORECASTING**

The comparative evaluations of dose rates for reference biota species of the Chernobyl NPP cooling pond, Uruskul' lake (East Urals radioactive trace), Techa and Yenisei rivers are given. Irradiation levels of biotas in early period after radiation accidents were rather high to induce biological radiation affects in reproductive system of fishes (Chernobyl NPP cooling pond) and reduction of fish lifetime (Uruskul' lake). The most sensitive component of freshwater ecosystem was benthic tropic chain. Present irradiation levels of biota in all water objects considered are lower than safe level (1 mGy/day). Dose-response relationships for chronic irradiation of biota can be used for evaluating permissible specific activities of radionuclides in water, bottom deposits and biota, when radiation safety of the most sensitive organisms and populations are assured

Приведены сравнительные оценки мощностей доз для референтных видов биоты водоема-охладителя Чернобыльской АЭС, оз. Урускуль (Восточно-Уральский радиоактивный след), рек Теча и Енисей. Уровни облучения биоты в ранний период после радиационных аварий были достаточно высоки, чтобы вызвать радиобиологические эффекты в

воспроизводительной системе рыб (водоем-охладитель ЧАЭС) и сокращение продолжительности жизни рыб (оз. Урускуль). Наиболее чувствительной компонентой пресноводной экосистемы была бентосная трофическая цепочка. Современные уровни облучения биоты во всех рассмотренных водных объектах ниже безопасного уровня 1мГр/сут. Зависимости мощность дозы - эффекты для хронического облучения биоты могут быть использованы для оценок допустимых удельных активностей радионуклидов в воде, донных отложениях и биоте, при которых обеспечивается радиационная безопасность наиболее чувствительных организмов и популяций

Ecological risks of the Chernobyl accident

S54

Kryshev, I.I.; Sazykina, T.G.; Kryshev, A.I. (FGBU #Left-Pointing Double Angle Quotation Mark#NPO#Left-Pointing Double Angle Quotation Mark#Tajfun#Right-Pointing Double Angle Quotation Mark#, Obninsk (RU))

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Ehkologicheskie riski Chernobyl'skoj avarii

p. 246-266

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36 refs., 4 figs., 3 tabs.

Shershakov, V.M. (ed.)

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Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

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CHERNOBYLSK-4 REACTOR; REACTOR ACCIDENTS; CONTAMINATION; ENVIRONMENT; RISK ASSESSMENT; ECOSYSTEMS; RADIONUCLIDE MIGRATION; NATURAL ATTENUATION; TIME DEPENDENCE; BIOLOGICAL RADIATION EFFECTS; CONTROLLED AREAS

On the base of risk analysis methodology ecological aspects of the Chernobyl accident related with radioactive contamination of environment and radiation effects on the objects of wildlife are under consideration. Some character time scales in forming the situation on the territory of the Chernobyl accident trace are marked. During the first stage (near a month) atmospheric transfer and radionuclide precipitation on the components of terrestrial and aquatic environments took place. On the second stage (years) redistribution of radionuclides among components of ecosystems took place. On the third stage (decades) step-by-step decrease of natural objects contamination due to radioactive decay is observed. It is pointed out that in whole area of the territories with increased levels of contamination are considerably more than the pieces where radiation injuring effects of the Chernobyl accident on biota are manifested

На основе методологии анализа риска рассматриваются экологические аспекты Чернобыльской аварии, связанные с радиоактивным загрязнением окружающей среды и радиационным воздействием на объекты живой природы. Выделено несколько характерных временных масштабов в формировании обстановки на территории аварийного чернобыльского следа. В течение первого этапа (примерно месяц) происходил атмосферный перенос и осаждение радионуклидов на компоненты наземной и водной среды. На втором этапе (годы) происходило перераспределение радионуклидов между компонентами экосистем. На третьем этапе (десятилетия) происходит постепенное снижение загрязнения объектов окружающей среды в основном в результате радиоактивного распада. Отмечается, что в целом площади территорий с повышенными уровнями радиоактивного загрязнения существенно больше участков, на которых проявились повреждающие эффекты радиационного воздействия Чернобыльской аварии на биоту

Multiscale of Cs-137 contamination levels of Bryansk Region landscapes (according to airborne gamma survey data)

S54

Linnik, V.G. (GEOKhI RAN, Moscow (RU); Geograficheskij Fakul'tet MGU, Moscow (RU)); Sokolov, A.V. (GEOKhI RAN, Moscow (RU); IPI RAN, Moscow (RU)); Sokolov, P.V. (GEOKhI RAN, Moscow (RU))

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Mnogomasshtabnost' urovnej zagryazneniya Cs-137 landshaftov Bryanskoj oblasti (po dannym aehrogammas"emki)

p. 267-297

(RU)

56 refs., 15 figs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

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CHERNOBYLSK-4 REACTOR; REACTOR ACCIDENTS; CONTAMINATION; ENVIRONMENT; RUSSIAN FEDERATION; RADIOACTIVE AEROSOLS; ATMOSPHERIC PRECIPITATIONS; SOILS; SURFACE WATERS; SPATIAL DISTRIBUTION

Multiscale of cesium-137 contamination as a result of the Chernobyl accident is investigated, it is manifested in spatial nonuniformity of contamination on different scale levels. Genesis of radionuclide contamination field is determined by dynamically developing synoptical situation (wind velocity and direction, precipitations, etc.), different intensity of radioactive aerosols input

from emergency reactor during the first decade after the accident, isotope fractionation on different distances from the Chernobyl-4 reactor. The field of contamination by *Left-Pointing Double Angle Quotation Mark*Chernobyl*Right-Pointing Double Angle Quotation Mark* radionuclides one can considered as a result of superposition of different-scale levels of radioactive aerosols deposition. The various roughness of geological substrate is also one of the factors forming density nonuniform fields of radioactive contamination. Transformation of radionuclide contamination field formed as a result of radioactive aerosols deposition in April-May, 1986, in the following years occurred under the influence of landscape factors. Using the Kostitsa river basin as an example the processes and the scale of lateral transport of cesium-137 and its deposition on biogeochemical barriers are investigated

Исследована многомасштабность радиоактивного загрязнения цезием-137 в результате аварии на ЧАЭС, которая проявилась в пространственной неоднородности загрязнения на разных масштабных уровнях. Генезис поля загрязнения радионуклидами определялся динамично менявшейся синоптической обстановкой (скорость и направление ветра, осадки и др.), различной интенсивностью поступления радиоактивных аэрозолей из аварийного блока в первую декаду после аварии, фракционированием изотопов на различных расстояниях от 4-го блока ЧАЭС. Поле загрязнения *Left-Pointing Double Angle Quotation Mark*чернобыльскими*Right-Pointing Double Angle Quotation Mark* радионуклидами можно рассматривать как результат суперпозиции разномасштабных уровней осаждения радиоактивных аэрозолей. Различная шероховатость подстилающей поверхности также является одним из факторов образования неоднородных по плотности полей радиоактивного загрязнения. Трансформация исходного поля радионуклидного загрязнения, сформировавшегося как результат осаждения радиоактивных аэрозолей в апреле-мае 1986 г., в последующие годы происходила в результате влияния ландшафтных факторов. На примере бассейна р. Костица исследованы процессы и масштабы латерального переноса цезия-137 и его осаждения на биогеохимических барьерах

Retrospective radiation-hygienic assessment of cesium-137 intake with feeding in the organisms of the Altai Territory habitants

S61

*Meshkov, N.A.; Val'tseva, E.A. (Federal'noe Gosudarstvennoe Byudzhetnoe Uchrezhdenie Left-Pointing Double Angle Quotation Mark*Nauchno-Issledovatel'skij Inst. Ehkologii Cheloveka i Gigieny Okruzhayushchej Sredy im. A.N. Sysina*Right-Pointing Double Angle Quotation Mark* Ministerstva Zdravookhraneniya Rossijskoj Federatsii, Moscow (RU))

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Retrospektivnaya radiatsionno-gigienicheskaya otsenka postupleniya tseziya-137 s ratsionom pitaniya v organizm zhitelej Altajskogo kraja

p. 298-304

(RU)

6 refs., 6 figs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

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SEMPALATINSK TEST SITE; NUCLEAR EXPLOSIONS; CONTAMINATION;
ENVIRONMENT; FOOD CHAINS; HUMAN POPULATIONS; CESIUM 137;
RADIOACTIVITY; MEAT; FOOD; TIME DEPENDENCE; INTAKE

Radioactive precipitations as the result of atmospheric nuclear tests on the Semipalatinsk test site turned to local soil contamination by cesium-137 on the territory of the Altai Territory and Gorny Altai. The distribution of long-lived radioisotopes, cesium-137 in particular, in the main food stuffs for local population is investigated. The retrospective analysis of cesium-137 specific activity in food products produced on these territories is carried out. It is ascertained that cesium-137 in meat has the general contribution to intake with food into organisms of adult population

Радиоактивные выпадения вследствие атмосферных ядерных испытаний на Семипалатинском полигоне привели к локальным загрязнениям почвенного покрова цезием-137 на территории Алтайского края и Горного Алтая. Изучено распределение долгоживущих радионуклидов, в частности цезия-137, в основных пищевых продуктах, входящих в рацион питания местного населения. Выполнен ретроспективный анализ удельной активности цезия-137 в пищевых продуктах, произведенных на этих территориях. Установлено, что основной вклад при поступлении в организм взрослого населения с рационом питания вносит цезий-137, содержащийся в мясе

On forecasting of rivers contamination as a result of Chernobyl NPP accident S54

Novitskij, M.A. (FGBU #Left-Pointing Double Angle Quotation Mark#NPO#Left-Pointing Double Angle Quotation Mark#Tajfun#Right-Pointing Double Angle Quotation Mark#, Obninsk (RU))

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O prognozirovanii zagryazneniya rek vsledstvie avarii na Chernobyl'skoj atomnoj

ehlektrostantsii

p. 305-311

(RU)

3 refs.

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Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po

Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

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CHERNOBYLSK-4 REACTOR; REACTOR ACCIDENT SIMULATION;
CONTAMINATION; RIVERS; FORECASTING; RADIOECOLOGICAL
CONCENTRATION; RAIN; RUNOFF; FLOODS

Contamination of rivers on the territory effected by the Chernobyl accident is under consideration. On the base of analysis field and laboratory experiments data understanding about formation of long-lived radionuclides concentration in rain and snow melt runoff was elaborated. The correctness of mathematical model used for forecasting radiation situation on rivers was confirmed by the data of rivers contamination levels in spring 1987

Рассмотрено загрязнение рек, протекающих на территории, загрязненной после аварии на Чернобыльской АЭС. На основании анализа данных полевых и лабораторных экспериментов были выработаны представления о формировании концентрации долгоживущих радионуклидов в поверхностном стоке дождевых и талых вод. Правильность примененной математической модели для прогноза радиационной обстановки на реках загрязненной зоны была подтверждена данными наблюдения за уровнями загрязнения рек весной 1987 года

Aerosol-carrier of #Left-Pointing Double Angle Quotation Mark#fresh#Right-Pointing Double Angle Quotation Mark# radionuclides on the Chernobyl NPP site after two weeks from Fukushima-1 NPP accident

S54

Ogorodnikov, B.I. (GNTs #Left-Pointing Double Angle Quotation Mark#Fiziko-Khimicheskij Inst. im. L.Ya. Karpova#Right-Pointing Double Angle Quotation Mark#, Moscow (RU); Inst. Problem Bezopasnosti AEhS NAN Ukrainy, Chernobyl' (UA)); Kalinovskij, A.K. (Inst. Problem Bezopasnosti AEhS NAN Ukrainy, Chernobyl' (UA))

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Aehrozoli-nositeli #Left-Pointing Double Angle Quotation Mark#svezhikh#Right-Pointing
Double Angle Quotation Mark# radionuklidov na promploshchadke Chernobyl'skoj AEhS
cherez dve nedeli posle avarii na AEhS #Left-Pointing Double Angle Quotation
Mark#Fukusima-1#Right-Pointing Double Angle Quotation Mark#

p. 312-328

(RU)

18 refs., 13 figs., 1 tab.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po
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FUKUSHIMA DAIICHI NUCLEAR POWER STATION; REACTOR ACCIDENTS;
CONTAMINATION; EARTH ATMOSPHERE; WIND; RADIOACTIVE AEROSOLS;
PARTICLE SIZE; RADIOISOTOPES

Discharge in air of radioactive substances (^{131}I , ^{132}I , ^{132}Te , ^{134}Cs , ^{137}Cs , ^{137}Cs) from reactor wreck of Fukushima-1 NPP began on March, 12, 2011. Because of prevailing winds the main transfer of accidental gases took place to the east. In North and Central Europe the first peak of radioactive substances was detected between 28 and 30 of March, the second was 3-5 days later. On the Chernobyl NPP site the aerosols of Fukushima genesis were also detected. It is established that their carriers were submicron particles of near 0.5 μm diameter, which are stable in atmosphere and run to great distances. Detected in the North America and Europe contents of radioactive substances were low and not hazard for population

Выбросы в атмосферу радиоактивных веществ (^{131}I , ^{132}I , ^{132}Te , ^{134}Cs , ^{137}Cs , ^{137}Cs) из разрушенных реакторов АЭС Фукусима-1 начались 12 марта 2011 г. Вследствие господствующих ветров основной перенос выброшенных при аварии веществ происходил в восточном направлении. В Северной и Центральной Европе первый пик радиоактивных веществ наблюдался между 28 и 30 марта, второй - на 3-5 сут позже. На промплощадке Чернобыльской АЭС также были обнаружены аэрозоли фукусимского генеза. Установлено, что их носителями являлись субмикронные частицы диаметром около 0,5 мкм, которые устойчивы в атмосфере и распространяются на большие расстояния. Зарегистрированные в Северной Америке и Европе содержания в воздухе радиоактивных веществ были низкие и не представляли большой опасности для населения

Radiation-ecological monitoring in the area of Ruppur NPP in the People's Republic of Bangladesh

S54

Panov, A.V.; Sanzharova, N.I.; Tsygvintsev, P.N.; Isamov, N.N.; Kurbakov, D.N. (FGBNU VNIIRAEh, Obninsk (RU))

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Radiatsionno-ehkologicheskij monitoring v rajone raspolozheniya AEhS Ruppur v Narodnoj
Respublike Bangladesh

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2 figs.

Shershakov, V.M. (ed.)

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BANGLADESH; NUCLEAR POWER PLANTS; REACTOR SITES; CONTROLLED AREAS; CONTAMINATION; ENVIRONMENT; RADIATION MONITORING; EARTH ATMOSPHERE; SOILS; SURFACE WATERS; ECOSYSTEMS

The program of radiation-ecological monitoring of the region of #Left-Pointing Double Angle Quotation Mark#Ruppur#Right-Pointing Double Angle Quotation Mark# NPP is developed, survey points are chosen and examined; the monitoring objects, list of observing parameters, regulation of observation are determined as well as the techniques of observation and regulatory and engineering provisions. On the base of radiation-ecological monitoring of atmospheric air, aquatic and terrestrial ecosystems carried out in 30-km area of the NPP in 2014-2015 it is possible to make a conclusion that ecological situation in the region is good in whole. Formed monitoring network of atmosphere, aquatic and terrestrial ecosystems will allow to record the change of situation in 30-km NPP area and to reveal the effect of the NPP operation on ecology in the region

Разработана программа радиационно-экологического мониторинга региона АЭС #Left-Pointing Double Angle Quotation Mark#Руппур#Right-Pointing Double Angle Quotation Mark#, выбраны и обследованы пункты наблюдений; определены объекты мониторинга, перечень наблюдаемых параметров, регламент наблюдений, а также определены методы проведения наблюдений и нормативно-техническое обеспечение. На основе проведенного в 2014-2015 гг. радиационно-экологического мониторинга атмосферного воздуха, наземных и водных экосистем в 30-км зоне этой АЭС можно заключить, что экологическая обстановка в регионе в целом благополучная. Заложенная сеть мониторинга атмосферы, наземных и водных экосистем позволит регистрировать изменение ситуации в 30-км зоны АЭС и выявлять влияние работы АЭС на экологическую обстановку в регионе

Analysis of cesium-137 vertical distribution in the profile of plowed chernozems at different schemes of their assaying

S54

Paramonova, T.A.; Komissarova, O.L. (Fakul'tet Pochvovedeniya MGU im. M.V. Lomonosova, Moscow (RU)); Belyaev, V.R.; Ivanov, M.M. (Geograficheskij Fakul'tet MGU im. M.V. Lomonosova, Moscow (RU))

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Analiz vertikal'nogo raspredeleniya tseziya-137 v profile pakhotnykh chernozemov pri razlichnykh skhemakh ikh oprobvaniya

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(RU)

9 refs., 3 figs., 1 tab.

Shershakov, V.M. (ed.)

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CHERNOBYLSK-4 REACTOR; REACTOR ACCIDENTS; SURFACE CONTAMINATION; CESIUM 137; DISTRIBUTION; RADIOECOLOGY; AGRICULTURE; MONITORING

In 2011-2015 the assessment of profile cesium-137 distribution in agrocenosis of chernozem zone on the territory of the Plavsk radioactive spot in Tula region formed after the Chernobyl accident has been carried out. It is shown that up until now non-uniformity of cesium-137 vertical distribution over the plowed chernozems profile may be occurred, it should be taken into account at radioecological survey of post-Chernobyl landscapes. For correct evaluation of radioecological state of plowed soils their systematic monitoring on the base of preliminary analysis of cesium-137 distribution and also with the account of agrotechnical peculiarities of various crops cultivation is recommended. On the Plavsk radioactive spot territory the most adequate assessments of cesium-137 stores in plowed chernozems one can obtain on the base of assaying the upper 30-cm soil depth, including not only current topsoil, but also old-arable horizon formed by deep rehabilitation plowing

В 2011-2015 гг. проведена оценка профильного распределения цезия-137 в агроценозах черноземной зоны на территории Плавского радиоактивного пятна Тульской области, образовавшегося после аварии на Чернобыльской АЭС. Показано, что вплоть до настоящего времени может отмечаться неоднородность вертикального распределения цезия-137 по профилю пахотных черноземов, что следует учитывать при радиоэкологическом обследовании пост-чернобыльских ландшафтов. Для корректной оценки радиоэкологического состояния пахотных почв рекомендуется проведение их систематического мониторинга на основе предварительного профильного анализа распределения цезия-137, а также с учетом агротехнических особенностей возделывания различных культур. На территории Плавского радиоактивного пятна наиболее адекватные оценки запасов цезия-137 в пахотных черноземах можно получить на основе опробования верхней 30-см толщи почв, включающей не только современный пахотный горизонт, но и созданный глубокой реабилитационной вспашкой старопашотный горизонт

Forest management in the areas of radioactive contamination: problems of getting round to normal functioning

S54

Razdajvodin, A.N.; Maradudin, I.I.; Romashkin, D.Yu.; Radin, A.I. (FBU Vserossijskij NII Lesovodstva i Mekhanizatsii Lesnogo Khozyajstva, Pushkino (RU))

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Lesnoe khozyajstvo v zonakh radioaktivnogo zagryazneniya: problemy vozvrashcheniya k normal'noj zhiznedeyatel'nosti

p. 347-353

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5 refs., 2 figs., 3 tabs.

Shershakov, V.M. (ed.)

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RUSSIAN FEDERATION; FORESTS; CONTAMINATION; SOILS; CHERNOBYLSK-4 REACTOR; REACTOR ACCIDENTS; DISTRIBUTION; ECOSYSTEMS; BIOLOGICAL RADIATION EFFECTS; CESIUM 137

As a result of the Chernobyl accident the part of forest fund on the territory of 15 subjects of the Russian Federation was contaminated by radionuclides (mostly by cesium-137). The consequences of contamination for forestry manifest as direct factors related with radiation: increased background radiation on forest compartments, forest soil contamination, excess of permissible limit of radionuclide content in forest resources; and indirect ones: decrease of biological and fire resistance of forests as a result of silvicultural practice limitation, necessity of additional charges for providing radiation protection of field works and radiation control of production, decrease of investment prospects in forest industry. Recommendations are given for accelerating solution of the problem of forestry reversion to normal functioning

В результате катастрофы на Чернобыльской АЭС часть лесного фонда на территории 15 субъектов Российской Федерации была загрязнена радионуклидами (в основном цезием-137). Последствия радиоактивного загрязнения для лесного хозяйства проявляются как в виде прямых факторов, связанных с радиацией: повышенный радиационный фон на лесных участках, радиоактивное загрязнение лесных почв, превышение допустимых уровней содержания радионуклидов в лесных ресурсах; так и косвенных: снижение биологической и противопожарной устойчивости насаждений в результате ограничений лесохозяйственных мероприятий, необходимость дополнительных затрат на обеспечение радиационной безопасности работ в лесу и радиационный контроль продукции, снижение инвестиционной привлекательности лесной отрасли. Даны рекомендации для ускорения решения проблемы возвращения лесного хозяйства на загрязненных территориях к нормальной жизнедеятельности

Small-size automated probe on the base of UAV with vertical takeoff and landing for meteorological support of environmentally hazardous objects

S54

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Malogabaritnyj avtomatizirovannyj zond na baze BPLA s vertikal'nym vzletom i posadkoj dlya
meteorologicheskogo obespecheniya ehkologicheski opasnykh ob"ektov

p. 354-358

(RU)

6 refs., 1 fig.

Shershakov, V.M. (ed.)

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AERIAL MONITORING; AIR POLLUTION MONITORING; METEOROLOGY;
AIRCRAFT; MEASURING INSTRUMENTS; TEMPERATURE DISTRIBUTION;
POLLUTANTS; AEROSOLS

Mock-up specimen of meteorological probe on the base of rotocopter is developed and its laboratory and field testings are carried out. The weight of the probe with 200 g airlift is near 1 kg. The probe is started up from the land, the further flight and landing occur in automatic mode. It allows to measure vertical distributions of meteorological parameters (temperature, pressure, moisture, wind velocity and direction) up to 1500 m. The given probe with equipment for measuring pollutant concentrations can be used for meteorological and ecological monitoring of environmentally hazardous objects (NPPs, chemical plants etc.)

Разработан макетный образец метеорологического зонда на базе летательного аппарата вертолетного типа, проведены его лабораторные и полевые испытания. Вес зонда грузоподъемностью 200 г - около 1 кг. Зонд запускается с земли, дальнейший полет и приземление происходит в автоматическом режиме. Он позволяет измерять вертикальные распределения метеорологических параметров (температура, давление, влажность, скорость и направление ветра) до высоты 1500 м. Данный зонд с установленной на борту аппаратурой для измерения концентрации загрязняющих примесей может быть использован для метеорологического и экологического мониторинга потенциально опасных объектов (АЭС, химические заводы и т.д.)

Monitoring of radon volume activity in the lowest atmospheric layer on the base of #gamma#-spectrometer SEG-017 readings: error analysis

S54

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Monitoring ob'emnoj aktivnosti radona v prizemnom atmosfernom sloe na osnove pokazanij
#gamma#-spektrometra SEhG-017: analiz pogreshnostej
p. 359-368

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416 p.

RADON 222; LEAD 214; BISMUTH 214; RADIATION MONITORING; GAMMA
SPECTROMETERS; ACCURACY; ERRORS; SIMULATION; APPROXIMATIONS

Consideration is given to simulation and error analysis of description of radon-222 and daughter products lead-214 and bismuth-214 volumetric activity dynamics on the base of recorded characteristics of #gamma#-radiation field, measured by SEG-017 spectrometer. The use of smooth approximation SvF and corresponding error analysis allow not only screen the errors and calculate volumetric activities dynamics but estimate the accuracy of measurements and simulation of the process on the whole. The possibility of radon-222, lead-214 and bismuth-214 on-line monitoring is shown

Рассмотрены моделирование и анализ погрешностей описания динамики объемной активности радона-222 и дочерних продуктов его распада свинца-214 и висмута-214 на основе регистрации характеристик поля #gamma#-излучения, измеряемого спектрометром СЭГ-017. Применение метода гладкой аппроксимации SvF и соответствующий анализ погрешностей позволяют не только отсеять ошибки измерений и рассчитать динамику объемных активностей, но и оценить точность измерений и моделирования процесса в целом. Показана возможность мониторинга радона-222, свинца-214 и висмута-214 в режиме реального времени

About dose coefficient dependence on distribution parameters of aerosol activity over particle diameters

S54

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Otnositel'no zavisimosti dozovogo koehffitsienta ot parametrov raspredeleniya aktivnosti

aehrozolya po diametram chastits

p. 369-371

(RU)

6 refs., 1 tab.

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Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

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RADIATION DOSES; RADIOACTIVE AEROSOLS; INTAKE; PARTICLE SIZE; DISTRIBUTION; APPROXIMATIONS; CORRELATIONS

With the view to ascertain the dependence of dose coefficient (dose per unit of intake) on standard geometric deviation (SGD), the calculations are carried out of dose coefficient at fixed value of activity median aerodynamic diameter in the range of SGD occurred in practice. The method of approximation of given distribution by the sum of lognormal distributions with table parameters is used for calculations. It is shown that the dependence of dose coefficient on SGD is quite weak. Thus, if the aerosol activity is distributed on its particles according to logarithmic-normal law then dose coefficient depends only on activity median aerodynamic diameter

С целью установить зависимость дозового коэффициента (доза на единицу поступления) от стандартного геометрического отклонения (СГО), проведены расчеты дозового коэффициента при фиксированном значении аэродинамического медианного по активности диаметра в диапазоне встречающихся на практике значений СГО. Расчеты выполнены методом аппроксимации заданного распределения суммой логнормальных распределений с табличными параметрами. Показано, что зависимость дозового коэффициента от СГО весьма слабая. Таким образом, если активность аэрозоля распределена по его частицам в соответствии с логарифмически нормальным законом, то можно считать, что дозовый коэффициент зависит только от аэродинамического медианного по активности диаметра

Radiation monitoring of the north seas

S54

Uvarov, A.D.; Vakulovskij, S.M.; Artem'ev, G.B.; Valetova, N.K.; Petrenko, G.I. (FGBU #Left-Pointing Double Angle Quotation Mark#NPO#Left-Pointing Double Angle Quotation Mark#Tajfun#Right-Pointing Double Angle Quotation Mark#, Obninsk (RU))

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Radiatsionnyj monitoring severnykh morej

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416 p.

ARCTIC OCEAN; SEAS; CONTAMINATION; RADIOISOTOPES; RADIATION MONITORING; RUSSIAN FEDERATION; NORWAY; ENVIRONMENTAL PROTECTION

The regular observation of Roshydromet on radionuclide content in the Barents and Kara Seas began in the 1960-s. From 1992 collaboration of Russian and Norwegian scientists started within a framework of cooperation in the field of environmental protection. The results of long-term observations are considered and their analysis is done

Регулярные наблюдения Росгидромета за содержанием радионуклидов в Баренцевом и Карском морях были начаты в 60-х годах 20 века. С 1992 г. начались совместные работы специалистов России и Норвегии в рамках сотрудничества в области охраны окружающей среды. Рассмотрены результаты многолетних наблюдений, и дан их анализ

Biological monitoring with the use of plants in the areas of RW storages influence

S63

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Geras'kin, S.A.; Dikareva, N.S. (FGBNU Vserossijskij Nauchno-Issledovatel'skij Inst. Radiologii i Agroehkologii, Obninsk (RU)); Pyatkova, S.V.; Gorshkova, T.A. (Obninskij Inst. Atomnoj Ehnergetiki - Filial NIYaU #Left-Pointing Double Angle Quotation Mark#MIFI#Right-Pointing Double Angle Quotation Mark#, Obninsk (RU)); Kiselev, S.M. (FGBU GNTs Federal'nyj Meditsinskij Biofizicheskij Tsentr im. A.I. Burnazyana FMBA Rossii, Moscow (RU))

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Biologicheskij monitoring s ispol'zovaniem rastenij v zonakh vliyaniya khranilishch RAO

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(RU)

8 refs., 4 figs.

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RADIOACTIVE WASTE STORAGE; CONTROLLED AREAS; BIOLOGICAL INDICATORS; SURFACE WATERS; GROUND WATER; BIOLOGICAL RADIATION EFFECTS; ALLIUM CEPA; ROOTS; MITOTIC INDEX; MONITORING; ENVIRONMENT; CYTOLOGY; GENES; TOXICITY

The results of biological monitoring in the areas of influence of three radioactive waste storages in various climate zones are generalized. Specific activity of main radionuclides are determined, as well as some physico-chemical parameters and content of certain chemical elements in natural media. On all objects the biotesting of natural (mainly, ground) waters with the use of plant test-system - Allium-test is carried out. The test-system allows to estimate cyto- and genotoxicity of water media of different composition. Using this test the sampling points, water of which shows sparing action on mitotic ability of plants and has increased mutagenic activity, are revealed. The results obtained can be used for optimization of monitoring and decision-making on organization of rehabilitation for reducing negative impacts of radiation-hazardous objects on environment

Обобщены результаты биологического мониторинга в зонах влияния трех хранилищ радиоактивных отходов, расположенных в различных климатических зонах. Определены удельные активности основных радионуклидов, а также ряд физико-химических параметров и содержание определенных химических элементов в природных средах. На всех объектах проведено биотестирование природных (в основном, подземных) вод с помощью растительной тест-системы - Allium-теста, которая позволяет оценить цито- и генотоксичность водных сред разного компонентного состава. С помощью этого теста выявлены точки пробоотбора, вода в которых оказывает угнетающее действие на митотическую активность растений и обладает повышенной мутагенной способностью. Полученные результаты могут быть использованы для оптимизации мониторинга и принятия решений по организации реабилитационных мероприятий для снижения негативного воздействия радиационно-опасных объектов на окружающую среду

Current radiation situation in the south-west territories of the Bryansk Region according to the results of complex certification

S54

Shubina, O.A.; Titov, I.E.; Krechetnikov, V.V. (FGBNU VNIIRAEh, Obninsk (RU))

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Sovremennaya radiatsionnaya obstanovka v yugo-zapadnykh rajonakh Bryanskoj oblasti po rezul'tatam kompleksnoj pasportizatsii

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RUSSIAN FEDERATION; CHERNOBYLSK-4 REACTOR; REACTOR ACCIDENTS; SURFACE CONTAMINATION; RADIOECOLOGY; ENVIRONMENTAL QUALITY; RURAL AREAS; AGRICULTURE; RADIATION DOSES; REMEDIAL ACTION; ECONOMY; ECONOMICS

Starting basis for realization of the rehabilitation program of the farms on the territories of the Bryansk Region contaminated after the Chernobyl accident is the data on radioecological monitoring of soils and their present condition, as well as the data on sociodemographic conditions and economic situation at objects selected. Since 2011 the activity is carried out on creation of certificates of safe living on contaminated territories, where all this information is collected. The aim of conditioning is provision of population and executive bodies by concrete and objective information for grounding and carrying out remedial activities and making decision on proving safe communities of population, farm management on contaminated territories. The agricultural lands must be certificated if cesium-137 contamination density is 185 kBq/m²

Исходной базой для реализации программ реабилитации сельскохозяйственных предприятий, расположенных на территориях Брянской области, загрязненных радионуклидами после Чернобыльской аварии, являются данные радиоэкологического мониторинга земель и их состояния на данный момент времени, а также данные о социально-демографической и экономической ситуации в выбранных объектах. С 2011 г. ведутся работы по созданию паспортов безопасности проживания на радиоактивно загрязненных территориях, в которых собраны все указанные выше данные. Цель паспортизации - обеспечение населения и органов исполнительной власти конкретной и объективной информацией для обоснования и проведения реабилитационных мероприятий и принятия решений по обеспечению безопасного проживания населения, ведения хозяйства на радиоактивно загрязненных территориях. Паспортизации подлежат сельскохозяйственные угодья, в которых плотность загрязнения цезием-137 составляет 185 кБк/м²

Forms of ¹³⁷Cs compounds in soils of natural and agroecosystems of radioactively contaminated territories of the Bryansk Region

S54

Shcheglov, A.I.; Tsvetnova, O.B.; Skryabinskij, D.A. (Moskovskij Gosudarstvennyj Univ. im. M.V. Lomonosova, Moscow (RU))

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Formy soedinenij ^{137}Cs v pochvakh prirodnykh i agroekosistem radioaktivno
zagryaznennykh territorij Bryanskoj oblasti

p. 404-415

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24 refs., 2 tabs.

Shershakov, V.M. (ed.)

Postoyannyj Komitet Soyuznogo Gosudarstva, Moscow (RU); Federal'naya Sluzhba po

Gidrometeorologii i Monitoringu Okruzhayushchej Sredy, Moscow (RU)

Radioactivity after the nuclear explosions and accidents: effects and ways to overcome.

Proceedings of the International scientific-practical conference

Radioaktivnost' posle yadernykh vzryvov i avarij: posledstviya i puti preodoleniya. Sbornik

trudov Mezhdunarodnoj nauchno-prakticheskoj konferentsii

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CHERNOBYLSK-4 REACTOR; REACTOR ACCIDENT SIMULATION; REACTOR
ACCIDENTS; SURFACE CONTAMINATION; SOILS; TERRESTRIAL ECOSYSTEMS;
AGRICULTURE; CESIUM 137; CESIUM COMPOUNDS; RADIONUCLIDE MIGRATION;
FORESTS; DISTRIBUTION; TIME DEPENDENCE; SOLUBILITY

Estimation is given to the present-day levels and further dynamics of cesium-137 forms in the soils of natural and agroecosystems of the Bryansk Region territories contaminated after the Chernobyl accident. According to the data of 2015 cesium-137 contamination density in agroecosystem is equal to 137.04, in birch forest - 2775.78 and in pine forest - 3354.16 kBq/m². The bulk of cesium-137 is concentrated in soils of pine forest in the depth of litter (64%), of birch forest - in 0-5 cm organo-mineral layer (57%) and of agroecosystem - in 0-30 cm of plough-layer. Relative amount of cesium-137 mobile fractions in the soils of agroecosystem is higher than in forest biogeocenoses, and nonexchangeable forms and residues is lower except for leaf layer of litter. The observations in 1998-2015 show that relative amount of cesium-137 mobile fractions reduces in course of time by 1.5-2 times, and of fixed ones grows up, and that mobile cesium-137 even in sand soils transfers into nonexchangeable state

Проведена оценка современных уровней нахождения и дальнейшей динамики форм соединений цезия-137 в почвах природных и агроэкосистем радиоактивно загрязненных территорий Брянской области, пострадавших в результате аварии на Чернобыльской АЭС. По данным 2015 г. плотность загрязнения почв цезием-137 составила в агроценозе 137,04, в березняке - 2775,78 и сосняке - 3354,16 кБк/м². Основной запас цезия-137 в почвах сосняка сосредоточен в толще подстилки (64%), березняка - в органоминеральном слое 0-5 см (57%), а агроценоза - в 0-30 см пахотного горизонта. Относительное содержание подвижных фракций цезия-137 в почвах агроценоза выше, чем в лесных биогценозах, а необменных форм и остатка меньше, исключая лиственный слой подстилки. Наблюдения 1998-2015 гг. показывают, что относительное содержание подвижных фракций цезия-137 со временем сокращается в 1,5-2 раза, а неподвижных возрастает, и что со временем подвижный цезий-137 даже в песчаных почвах переходит в необменное состояние

S75;S36

Panasyuk, M.I. (ed.)

Moskovskij Gosudarstvennyj Univ. im. M.V. Lomonosova, Nauchno-Issledovatel'skij Inst. Yadernoj Fiziki im. D.V. Skobel'tsyna, Moscow (RU)

XLIII International Tulinov conference on physics of interactions of charged particles with crystals

Moscow (RU)

28-30 May 2013

XLIII mezhdunarodnaya Tulinovskaya konferentsiya po fizike vzaimodejstviya zaryazhennykh chastits s kristallami

Tezisy dokladov XLIII mezhdunarodnoj Tulinovskoj konferentsii po fizike vzaimodejstviya zaryazhennykh chastits s kristallami

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LEADING ABSTRACT; MEETINGS; ELECTRON BEAMS; POSITRON BEAMS; CRYSTALS; SURFACE PROPERTIES; CHANNELING; ION IMPLANTATION; ION BEAMS; SPUTTERING; SURFACE COATING

The collection contains summaries of reports of the XLIII International Tulinov conference on physics of interactions of charged particles with crystals. The problems of physics of orientation effects are considered. The recent results of investigations of electrons and positrons radiation in solids are presented. The problems of scattering, sputtering and emission of secondary particles are discussed. The particular attention is given to modification of materials surfaces by means of charged particles

Сборник содержит тезисы докладов XLIII международной Тулиновской конференции по физике взаимодействия заряженных частиц с кристаллами. Рассмотрены проблемы физики ориентационных эффектов. Представлены последние результаты исследований излучения электронов и позитронов в твердом теле. Обсуждаются вопросы рассеяния, распыления и эмиссии вторичных частиц. Особое внимание уделено модификации поверхностей материалов с помощью пучков заряженных частиц

Effect of #gamma#-irradiation of Pd-rod on change of element composition of surfaces of all components of deuterium high-pressure chamber

S75

Didyk, A.Yu. (Ob''edinennyj Inst. Yadernykh Issledovanij, Lab. Yadernykh Reaktsij imeni G.N. Flerova, Dubna (XJ)); Vishnevskij, R. (Narodove Centrum Badan Jadrowych, Otwock-Swierk (PL))

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p. 3

(RU)

4 refs.

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Nuclear interaction of charged particles at low energy in nonstationary systems (low-temperature nuclear synthesis: from theory to technology)

S73

Vysotskij, V.I.; Vysotskij, M.V. (Kievskij Univ. im. T. Shevchenko, Kiev (UA))

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Yadernoe vzaimodejstvie zaryazhennykh chastits pri nizkoj ehnergii v nestatsionarnykh sistemakh (nizkotemperaturnyj yadernyj sintez: ot teorii k tekhnologii?)

p. 4

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4 refs., 1 fig.

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Hydrogen yield from zirconium and titanium at thermal and radiation exposure
S75

Krasnov, D.N.; Sypchenko, V.S.; Tyurin, Yu.I.; Chernov, I.P. (Natsional'nyj Issledovatel'skij Tomskij Politekhnikeskij Univ., Tomsk (RU))

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Vykhod vodoroda iz tsirkoniya i titana pri termicheskom i radiatsionnom vozdejstvii

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Ionization energy losses of ultrarelativistic electron with nonequilibrium field
S75

Trofimenko, S.V.; Shul'ga, N.F. (Inst. Teoreticheskoy Fiziki im. A.I. Akhiezer NNTs KhFTI, Khar'kov (UA))

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Ionizatsionnye poteri ehnergii ul'trarelyativistskogo ehlektrona s neravnovesnym polem

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**Effect of density in equilibrium charge distributions of carbon and oxygen ions
S75**

Novikov, N.V.; Teplova, Ya.A. (NIIYaF MGU, Moscow (RU))

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p. 7

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**Study of far-action effect by means electron-diffraction spectrograph
S36**

Levshunova, V.L.; Pitirimova, E.A. (Nauchno-Issledovatel'skij Fiziko-Tekhnicheskij Inst. NNGU, Nizhnij Novgorod (RU)); Pokhil, G.P.; Tetel'baum, D.I. (Nauchno-Issledovatel'skij Inst. Yadernoju Fiziki imeni D.V. Skobel'tsyna, MGU imeni M.V. Lomonosova, Moscow (RU))

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Issledovanie ehffekta dal'nodejstviya na ehlektronografe

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Spiral orientation motion of proton beam around dielectric crystalline fibers **S75**

Maksyuta, N.V.; Vysotskij, V.I.; Efimenko, S.V. (Kievskij Natsional'nyj Univ. imeni Tarasa Shevchenko, Kiev (UA))

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Spiral'noe orientatsionnoe dvizhenie puchka protonov vokrug dielektricheskikh kristallicheskikh volokon

p. 9

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Atomistic simulation of interaction of mobile edge dislocations with radiation defects in austenite alloys Fe-Ni-Cr

S36

Bakaev, A.V.; Grigor'ev, P.Yu. (SPbGPU, Sankt-Peterburg (RU)); SCK-CEN, Mol (BE)); Terent'ev, D.A. (SCK-CEN, Mol (BE)); Zhurkin, E.E. (SPbGPU, Sankt-Peterburg (RU))

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p. 10

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To statistical properties of energy levels in quantum theory of axial channeling
S75

Shul'ga, N.F. (ITF im. A.I. Akhiezer NNTs KhFTI, Khar'kov (UA)); Syshchenko, V.V.; Neryabova, V.S. (NIU BelGU, Belgorod (RU)); Isupov, A.Yu. (LVEh OIYaI, Dubna (XJ))

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ELECTRON CHANNELING; CRYSTALS; QUANTUM SYSTEMS; QUANTUM STATES;
SILICON; CHAOS THEORY; STATISTICS; EIGENVALUES; THEORETICAL DATA

Study of interaction of 10 keV electron beam with dielectric surface

S75

Vokhmyanina, K.A.; Zhukova, P.N.; Kubankin, A.S.; Le Tkhi Khoaj; Nazhmudinov, R.M.; Pleskanev, A.A.; Nasonov, N.N. (Lab. Radiatsionnoj Fiziki, NIU BelGU, Belgorod (RU)); Pokhil, G.P. (NIIYaF im. D.V. Skobel'tsyna, MGU im. M.V. Lomonosova, Moscow (RU))

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Issledovanie vzaimodejstviya 10 keV ehlektronov s dielektricheskoy poverkhnost'yu

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Mathematical simulation of three-dimensional diffusion of excitons excited by electron beam in wide-band-gap semiconducting materials

S75

Amrastanov, N.N.; Polyakov, A.N.; Stepovich, M.A. (Kaluzhskij Gosudarstvennyj Univ. im. K.Eh. Tsiolkovskogo, Kaluga (RU))

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Matematicheskoe modelirovanie trekhmernoj diffuzii ehksitonov, vzbuzhdaemykh ehlektronnym puchkom v shirokozonnnykh poluprovodnikovyykh materialakh

p. 13

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Dimensions of probe particles in weak dissipative KAM theory

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Bogdanov, R.I. (NII YaF MGU, Moscow (RU)); Bogdanov, M.R. (MGU, IEh, Moscow (RU))

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Razmery probnykh chastits v slabo-dissipativnoj KAM-teorii

p. 14

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Energy losses of beryllium ions at transmission thorough thin films till achievement of charge equilibrium

S75

Belkova, Yu.A.; Teplova, Ya.A. (NIIYaF MGU, Moscow (RU))

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Transmission of ions through cone dielectric capillary

S75

Pokhil, G.P.; Cherdyntsev, V.V. (Nauchno-Issledovatel'skij Inst. Yadernoj Fiziki imeni D.V. Skobel'tsyna, MGU imeni M.V. Lomonosova, Moscow (RU))

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Koshcheev, V.P. (NIU MAI, Filial #Left-Pointing Double Angle Quotation Mark#Strela#Right-Pointing Double Angle Quotation Mark#, Zhukovskij Moskovskoj Obl. (RU)); Morgun, D.A.; Shtanov, Yu.N.; Panina, T.A. (GBOU VPO #Left-Pointing Double Angle Quotation Mark#Surgutskij Gosudarstvennyj Univ. KhMAO - Yugry#Right-Pointing Double Angle Quotation Mark#, Surgut (RU))

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Babaev, A.A. (Tomskij Politehnicheskij Inst., Tomsk (RU); INFN, Sez. Roma La Sapienza, Rome (IT)); Kavoto, Zh.-L. (INFN, Sez. Roma La Sapienza, Rome (IT)); Dabagov, S.B. (INFN, Lab. Nazionali di Frascati, Frascati (IT)); Fizicheskij Inst. im. P.N. Lebedeva RAN, Moscow (RU))

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Vysotskij, V.I.; Vasilenko, A.O. (Kievskij Univ. im. T. Shevchenko, Kiev (UA)); Kornilova, A.A.; Sysoev, N.N. (MGU im. M.V. Lomonosova, Moscow (RU)); Tomak, V.I. (NII EhM pri MGTU (MVTU) im. N.Eh. Bauman, Moscow (RU))

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Maksyuta, N.V. (Kievskij Natsional'nyj Univ. imeni Tarasa Shevchenko, Kiev (UA))

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Kudriavtsev, Yu.; Asomoza, R. (Cinvestav-IPN, Mexico DF, Mexico Seccion Electronica del Estado Solido, Dept. Ingenieria Electrica, CINVESTAV-IPN, Mexico (MX))

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Balashova, L.L. (NII Yadernoj Fiziki imeni D.V. Skobel'tsyna, MGU imeni M.V. Lomonosova, Moscow (RU)); Trikalinos, Kh. (Athens Univ., Physics Dept., Athens (GR))

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Chumanov, V.Ya.; Kadenskij, A.G.; Chechenin, N.G. (NIIYaF MGU, Moscow (RU))

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Vukolov, A.V.; Kaplin, V.V.; Sukhikh, L.G.; Uglov, S.R. (Tomskij Politehnicheskij Univ., Tomsk (RU))

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S75

Abdrashitov, S.V. (Tomskij Politehnicheskij Univ., Tomsk (RU); Tomskij Gosudarstvennyj Univ., Tomsk (RU)); Korotchenko, K.B.; Pivovarov, Yu.L. (Tomskij Politehnicheskij Univ., Tomsk (RU))

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Maksyuta, N.V.; Vysotskij, V.I.; Efimenko, S.V. (Kievskij Natsional'nyj Univ. imeni Tarasa Shevchenko, Kiev (UA))

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S75

Shul'ga, N.F.; Fomin, S.P.; Truten', V.I. (ITF NNTs KhFTI, Khar'kov (UA))

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S46

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Ligidov, A.Z. (NIYaU MIFI, Moscow (RU)); Anania, M.P.; Ferrario, M. (INFN - LNF, Rim (IT)); Dabagov, S.B. (NIYaU MIFI, Moscow (RU); INFN - LNF, Rim (IT))

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S75

Pivovarov, Yu.L.; Sinitsyn, E.A. (Natsional'nyj Issledovatel'skij Tomskij Politekhnikeskij Univ., Tomsk (RU))

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Pivovarov, Yu.L.; Tukhfatullin, T.A. (Tomsk Polytechnic Univ., Tomsk (RU)); Takabayashi, Y. (SAGA Light Source, Tosu (JP))

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S75

Bogdanov, O.V. (Natsional'nyj Issledovatel'skij Tomskij Politekhneskij Univ., Tomsk (RU); Natsional'naya Lab. Fraskati, Fraskati (IT)); Pivovarov, Yu.L.; Fiks, E.I. (Natsional'nyj Issledovatel'skij Tomskij Politekhneskij Univ., Tomsk (RU))

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Ergunov, I.A.; Dabagov, C.B. (INFN Lab. Nazionali di Frascati, Frascati (IT); NIYaU MIFI, Moscow (RU); FIAN RAN, Moscow (RU))

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S75

Grishin, V.K. (NIIYaF MGU, Moscow (RU))

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S07

Vnukov, I.E.; Laktionova, S.A.; Pligina, O.O.; Ryabchuk, V.V.; Sidnin, M.A.; Skhomenko, Ya.T.; Shatokhin, R.A. (Belgorodskij Gosudarstvennyj Natsional'nyj Issledovatel'skij Univ., Belgorod (RU))

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Vnukov, I.E.; Laktionova, S.A.; Pligina, O.O.; Ryabchuk, V.V.; Sidnin, M.A.; Skhomenko, Ya.T.; Shatokhin, R.A. (Belgorodskij Gosudarstvennyj Natsional'nyj Issledovatel'skij Univ., Belgorod (RU))

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Syshchenko, V.V.; Tarnovskij, A.I. (Belgorodskij Gosudarstvennyj Natsional'nyj Issledovatel'skij Univ., Belgorod (RU))

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S36

Pavlenko, V.I.; Edamenko, O.D.; Cherkashina, N.I. (Belgorodskij Gosudarstvennyj Tekhnologicheskij Univ. im. V.G. Shukhova, Belgorod (RU)); Noskov, A.V. (Belgorodskij Univ. Kooperatsii, Ehkonomiki i Prava, Belgorod (RU))

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Blazhevich, S.V.; Gladkikh, Yu.P.; Noskov, A.V. (Belgorodskij Gosudarstvennyj Univ., Belgorod (RU))

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S07

Blazhevich, S.V.; Kolosova, I.V.; Grazhdankin, G.A.; Noskov, A.V. (Belgorodskij Gosudarstvennyj Univ., Belgorod (RU))

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Zhukova, P.N.; Le Tkhi Khoaj; Nasonov, N.N. (Lab. Radiatsionnoj Fiziki, NIU BelGU, Belgorod (RU))

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S75

Bogdanov, O.V (Natsional'naya Lab. Fraskati, Fraskati (IT); Tomskij Politekhnikeskij Univ., Tomsk (RU)); Dabagov, S.B. (Natsional'naya Lab. Fraskati, Fraskati (IT); Fizicheskij Inst. im. Lebedeva, Moscow (RU))

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S75

Abdrashitov, S.V. (Tomskij Politekhnikeskij Univ., Tomsk (RU); Tomskij Gosudarstvennyj Univ., Tomsk (RU)); Bogdanov, O.V. (Tomskij Politekhnikeskij Univ., Tomsk (RU)); Natsional'naya Lab. Fraskati, Fraskati (IT)); Pivovarov, Yu.L. (Tomskij Politekhnikeskij Univ., Tomsk (RU))

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Zakharova, M.A.; Pivovarov, Yu.L.; Tukhfatullin, T.A. (Natsional'nyj Issledovatel'skij Tomskij Politekhnikeskij Univ., Tomsk (RU))

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Tyschenko, I.E. (Inst. Fiziki Poluprovodnikov im. A.V. Rzhanova, Novosibirsk (RU))

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Gomoyunova, M.V.; Grebenyuk, G.S.; Pronin, I.I. (FTI im. A.F. Ioffe RAN, Sankt-Peterburg (RU))

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Kristya, V.I.; Je Naing Tun (Kaluzhskij Filial MGTU im. N.Eh. Baumana, Kaluga (RU))

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Popov, V.P.; Tyschenko, I.E. (Inst. of Semiconductor Physics, Novosibirsk (RU)); Avila, T.S.; Grande, P.L. (Inst. of Physics, UFRGS, Porto Alegre (BR))

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Ermakov, Yu.A. (Nauchno-Issledovatel'skij Inst. Yadernoj Fiziki imeni D.V. Skobel'tsyna, MGU imeni M.V. Lomonosova, Moscow (RU)); Ieshkin, A.E.; Chernysh, V.S. (MGU imeni M.V. Lomonosova, Fizicheskij Fakul'tet, Moscow (RU))

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Afanas'ev, V.P.; Lubenchenko, A.V.; Kostanovskij, I.A.; Ivanov, D.A. (NIU MEhI, Moscow (RU)); Ermilov, A.N.; Tyuryukanov, P.M.; Nefedova, V.Eh. (FGUP VEhI, Moscow (RU))

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Arustamov, V.N.; Ashurov, Kh.B.; Kadyrov, Kh.Kh.; Nagajbekov, R.B.; Khudajkulov, I.Kh. (Inst. Ionno-Plazmennyykh i Lazernyykh Tekhnologij AN RUz, Tashkent (UZ))

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S36

Kalin, B.A.; Volkov, N.V.; Olejnikov, I.V.; Valikov, R.A. (Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI (NIYaU MIFI), Moscow (RU))

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Novgorod (RU)); Shengurov, V.G. (Nauchno-Issledovatel'skij Fiziko-Tekhnicheskij Inst. pri

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Oksengendler, B.L.; Maksimov, S.E. (Inst. Ionno-Plazmennyykh i Lazernyykh Tekhnologiy AN RUz, Tashkent (UZ)); Vojtsekhovskij, I.A. (Alderson-Broadus College, Philippi, WV (US))

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Dzhemilev, N.Kh.; Kovalenko, S.F.; Maksimov, S.E.; Tukfatullin, O.F.; Khozhiev, Sh.T. (Inst. Ionno-Plazmennyykh i Lazernyykh Tekhnologiy AN RUz, Tashkent (UZ))

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S75

Dzhemilev, N.Kh.; Maksimov, S.E.; Khozhiev, Sh.T. (Inst. Ionno-Plazmennyykh i Lazernyykh Tekhnologij AN RUz, Tashkent (UZ))

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S75

Didyk, A.Yu. (Ob''edinennyj Inst. Yadernykh Issledovanij, Lab. Yadernykh Reaktsij imeni G.N. Flerova, Dubna (XJ)); Vishnevskij, R. (Narodove Centrum Badan Jadrowych, Otwock-Swierk (PL)); Zatekin, V.V.; Kulikauskas, V.S. (Nauchno-Issledovatel'skij Inst. Yadernoj Fiziki imeni D.V. Skobel'tsyna Moskovskogo Gosudarstvennogo Univ. imeni M.V. Lomonosova, Moscow (RU)); Serushkin, N.V. (MGU imeni N.E. Bauman, Moscow (RU))

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S75

Didyk, A.Yu. (Ob''edinennyj Inst. Yadernykh Issledovanij, Lab. Yadernykh Reaktsij imeni G.N. Flerova, Dubna (XJ)); Vishnevskij, R. (Narodove Centrum Badan Jadrowych, Otwock-Swierk (PL)); Kulikauskas, V.S. (Nauchno-Issledovatel'skij Inst. Yadernoj Fiziki imeni D.V. Skobel'tsyna Moskovskogo Gosudarstvennogo Univ. imeni M.V. Lomonosova, Moscow (RU))

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Grigor'ev, P.Yu.; Bakaev, A.V. (Sankt-Peterburgskij Gosudarstvennyj Politehnicheskij Univ. (SPbGPU), Sankt-Peterburg (RU); Bel'gijskij Tsentri Yadernykh Issledovanij SCK-CEN, Mol (BE)); Dubinko, V.I. (Natsional'nyj Nauchnyj Tsentri Khar'kovskij Fiziko-Tekhnicheskij Inst., Khar'kov (UA)); Terent'ev, D.A. (Bel'gijskij Tsentri Yadernykh Issledovanij SCK-CEN, Mol (BE)); Zhurkin, E.E. (Sankt-Peterburgskij Gosudarstvennyj Politehnicheskij Univ. (SPbGPU), Sankt-Peterburg (RU))

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S75

Borisov, A.M.; Vostrikov, V.G.; Romanovskij, E.A.; Tkachenko, N.V. (NIIYaF MGU, Moscow (RU)); Vinogradov, A.V.; Savushkina, S.V. (MATI - RGTU imeni K.Eh. Tsiolkovskogo, Moscow (RU)); Polyanskij, M.N. (GNTs #Left-Pointing Double Angle Quotation Mark#Issledovatel'skij Tsentri imeni M.V. Keldysha#Right-Pointing Double Angle Quotation Mark#, Moscow (RU))

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Borisov, A.M.; Vostrikov, V.G.; Romanovskij, E.A.; Tkachenko, N.V. (NIIYaF MGU, Moscow (RU)); Savushkina, S.V. (#Left-Pointing Double Angle Quotation Mark#MATI - RGTU imeni K.Eh. Tsiolkovskogo#Right-Pointing Double Angle Quotation Mark#, Moscow (RU)); Polyanskij, M.N. (GNTs #Left-Pointing Double Angle Quotation Mark#Issledovatel'skij Tsentri imeni M.V. Keldysha#Right-Pointing Double Angle Quotation Mark#, Moscow (RU))

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Ermakov, Yu.A. (Nauchno-Issledovatel'skij Inst. Yadernoj Fiziki imeni D.V. Skobel'tsyna MGU imeni M.V. Lomonosova, Moscow (RU)); Ieshkin, A.E.; Chernysh, V.S. (MGU imeni M.V. Lomonosova, Fizicheskij Fakul'tet, Moscow (RU))

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Opanasyuk, A.S.; Koval', P.V. (Sumskij Gosudarstvennyj Univ., Sumy (UA)); Magilin, D.V.; Ponomarev, A.A. (Inst. Prikladnoj Fiziki NAN Ukrainy, Sumy (UA)); Cheong, H. (Sogang Univ., Seoul (KR))

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Petukhov, V.P.; Kulikauskas, V.S.; Novikov, L.S.; Petrov, D.V.; Shemukhin, A.A.; Chernik, V.N. (NIIYaF MGU, Moscow (RU)); Smirnova, T.N.; Shumov, A.E. (GKNPTs im. M.V. Khrunicheva,

Moscow (RU)); *Sal'nikova, I.A.; Sedov, V.V. (NII EhMI, Moscow (RU))*

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Mikhalkovich, O.M.; Tashlykov, I.S. (Belorusskij Gosudarstvennyj Pedagogicheskij Univ., Minsk (BY))

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S75

Bobrovich, O.G. (Belorusskij Gosudarstvennyj Tekhnologicheskij Univ., Minsk (BY));

Mikhalkovich, O.M.; Tashlykov, I.S. (Belorusskij Gosudarstvennyj Pedagogicheskij Univ., Minsk (BY))

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S75

Turovets, A.I.; Tashlykov, I.S. (BGPU im. M. Tanka, Minsk (BY)); Kulikauskas, V.S. (NIIYaF MGU, Moscow (RU))

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Influence of hydrogen on physical properties titanium alloy modified by pulsed ion beam

S36

Berezneeva, E.V.; Berezneev, D.V.; Pushilina, N.S.; Chernov, I.P. (Natsional'nyj Issledovatel'skij Tomskij Politehnicheskij Univ., Tomsk (RU))

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Kovivchak, V.S. (Omskij Gosudarstvennyj Univ. im. F.M. Dostoevskogo, Omsk (RU)); Omskij Nauchnyj Tsentr SO RAN, Omsk (RU)); Panova, T.V. (Omskij Gosudarstvennyj Univ. im. F.M. Dostoevskogo, Omsk (RU)); Krivozubov, O.V.; Knyazev, E.V. (Omskij Nauchnyj Tsentr SO RAN, Omsk (RU)); Leont'eva, N.N. (Inst. Problem Pererabotki Uglevodorodov SO RAN, Omsk (RU))

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S36

Kovivchak, V.S.; Panova, T.V.; Mikhajlov, K.A. (Omskij Gosudarstvennyj Univ. im. F.M. Dostoevskogo, Omsk (RU))

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S36

Serba, P.V.; Avdeev, S.P.; Lugovoj, E.V. (Yuzhnyj Federal'nyj Univ., Rostov-na-Donu (RU)); Petrov, S.N.; Garanzha, S.N. (OAO NPP Kosmicheskogo Priborostroeniya #Left-Pointing Double Angle Quotation Mark#Kvant#Right-Pointing Double Angle Quotation Mark#, Rostov-na-Donu (RU))

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Trokhimchuk, P.P. (Vostochnoevropejskij Natsional'nyj Univ. imeni Lesi Ukrainki, Lutsk (UA))

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S36

Uglov, V.V. (Belorusskij Gosudarstvennyj Univ., Minsk (BY)); Kvasov, N.T.; Kudaktin, R.S.; Petukhov, Yu.A. (Belorusskij Gosudarstvennyj Univ. Informatiki i Radioehlektroniki, Minsk (BY))

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Remnev, G.E.; Linnik, S.A. (Tomskij Politehnicheskij Univ., Tomsk (RU)); Uglov, V.V.;

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Uglov, V.V.; Kalin, A.V. (Belorusskij Gosudarstvennyj Univ., Minsk (BY)); Koval', N.N.; Ivanov, Yu.F.; Teresov, A.D. (Inst. Sil'notochnoj Ehlektroniki SO RAN, Tomsk (RU)); Petukhov, Yu.A. (Belorusskij Gosudarstvennyj Univ. Informatiki i Radioehlektroniki, Minsk (BY))

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Shimanskij, V.I.; Cherenda, N.N.; Uglov, V.V. (Belorusskij Gosudarstvennyj Univ., Minsk (BY)); Astashinskij, V.M. (Inst. Fiziki im. B.I. Stepanova NANB, Minsk (BY))

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S75

Tashlykov, I.S. (Belorusskij Gosudarstvennyj Pedagogicheskij Univ., Minsk (BY)); Tul'ev, V.V. (Belorusskij Gosudarstvennyj Tekhnologicheskij Univ., Minsk (BY))

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Elemental composition, morphology and wettability of surfaces of aluminium alloy foils

S36

Tashlykova-Bushkevich, I.I. (Belorusskij Gosudarstvennyj Univ. Informatiki i Radioelektroniki, Minsk (BY)); Zyryanov, S.S.; Neshov, F.G.; Ryabukhin, O.V. (Ural'skij Federal'nyj Univ. im. Pervogo Prezidenta Rossii B.N. El'tsina, Ekaterinburg (RU)); Kulikauskas, V.S.; Chernykh, P.N. (NIYaF MGU, Moscow (RU)); Shepelevich, V.G. (Belorusskij Gosudarstvennyj Univ., Minsk (BY)); Yakovenko, Yu.S. (Belorusskij Gosudarstvennyj Pedagogicheskij Univ., Minsk (BY))

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Composition and microstructure of layers shaped by ion-assisted deposition of rare-earth metals and platinum from pulsed arc-discharge plasma on tantalum S75;S36

Poplavskij, V.V.; Dorozhko, A.V. (Belorusskij Gosudarstvennyj Tekhnologicheskij Univ., Minsk (BY))

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Sostav i mikrostruktura sloev, formiruemykh ionno-assistiruемым osazhdeniem

redkozemel'nykh metallov i platiny iz plazmy impul'snogo dugovogo razryada na tantal

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Semiempirical method of determination of nuclear backscattering cross sections S75

Zyryanov, S.S.; Kruzhalov, A.V.; Ryabukhin, O.V.; Neshov, F.G. (FGAOU VPO #Left-Pointing Double Angle Quotation Mark#Ural'skij Federal'nyj Univ. imeni Pervogo Prezidenta Rossii B.N. El'tsina#Right-Pointing Double Angle Quotation Mark#, Fiziko-Tekhnologicheskij Inst., Kafedra Eksperimental'noj Fiziki, Ekaterinburg (RU)), e-mail: rov@dpt.ustu.ru

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Measurement of integral absorbed dose of ionizing radiations by means of MOS-transistors

S75

Andreev, V.V.; Solov'ev, I.V.; Akhmelkin, D.M.; Romanov, A.V. (MGTU im. N.Eh. Baumana, Kaluzhskij Filial, Kaluga (RU)); Bondarenko, G.G. (Moskovskij Gosudarstvennyj Inst. Ehlektroniki i Matematiki, Moscow (RU))

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Features of state of protection oxide films on ion-modified surface of zirconium tubes

S36

Valikov, R.A.; Volkov, N.V. (Natsional'nyj Issledovatel'skij Yadernyj Univ. MIFI (NIYaU MIFI), Moscow (RU))

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Baskov, V.A. (Fizicheskij Inst. im. P.N. Lebedeva RAN, Moscow (RU))

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Chernik, V.N. (NIIYaF MGU, Moscow (RU))

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Vershinin, G.A.; Volkov, V.A.; Bukhbinder, G.L.; Gering, G.I. (Omskij Gosudarstvennyj Univ. im. F.M. Dostoevskogo, Omsk (RU))

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Timchenko, N.A.; Galimov, R.M. (Natsional'nyj Issledovatel'skij Tomskij Politekhnikeskij Univ., Tomsk (RU)); Krysina, O.V. (Inst. Sil'notochnoj Ehlektroniki SO RAN, Tomsk (RU))

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Kabyshev, A.V.; Konusov, F.V.; Remnev, G.E. (Inst. Fiziki Vysokikh Tekhnologij, Tomsk (RU))

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S75;S46

Maslov, N.I. (IFVEhYaF NNTs KhFTI, Khar'kov (UA))

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Badawi, M.S.; Krar, M.E.; El-Khatib, A.M. (Physics Dept., Faculty of Science, Alexandria Univ., Alexandria (EG))

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S36;S37

Vykhodets, V.B.; Kurennykh, T.E.; Obukhov, S.I. (Inst. Fiziki Metallov UrO RAN, Ekaterinburg (RU)); Beketov, I.V.; Samatov, O.M. (Inst. Ehlektrofiziki UrO RAN, Ekaterinburg (RU))

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S36;S37

Vykhodets, V.B.; Kurennykh, T.E. (IFM UrO RAN, Ekaterinburg (RU)); Fedorova, O.M.; Fishman, A.Ya. (IMET UrO RAN, Ekaterinburg (RU))

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Oxidation of polyethylene under irradiation by fast ions

S37;S36

Zyryanov, S.S.; Kruzhalov, A.V.; Ryabukhin, O.V.; Neshov, F.G. (FGAOU VPO #Left-Pointing Double Angle Quotation Mark#Ural'skij Federal'nyj Univ. imeni Pervogo Prezidenta Rossii B.N. El'tsina#Right-Pointing Double Angle Quotation Mark#, Ekaterinburg (RU))

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Zyryanov, S.S.; Kruzhalov, A.V.; Ryabukhin, O.V.; Neshov, F.G. (FGAOU VPO #Left-Pointing Double Angle Quotation Mark#Ural'skij Federal'nyj Univ. imeni Pervogo Prezidenta Rossii B.N. El'tsina#Right-Pointing Double Angle Quotation Mark#, Ekaterinburg (RU))

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Egorov, V.K.; Egorov, E.V. (IPTM RAN, Chernogolovka (RU)); Afanas'ev, M.S. (IREh RAN, Moscow (RU))

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Effect of ion irradiation of graphite surface on its wetting by liquid copper S36

Pashkov, I.N. (NITU MISiS, Moscow (RU)); Ligacheva, E.A. (RGTU- MATI im. K.Eh. Tsiolkovskogo, Moscow (RU)); Golosov, E.V. (NIU - Belgorodskoj Gosudarstvennyj Univ., Belgorod (RU)); Ligachev, A.E. (Inst. Obshchej Fiziki im. A.M. Prokhorova RAN, Moscow (RU)); Emlin, R.D. (Inst. Ehlektrofiziki UrO RAN, Ekaterinburg (RU))

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S75;S36

Shemukhin, A.A. (NIIYaF MGU, Moscow (RU)); Chernysh, V.S.; Balakshin, Yu.V. (MGU imeni M.V. Lomonosova, Fizicheskij Fakul'tet, Moscow (RU))

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Andrianova, N.N.; Borisov, A.M.; Vostrikov, V.G.; Mashkova, E.S.; Petrov, D.A.; Tkachenko, N.V. (Nauchno-Issledovatel'skij Inst. Yadernoj Fiziki imeni D.V. Skobel'tsyna, MGU imeni M.V. Lomonosova, Moscow (RU)); Bejlina, N.Yu.; Chernenko, D.N.; Chernenko, N.M. (OAO #Left-Pointing Double Angle Quotation Mark#NIIgrafit#Right-Pointing Double Angle Quotation Mark#, Moscow (RU))

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Ruzibaeva, M.K.; Umirzakov, B.E. (Inst. Ionno-Plazmennyykh i Lazernyykh Tekhnologij AN RUz, Tashkent (UZ))

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Isakhanov, Z.A.; Kadyrov, T.; Khalmatov, A.S.; Abduvaitov, A.A. (Inst. Ionno-Plazmennykh i Lazernykh Tekhnologij AN RUz, Tashkent (UZ))

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Umirzakov, B.E.; Danaev, S.B.; Tashatov, A.K.; Kholmukhamedova, V.Kh. (Tashkentskij Gosudarstvennyj Tekhnicheskij Univ., Tashkent (UZ))

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Rumi, D.S. (NTP #Left-Pointing Double Angle Quotation Mark#PROTON#Right-Pointing Double Angle Quotation Mark#, Tashkent (UZ)); Nimatov, S.Zh. (IIPiLT AN RUz, Tashkent (UZ))

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Kulikauskas, V.S.; Petrov, D.V.; Shemukhin, A.A. (NIIYaF im. D.V. Skobel'tsyna, MGU im. M.V. Lomonosova, Moscow (RU)); Sarajkin, V.V.; Trifonov, A.Yu. (NIIFP, Zelenograd, Moscow (RU)); Privezentsev, V.V. (FTIAN RAN, Moscow (RU))

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*Khripunov, B.I.; Danelyan, L.S.; Kojdan, V.S.; Latushkin, S.T.; Petrov, V.B.; Ryazanov, A.I.;
Unezhev, V.N. (NITs #Left-Pointing Double Angle Quotation Mark#Kurchatovskij Inst.#Right-
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*Podsvirov, O.A.; Churaev, D.V. (Gosudarstvennyj Politekhnikeskij Univ., Sankt-Peterburg
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Bednyakov, A.A. (NIIYaF MGU, Moscow (RU))

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Zastavnoj, A.V.; Korol', V.M. (NII Fiziki YuFU, Rostov-na-Donu (RU))

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